

July 3, 1990

Docket No. 50-244

Dr. Robert C. Mecredy  
General Manager, Nuclear Production  
Rochester Gas & Electric Corporation  
89 East Avenue  
Rochester, New York 14649

Dear Dr. Mecredy:

SUBJECT: CORRECTION TO AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO.  
DPR-18

By letter, dated May 30, 1989, we issued Amendment No. 37 to Facility Operating License No. DPR-28 for the R. E. Ginna Nuclear Power Plant.

Technical Specification Page 4.2-2 of Amendment No. 37 inadvertently deleted changes that had been issued with Amendment No. 35, dated April 24, 1989.

Please replace the erroneous page 4.2-2 with the new page 4.2-2 provided as an enclosure. Page 4.2-3 remains the same, and a new page 4.2-4 containing the remainder of the Basis from page 4.2-3 of Amendment No. 35 is provided as page 4.2-4.

Sincerely,

151

for Victor Nerses  
Allen Johnson, Project Manager  
Project Directorate I-3  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

Enclosures:

T.S. Pages 4.2-2 and 4.2-4

cc w/enclosures:

See next page

LA:PMI-3  
MRushbrook

7/3/90

PM:FDI-3  
AJohnson

7/3/90

ACTINGPD:I-3  
VNerses

7/3/90

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Document Name: CORRECTION TO AMEND. 37

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

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General Manager, Nuclear Production  
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Sincerely,

A handwritten signature in cursive script, reading "Allen Johnson", is written over the typed name.

Allen Johnson, Project Manager  
Project Directorate I-3  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

Enclosures:  
T.S. Pages 4.2-2 and 4.2-4

cc w/enclosures:  
See next page

CORRECTION TO AMENDMENT NO. 37 TO DPR-18 - R. E. GINNA NUCLEAR POWER PLANT  
DATED July 3, 1990

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Dr. Robert C. Mecredy      Ginna

July 3, 1990

cc:

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- 4.2.1.1 The inspection interval for Quality Group A components shall be ten year intervals of service commencing on January 1, 1970.
- 4.2.1.2 The inspection intervals for Quality Group B and C Components shall be ten year intervals of service commencing with May 1, 1973, January 1, 1980, 1990 and 2000, respectively.
- 4.2.1.3 The inspection intervals for the High Energy Piping Outside of Containment shall be ten year intervals of service commencing May 1, 1973, January 1, 1980, 1990 and 2000, respectively. The inspection program during each third of the first inspection interval provides for examination of all welds at design basis break locations and one-third of all welds at locations where a weld failure would result in unacceptable consequences. During each succeeding inspection interval, the program shall provide for an examination of each of the design basis break location welds, and each of the welds at locations where a weld failure would result in unacceptable consequences.
- 4.2.1.4 The inspection intervals for Steam Generator Tubes shall be specified in the "Inservice Inspection Program" for the applicable forty month period commencing with May 1, 1973.
- 4.2.1.4.a Steam generator tubes that have imperfections greater than 40% through wall, as indicated by eddy current, shall be repaired by plugging or sleeving.
- 4.2.1.4.b Steam generator sleeves that have imperfections greater than 30% through wall, as indicated by eddy current, shall be repaired by plugging.

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P PDC

The repair criteria of 4.2.1.4.a and 4.2.1.4.b are based on the requirements of USNRC Regulatory Guide 1.121, "Bases for Plugging Degraded PWR Steam Generator Tubes" as implemented by RG&E (Reference 1). This guide describes a method acceptable to the NRC staff for establishing the limiting safe conditions of tube degradation of steam generator tubing. The repair criteria is based on structural allowances, an allowance for eddy current measurement error and an allowance for degradation during the operating period. These allowances are added together to determine the repair criteria which is typically 40% for steam generator tubes. Based on calculations the appropriate sleeve plugging limit is a 42% thru wall defect. In order to allow for conservatism, a 30% plugging limit for sleeves will be utilized.

Reference 1: "Steam Generator Rapid Sleaving Program Design Verification Report", R.E. Ginna Nuclear Power Plant, August 1982.