May 31, 1988

Docket No. 50-244

Mr. Roger W. Kober, Vice President
Electric and Steam Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Mr. Kober:

SUBJECT: ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE NO. DPR-18

The Commission has issued the enclosed Amendment No. 27 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated, September 23, 1987.

The amendment revises the Technical Specifications to change the reporting requirements for iodine spiking and eliminate the requirement for plant shutdown if iodine activity limits are exceeded for 800 hours in a 12 month period.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/S/

Carl Stahle, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

8806090283 880531
PDR ADOCK 05000244
P PDR

Enclosures:
1. Amendment No. 27 to
   License No. DPR-18
2. Safety Evaluation

cc w/enclosures:
See next page

*See previous concurrence

OFC	:PDI-3	:PDI-3	:OGC	:PDI-3/DIR	:	:	: :
NAME	:MRushbrook	:CStahle:ck	:	:RWessman	:	:	: :
DATE	:4/08/88	:5/21/88	:5/24/88	: 5/31/88	:	:	:

OFFICIAL RECORD COPY

Amendment No. 27 to Facility Operating Licensee DPR-18 - R. E. Ginna Nuclear Power Plant

DISTRIBUTION:
Docket File 50-244
NRC PDR
Local PDR
PDI-3 R/F
CStahle
MRushbrook
OGC-Bethesda
DHagan
EJordan
JPartlow
EButcher
TBarnhart (4)
Wanda Jones
Tech. Reveiw Br.
ACRS (10)
GPA/PA
ARM/LFMB

Docket No. 50-244

Mr. Roger W. Kober, Vice President
Electric and Steam Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Mr. Kober:

SUBJECT: ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE NO. DPR-18

The Commission has issued the enclosed Amendment No. to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated, September 23, 1987.

The amendment revises the Technical Specifications to change the reporting requirements for iodine spiking and eliminate the requirement for plant shutdown if iodine activity limits are exceeded for 800 hours in a 12 month period.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Carl Stahle, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

Enclosures:
1. Amendment No. to
License No. DPR-18
2. Safety Evaluation

cc w/enclosures:
See next page

*See previous concurrence

OFC	:PDI-3	:PDI-3	:OGC	:PDI-3/DIR	:	:	:
NAME	:MRushbrook	:CStahle:ck	:SH Lewis	:RWessman	:	:	:
DATE	:4/08/88	:5/10/88	:4/26/88	:	:	:	:

OFFICIAL RECORD COPY

5/24/88
No legal objection, subject to notations on SE and amendment and to confirming call from PM. Complete 5/24/88

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

MAY 31 1988

Docket No. 50-244

Mr. Roger W. Kober, Vice President
Electric and Steam Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Mr. Kober:

SUBJECT: ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE NO. DPR-18

The Commission has issued the enclosed Amendment No. 27 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated, September 23, 1987.

The amendment revises the Technical Specifications to change the reporting requirements for iodine spiking and eliminate the requirement for plant shutdown if iodine activity limits are exceeded for 800 hours in a 12 month period.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Carl Stahle

Carl Stahle, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

Enclosures:
1. Amendment No. 27 to
   License No. DPR-18
2. Safety Evaluation

cc w/enclosures:
See next page

cc:

Harry H. Voigt, Esquire
LeBoeuf, Lamb, Leiby and MacRae
1333 New Hampshire Avenue, N.W.
Suite 1100
Washington, D.C. 20036

Ezra Bialik
Assistant Attorney General
Environmental Protection Bureau
New York State Department of Law
2 World Trade Center
New York, New York 10047

Resident Inspector
R.E. Ginna Plant
c/o U.S. NRC
1503 Lake Road
Ontario, New York 14519

Stanley B. Klimberg, Esquire
General Counsel
New York State Energy Office
Agency Building 2
Empire State Plaza
Albany, New York 12223

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Supervisor of the Town of Ontario
1850 Ridge Road
Ontario, New York 14519

Ms. Donna Ross
Division of Policy Analysis & Planning
New York State Energy Office
Agency Building 2
Empire State Plaza
Albany, New York 12223

Mr. Bruce A. Snow, Superientendent
Nuclear Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, N.Y. 14649-0001

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION
DOCKET NO. 50-244
R. E. GINNA NUCLEAR POWER PLANT
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 27
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated September 23, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

8806090288 880531
PDR ADOCK 05000244
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 27 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 31, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 27

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE	INSERT
3.1-21	3.1-21
3.1-22	3.1-22*
6.9-6	6.9-6
6.9-7	6.9-7

*No change - repositioned on page

3.1.4 Maximum Coolant Activity Specifications

3.1.4.1 Whenever the reactor is critical or the reactor coolant average temperature is greater than 500°F:

a. The total specific activity of the reactor coolant shall not exceed $84/\bar{E}$ μCi/gm, where $\bar{E}$ is the average beta and gamma energies per disintegration in Mev.

b. The I-131 equivalent of the iodine activity in the reactor coolant shall not exceed 0.2 μCi/gm.

c. The I-131 equivalent of the iodine activity on the secondary side of a steam generator shall not exceed 0.1 μCi/gm.

3.1.4.2 If the limit of 3.1.4.1.a is exceeded, then be subcritical with reactor coolant average temperature less than 500°F within 8 hours.

3.1.4.3 a. If the I-131 equivalent activity in the reactor coolant exceeds the limit of 3.1.4.1.b but is less than the allowable limit shown on Figure 3.1.4-1, operation may continue for up to 168 hours. If the I-131 equivalent activity in the reactor coolant exceeds the limit of 3.1.4.1.b for more than 500 hours in any consecutive 6-month period, then prepare and submit a report to the Commission pursuant to Specification 6.9.2.

The reactor may be taken critical or reactor coolant average temperature may be increased above a 500°F with the I-131 equivalent activity greater than the limit of 3.1.4.1.b as long as the provisions of this paragraph are met.

b. If the I-131 equivalent activity exceeds the limit of 3.1.4.1.b for more than 168 hours during one continuous time interval or exceeds the limit shown on Figure 3.1.4-1, be subcritical with reactor coolant average temperature less than 500°F within 8 hours.

c. If the I-131 equivalent activity exceeds the limit of 3.1.4.1.b, then perform sampling and analysis as required by Table 4.1-4, item 4a, until the activity is reduced to less than the limit of 3.1.4.1.b.

3.1.4.4 If the limit of 3.1.4.1.c is exceeded, then be at hot shutdown within 8 hours and in cold shutdown within the following 32 hours.

Basis:

The total activity limit for the primary system corresponds to operation with the plant design basis of 1% fuel defects.(1) Radiation shielding and the radioactive waste disposal systems

6.9.2 Unique Reporting Requirements

6.9.2.1 Annually: Results of required leak test performed on sources if the tests reveal the presence of 0.005 microcurie or more of removable contamination.

6.9.2.2 Annually: A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man-rem exposure according to work and job functions, e.g., reactor operations and surveillance, in-service inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions. (NOTE: This tabulation supplements the requirements of Section 20.407 of 10CFR Part 20.)

6.9.2.3 Annually: The results of specific activity analysis in which the primary coolant exceeded the limits of Specification 3.1.4.1.a and b. The following information shall be included: (1) Reactor power history starting 48 hours prior to the first sample in which the limit was exceeded; (2) Results of the last isotopic analysis for radioiodine performed prior to exceeding the limit,

results of analyses while the limit was exceeded and results of one analysis after the radioiodine activity was reduced to less than the limit. Each result should include the date and time of sampling and the radioiodine concentrations; (3) Clean-up system flow history starting 48 hours prior to the first sample in which the limit was exceeded; (4) Graph of the I-131 concentration and one other radioiodine isotope concentration as a function of time for the duration of the specific activity above the steady-state level; and (5) The time duration when the specific activity of the primary coolant exceeded the radioiodine limit.

6.9.2.4 Reactor Overpressure Protection System Operation

In the event either the PORVs or the RCS vent(s) are used to mitigate a RCS pressure transient, a Special Report shall be prepared and submitted to the Commission within thirty days. The report shall describe the circumstances initiating the transient, the effect of the PORVs or vent(s) on the transient and any other corrective action necessary to prevent recurrence.

6.9.2.5 Special reports shall be submitted to the Director of the NRC Regional Office listed in Appendix D, 10CFR Part 20, with a copy to the Director, Office of Inspection and Enforcement, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555 within the time period specified for each report.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 27 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

INTRODUCTION

1.0 On September 23, 1987, the Rochester Gas and Electric Corporation requested an amendment to the Technical Specifications to change the reporting requirements for primary coolant iodine spiking and the elimination of the 800 hour shutdown requirement.

2.0 EVALUATION

Revised guidance as to the reporting and 800 hour shutdown requirements was provided in Generic Letter 85-19, dated September 27, 1985, "Reporting Requirements on Primary Coolant Iodine Spikes". The letter stated that (1) the reporting of primary coolant iodine spiking activity levels can be reduced from a short-term report (i.e., Special Report or Licensee Event Report) to an item to be evaluated in the Annual Report and (2) existing shutdown requirements based on exceeding the primary coolant specific activity limits for an accumulated period of over 800 hours were no longer necessary. The change in these requirements is based on an improvement in the quality of nuclear fuel over the past 10 years; and the fact that appropriate actions would be initiated long before approaching the limit specified in the Technical Specification. An example is that a Technical Specification provision requires the reactor to be subcritical within 8 hours if the primary coolant iodine level exceeds a certain value after 168 hours of continous operation. Also, Generic Letter 85-19 provided guidance on the reporting of information. The proposed change to the Technical Specification on reporting is revised in accordance with the Generic Letter. The staff finds the revisions to be acceptable.

8806090290 880531
PDR ADOCK 05000244
P PDR

3.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to use of a facility component located within the restricted area as deferred in 10 CFR Part 20. This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding.

This amendment also involves a change in the recordkeeping, reporting, or administrative procedures or requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has evaluated the licensee's request to change the reporting requirements related to primary coolant specific activity levels, specifically the reporting of primary coolant iodine spikes can be reduced from a short-term report to an annual report. The Annual Report requirements are specified to more clearly designate the results to be included from the specific activity analysis than was previously required.

The staff has concluded, based on the considerations discussed above, that; (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations; and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

5.0 Acknowledgement

Prinicipal Contributors: Carl Stahle

Dated: May 31, 1988