

November 17, 1989

Docket No. 50-244

Dr. Robert C. Mecredy
General Manager, Nuclear Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Dr. Mecredy:

SUBJECT: ISSUANCE OF AMENDMENT NO.38 TO FACILITY OPERATING LICENSE NO.
DPR-18 - R. E. GINNA NUCLEAR POWER PLANT (TAC NO.74542)

The Commission has issued the enclosed Amendment No.38 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated August 30, 1989.

This amendment revises the requirements of the Technical Specifications in order to make an administrative change in title from Vice President, Production and Engineering to Senior Vice President, Production and Engineering.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

15/
Allen Johnson, Project Manager
Project Directorate I-3
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 38 to License No. DPR-18
2. Safety Evaluation

cc w/enclosures:

See next page

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OFC	:PDI-3/LA	:PDI-3/PM	:OGC	:PDI-3/D	:	:	:
NAME	:MRusbrook	:AJohnson	:	:RHessman	:	:	:
DATE	:11/3/89	:11/2/89	:11/6/89	:11/17/89	:	:	:

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3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

15/
Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 17, 1989

OFC	: PDI-3/LA	: PDI-3/PM	: OGC	: PDI-3/D	:	:	:
NAME	: MRushbrook	: AJohnson	:	: RWessman	:	:	:
DATE	: 11/2/89	: 11/2/89	: 11/17/89	:	:	:	:

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AMENDMENT NO.38 TO DPR-18 - R. E. GINNA NUCLEAR POWER PLANT DATED November 17, 1989

DISTRIBUTION:

~~Docket File~~ 50-244

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

November 17, 1989

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Rochester Gas & Electric Corporation
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1. Amendment No. 38 to
License No. DPR-18
2. Safety Evaluation

cc w/enclosures:
See next page

Dr. Robert C. Mecredy

-2-

November 17, 1989

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 38
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Rochester Gas and Electric Corporation (the licensee) dated August 30, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 38, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 17, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 38

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

Remove

Page 6.2-1
Page 6.5-3
Page 6.5-4a
Page 6.5-10
Page 6.5-11
Page 6.6-1
Page 6.7-1

Insert

Page 6.2-1
Page 6.5-3
Page 6.5-4a
Page 6.5-10
Page 6.5-11
Page 6.6-1
Page 6.7-1

6.2

ORGANIZATION

6.2.1

Onsite and Offsite Organization

An onsite and an offsite organization shall be established for unit operation and corporate management. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility and communication shall be established and defined from the highest management levels through intermediate levels to and including all Plant management positions. Those relationships shall be documented and updated, as appropriate, in the form of organization charts. These organization charts will be documented in the UFSAR and updated in accordance with 10CFR50.71.
- b. The Senior Vice President, Production and Engineering, shall have corporate responsibility for overall Plant nuclear safety, and shall take any measures needed to assure acceptable performance of the staff in operating, maintaining, and providing technical support in the Plant so that continued nuclear safety is assured.
- c. The Plant Manager, Ginna Station shall have responsibility for overall unit operation and shall have control over those resources necessary for safe operation and maintenance of the Plant.

RESPONSIBILITIES

6.5.1.6 The PORC shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Plant Manager, Ginna Station to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Senior Vice President, Production and Engineering, and to the Chairman of the Nuclear Safety Audit and Review Board.
- f. Review of facility operations to detect potential safety hazards.
- g. Performance of special reviews and investigations and reports thereon as requested by the Chairman of the Nuclear Safety Audit and Review Board.

- c. Provide immediate written notification to the Senior Vice President, Production and Engineering, and the Nuclear Safety Audit and Review Board of disagreement between the PORC and the Plant Manager, Ginna Station; however, the Plant Manager, Ginna Station shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

- 6.5.1.8 The PORC shall maintain written minutes of each meeting and copies shall be provided to the Senior Vice President, Production and Engineering, the Chairman of the Nuclear Safety Audit and Review Board, and such others as the Chairman may designate.

AUDITS (Continued)

- g. The Facility Fire Protection Program and implementing procedures at least once per two years.
- h. An independent fire protection and loss prevention program inspection and audit performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- i. An inspection and audit of the fire protection and loss prevention program performed by non-licensee personnel at least once per 36 months. The personnel may be representatives of ANI, an insurance brokerage firm, or other qualified individuals.
- j. The radiological environmental monitoring program and the results thereof at least once per 12 months.
- k. The Offsite Dose Calculation Manual and implementing procedures at least once per 24 months.
- l. The Process Control Program and implementing procedures at least once per 24 months.
- m. Any other area of facility operation considered appropriate by the NSARB or the Senior Vice President, Production and Engineering.

AUTHORITY

- 6.5.2.9 a. The Chairman of the Nuclear Safety Audit and Review Board is responsible to the President on all activities for which the Review Board is responsible.
- b. The NSARB shall report to and advise the Senior Vice President, Production and Engineering, on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

- 6.5.2.10 Records of NSARB activities shall be prepared, approved, and distributed as indicated below:
- a. Minutes shall be recorded of all meetings of this Board. Copies of the minutes shall be forwarded within 14 days following each meeting to the Corporate Chairman of the Board, Senior Vice President, Production and Engineering and such others as the Chairman of the NSARB may designate.
- b. Reports of reviews encompassed by Section 6.5.2.7 e,f,g and h above, shall be prepared, approved and forwarded to the Senior Vice President, Production and Engineering within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Senior Vice President, Production and Engineering and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6

REPORTABLE EVENT ACTION

- 6.6.1 The following actions shall be taken for Reportable Events:
- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
 - b. Each Reportable Event shall be reviewed by the PORC and the results of this review shall be submitted to the NSARB and the Senior Vice President, Production and Engineering.

6.7

SAFETY LIMIT VIOLATION

6.7.1

The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR Section 50.36(c)(1)(i) shall be complied with immediately.
- b. The Safety Limit violation shall be reported to the Senior Vice President, Production and Engineering, to the NSARB and to the Commission immediately.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NSARB, and the Senior Vice President, Production and Engineering within two weeks of the violation.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO.38 TO FACILITY OPERATING LICENSE NO. DPR-18
ROCHESTER GAS AND ELECTRIC CORPORATION
R. E. GINNA NUCLEAR POWER PLANT
DOCKET NO. 50-244

1. INTRODUCTION

By letter dated August 30, 1989, the Rochester Gas and Electric Corporation (the licensee) proposed changes to Appendix A of the operating license for the R. E. Ginna Nuclear Power Station. The proposed changes affect Section 6.0, and reflect a new title given to a key managerial position. The staff has reviewed the submittal and has prepared the following evaluation.

2.0 EVALUATION

The revised Technical Specification addresses title changes for the corporate positions having responsibility for overall plant safety. Specifically, the following changes are:

- (1) Vice President, Production and Engineering to Senior Vice President, Production and Engineering.

This amendment does not involve a significant reduction in the margins of safety. The revision does not in any way alter RG&E's commitment to maintaining a management structure that contributes to the safe operation and maintenance of Ginna Station. A single corporate officer (Senior Vice President) is responsible for plant safety and will communicate and direct the lines of authority and responsibility for safe nuclear plant operation. This officer continues to report directly to the President. A single individual is responsible for overall safe unit operation and maintenance (Plant Manger).

The staff has determined that the proposed changes in the technical specifications are acceptable since there are no changes to the function or responsibilities of the affected management position.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to recordkeeping reporting or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register on September 20, 1989 (54 FR 38767). The Commission consulted with the State of New York. No public comments were received, and the State of New York did not have any comments.

On the basis of the considerations discussed above, the staff concludes that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operating in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: A. Johnson

Dated: November 17, 1989