March 8, 1985

Docket No.: 50-244
LS05-85- 03-006

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Mr. Roger W. Kober, Vice President
Electric and Steam Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Mr. Kober:

SUBJECT: TEMPORARY CLOSURE PLATE

Re: R. E. Ginna Nuclear Power Plant

The Commission has issued the enclosed Amendment No. 2 to Facility Operating License No. DPR-18 for the R.E. Ginna Nuclear Power Plant. This amendment is in response to your application dated January 25, 1985.

The amendment changes the Technical Specifications to authorize the use of a temporary closure plate in place of the equipment hatch or equipment door during refueling operations.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on February 5, 1985 (50 FR 5020). No public comments or requests for hearing were received.

A copy of our related Safety Evaluation is also enclosed. This action will appear in the Commission's Monthly Notice publication in the Federal Register.

Sincerely,

Original signed by
John A. Zwolinski, Chief
Operating Reactors Branch #5
Division of Licensing

Enclosures:
1. Amendment No. 2 to License No. DPR-18
2. Safety Evaluation

cc w/enclosures:
See next page

SEQ1
1/1 Ex(07)

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 2
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated January 25, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 2 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John A. Zwolinski, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 8, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 2

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE	INSERT
3.8-1	3.8-1
3.8-5	3.8-5

3.8 REFUELING

Applicability

Applies to operating limitations during refueling operations.

Objective

To ensure that no incident could occur during refueling operations that would affect public health and safety.

Specification

3.8.1 During refueling operations the following conditions shall be satisfied.

a. The equipment door, or a closure plate that restricts air flow from the containment, and at least one personnel door in the equipment door or closure plate and in the personnel air lock shall be properly closed. In addition, all automatic containment isolation valves shall be operable or at least one valve in each line shall be locked closed.

b. Radiation levels in the containment shall be monitored continuously.

c. Core subcritical neutron flux shall be continuously monitored by at least two source range neutron monitors, each with continuous visual indication in the control room and one with audible indication in the containment and control room available whenever core geometry is being changed. When core geometry is not being changed at

provided on the lifting hoist to prevent movement of more than one fuel assembly at a time. The spent fuel transfer mechanism can accommodate only one fuel assembly at a time. In addition interlocks on the auxiliary building crane will prevent the trolley from being moved over stored racks containing spent fuel.

The operability requirements for residual heat removal loops will ensure adequate heat removal while in the refueling mode. The requirement for 23 feet of water above the reactor vessel flange while handling fuel and fuel components in containment is consistent with the assumptions of the fuel handling accident analysis.

The analysis[4] for a fuel handling accident inside containment establishes acceptable offsite limiting doses following rupture of all rods of an assembly operated at peak power. No credit is taken for containment isolation or effluent filtration prior to release. Requiring closure of the containment openings and penetrations establishes additional margin for the fuel handling accident and establishes a seismic envelope to protect against seismic events during refueling.

References

(1) FSAR - Section 9.5.2

(2) Reload Transition Safety Report, Cycle 14

(3) FSAR - Section 9.3.1

(4) Updated Final Safety Analysis Report, Section 15.7

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 2 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

In a letter dated January 25, 1985, Rochester Gas and Electric Corporation proposed an amendment to Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. The amendment involves a change to Technical Specification 3.8.1(a) (refueling) which would authorize the use of a temporary, specially designed closure plate (with sealed penetrations) in place of the equipment hatch (equipment door) during refueling. The equipment hatch is described in Section 3.8.1.5.4 of the R. E. Ginna updated FSAR. The temporary closure plate is required to facilitate steam generator and other outage maintenance activities while maintaining primary containment integrity during refueling operations.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on February 5, 1985 (50 FR 5020). No public comments or requests for hearing were received.

2.0 EVALUATION

The R. E. Ginna Nuclear Power Plant includes an equipment door approximately fourteen (14) feet in diameter, to permit transfer of large components into and out of the containment. It is constructed of welded steel and has a double-gasketed flange and bolted dished door and is designed to withstand design basis accident pressure. Technical Specification 3.8 specifies the required operating limitations during refueling operations. These requirements ensure that the release of radioactivity within the containment will be restricted from leakage to the environment. The radioactive material released from a postulated fuel handling accident would be retained within the building due to the lack of containment pressurization potential while in the refueling mode coupled with the penetration integrity requirements.

The specially fabricated temporary closure plate proposed to be utilized during refueling outages will contain sealed penetrations for temporary services and a personnel door that will provide emergency egress. It will be seismically designed, but will not be designed to withstand high pressure. The temporary closure plate will perform the required functions, i.e., provide the required margin of safety for a fuel handling accident by restricting direct communication with the environment and provide a seismic envelope to restrict the potential escape of radioactivity resulting from a postulated seismic event during refueling. The closure plate will conform to the guidance of the NRC Standard Technical Specifications for refueling operations (Section 3/4.9.4) which require that the equipment door be held in place by a minimum of four (4) bolts and that penetrations providing direct access from the containment atmosphere to the outside atmosphere be closed or capable of automatic closure. The intent is that the direct air flow to the environment be restricted but that the containment need not be in a condition to mitigate design basis accidents.

Running temporary service lines through the sealed penetrations will replace the current practice of running them through an open personnel door within the equipment door or attaching a special closure to the personnel door with appropriately sealed penetrations. This will improve personnel safety.

Based on the staff's review, the staff concludes that the licensee's proposal will provide adequate containment integrity during refueling operations. Therefore, the staff finds the proposed modification and Technical Specification change acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

5.0 ACKNOWLEDGEMENT

Charles L. Miller prepared this Safety Evaluation.

Dated: March 8, 1985