



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 1, 1985

Docket No. 50-244
LS05-85-04-001

Mr. Roger W. Kober, Vice President
Electric and Steam Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Mr. Kober:

SUBJECT: TECHNICAL SPECIFICATION CHANGES TO REMOVE BATTERY CHARGER
DESCRIPTIVE INFORMATION

Re: R. E. Ginna Nuclear Power Plant

The Commission has issued the enclosed Amendment No. 3 to Facility Operating License No. DPR-18 for the R.E. Ginna Nuclear Power Plant. This amendment is in response to your application dated December 5, 1984.

The amendment consists of changes to the Technical Specifications responsive to your request dated December 5, 1984. The changes eliminate descriptive battery charger wording, specifically 75 and 150 ampere chargers. Prior to this change, it could have been construed that the 150 ampere battery charge rate required to satisfy the technical specifications could be provided only by 150 and 75 ampere battery chargers arranged as described.

Our review has revealed that:

- 10 CFR 50.71(e)(3)(ii) requires the licensee to maintain a Final Safety Analysis Report that includes this type of equipment description.
- The accepted Standard Technical Specifications for PWRs do not specify the capacity of individual battery chargers to satisfy the technical specification for battery charging rate.
- Removal of extraneous information simplifies these selected technical specifications and clarifies the safety requirements by focusing on battery recovery time, i.e., battery charge rate.

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Mr. Roger W. Kober

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For these reasons, we have concluded that the proposed technical specification changes are permitted administrative changes. We have also concluded that the planned battery and battery charger upgrading expected to be implemented during the 1985 Spring refueling outage presents a timely opportunity to make these changes. Accordingly, this amendment authorizes the changes you have proposed.

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on February 27, 1985 (50 FR 8004). No request for hearing and no comments were received. This action will appear in the Commission's monthly notice publication in the Federal Register.

Sincerely,
Original signed by
John A. Zwolinski, Chief
Operating Reactors Branch #5
Division of Licensing

Enclosure:
Amendment No. 3 to
License No. DPR-18
cc w/enclosure:
See next page

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Mr. Roger W. Kober

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April 1, 1985

cc

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 3
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated December 5, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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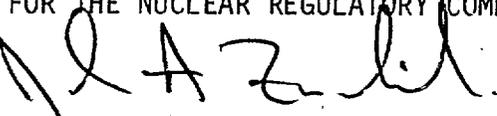
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 3, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John A. Zwolinski, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 1, 1985.

ATTACHMENT TO LICENSE AMENDMENT NO. 3

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.7-1

3.7-4

INSERT

3.7-1

3.7-4

3.7

AUXILIARY ELECTRICAL SYSTEMS

Applicability

Applies to the availability of electrical power for the operation of plant auxiliaries.

Objective

To define those conditions of electrical power availability necessary (1) to provide for safe reactor operation, and (2) to provide for the continuing availability of engineered safeguards.

Specification

- 3.7.1 The reactor shall not be maintained critical without:
- a. The 34.5 KV-4160 Volt station service transformer in service.
 - b. 480-volt buses, 14, 16, 17 and 18 energized.
 - c. 4160-volt buses 12A and 12B energized.
 - d. Two diesel generators operable with onsite supply of 10,000 gallons of fuel available.
 - e. Both batteries and both d.c. systems operable, and battery chargers with a capacity of at least 150 amps in service for each battery.
- 3.7.2 During reactor operation the requirements of 3.7.1 may be modified as follows:

maintain the containment pressure within the design value. The minimum diesel fuel oil inventory at all times is maintained to assure the operation of both diesels carrying design load of all the engineered safeguards equipment for at least 40 hours.⁽¹⁾ Commercial oil supplies and trucking facilities exist to assure deliveries within 8 hours.

Battery chargers with at least 150 amps capacity shall be in service for each battery so that the batteries will always be at full charge. This ensures that adequate dc power will be available.

The plant can be safety shut down without the use of offsite power since all vital loads (safety systems, instruments, etc.) can be supplied from the emergency diesel generators.

The two diesel generators, each capable of supplying safeguards loads, and the station auxiliary transformer provide three separate sources of power immediately available for operation of these loads. Thus, the power supply system meets the single failure criteria required of safety systems.⁽²⁾

References:

(1) FSAR - Section 8.2.1

(2) FSAR - Appendix 8A