UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

June 19, 1985

Docket No. 50-244
LS05-85-06-024

Mr. Roger W. Kober, Vice President
Electric and Steam Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Mr. Kober:

SUBJECT: INSERVICE INSPECTION START DATES

Re: R. E. Ginna Nuclear Power Plant

The Commission has issued the enclosed Amendment No. 5 to Facility Operating License No. DPR-18 for the R.E. Ginna Nuclear Power Plant. This amendment is in response to your application dated December 19, 1983.

The amendment revises the second and subsequent ten year interval start dates of the Quality Group B and C programs of the Inservice Inspection Program, and the program for High Energy Piping Outside Containment, to coincide with the interval of the Quality Group A program. The change also incorporates inspection intervals for the Inservice Pump and Valve Testing program with corresponding interval start dates for the second and subsequent intervals.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on April 25, 1984 (49 FR 17871). No public comments or requests for hearing were received.

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A copy of our related Safety Evaluation is also enclosed. This action will appear in the Commission's monthly notice publication in the Federal Register.

Sincerely,

Original signed by:

John A. Zwolinski, Chief
Operating Reactors Branch #5
Division of Licensing

Enclosures:
1. Amendment No. to License No. DPR-18
2. Safety Evaluation

cc w/enclosures:
See next page

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Initial revision to SE

SE01
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Mr. Roger W. Kober
Rochester Gas and Electric Corporation

R. E. Ginna Nuclear Power Plant

cc
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 5
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated December 19, 1983 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 5 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John A. Zwolinski, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 19, 1985.

ATTACHMENT TO LICENSE AMENDMENT NO. 5

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE	INSERT
4.2-1	4.2-1
4.2-2	4.2-2

4.2 Inservice Inspection

Applicability

Applies to the inservice inspection of Quality Groups A, B and C Components, High Energy Piping Outside of Containment and Steam Generator Tubes. It also applies to inservice pump and valve testing.

Objectives

To provide assurance of the continuing structural and operational integrity of the structures, components and systems in accordance with the requirements of 10 CFR 50.55a(g).

Specification

4.2.1 The inservice inspection program for Quality Groups A, B and C components, High Energy Piping Outside of Containment and Steam Generator tubes shall be in accordance with Appendix B of the Ginna Station Quality Assurance Manual. The inservice pump and valve testing program shall be in accordance with Appendix C of the Ginna Station Quality Assurance Manual. These in-service inspection programs shall define the specific requirements of the edition and Addenda of the ASME Boiler and Pressure Vessel Code, Section XI, which are applicable for the forty month period of the ten year inspection interval. The programs' ten year inspection intervals shall be based on the following commencing dates.

4.2.1.1 The inspection interval for Quality Group A components shall be ten year intervals of service commencing on January 1, 1970.

4.2.1.2 The inspection intervals for Quality Group B and C Components shall be ten year intervals of service commencing May 1, 1973, January 1, 1980, 1990 and 2000, respectively.

4.2.1.3 The inspection intervals for the High Energy Piping Outside of Containment shall be ten year intervals of service commencing May 1, 1973, January 1, 1980, 1990

and 2000, respectively. The inspection program during each third of the first inspection interval provides for examination of all welds at design basis break locations and one-third of all welds at locations where a weld failure would result in unacceptable consequences. During each succeeding inspection interval, the program shall provide for an examination of each of the design basis break location welds, and each of the welds at locations where a weld failure would result in unacceptable consequences.

4.2.1.4 The inspection intervals for Steam Generator Tubes shall be specified in the "Inservice Inspection Program" for the applicable forty month period commencing with May 1, 1973.

4.2.1.5 Inservice Inspection of ASME Code Class 1, Class 2 and Class 3 components (Quality Groups A, B and C) shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).

4.2.1.6 The inspection interval for the Inservice Pump and Valve Testing Program shall be ten year intervals commencing with January 1, 1981, 1990 and 2000.

Basis:

The inservice inspection programs provide assurance for the continued structural integrity of the structures, components and systems of Ginna Station. The programs comply with the ASME Boiler and Pressure Vessel Code Section XI "Rules for Inservice Inspection of Nuclear Power Plant Components" as practicable, with due consideration to the design and physical access of the structures, components and systems as manufactured and constructed. This compliance will constitute an acceptable basis for satisfying the requirementsof General Design Criterion 32, Appendix A of 10 CFR Part 50 and the requirements of Section 50.55a, paragraph g of 10 CFR Part 50.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 5 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

10 CFR 50.55a(g)(4)(ii) requires that licensees update their pump and valve Inservice Testing (IST) programs to a newer edition of Section XI of the ASME Code (the Code) each 10 years based on the date of facility commercial operation. By letter dated December 19, 1983, Rochester Gas and Electric Corporation (the licensee) transmitted to the NRC a proposed Technical Specification change to permit an adjustment of the 10-year interval start date for inservice inspection at Ginna Unit No. 1. The purpose of the change is to allow inspection and testing programs to coincide with a common interval start date, thereby improving administrative controls. The change also incorporates inspection intervals for the Inservice Pump and Valve Testing Program; with corresponding start dates for the second and subsequent intervals.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on April 25, 1984 (49 FR 17871). No public comments or requests for hearing were received.

2.0 EVALUATION

The staff has reviewed the proposed Technical Specification changes in section 4.2. Section 50.55a(g) of 10 CFR Part 50 requires that inservice inspection programs be updated to later editions of Section XI every 10 years commencing with the date of facility commercial operation, or July 1, 1970 for Ginna. The licensee has requested to use January 1, 1980, (January 1, 1981 for Inservice Pump and Valve Testing), January 1, 1990, and January 1, 2000 as the interval start dates in lieu of July 1, 1980, 1990 and 2000. Section XI of the 1977 edition of the ASME Boiler and Pressure Vessel Code (IWA-2400 Inspection Intervals) states that "each inspection interval may be decreased or extended (but not cumulatively) by as much as one year." The Technical Specification changes requested have new dates that fall within this time frame and are acceptable. The change in start dates will not affect the required examinations of piping and components nor decrease the plant's safety margin. In addition, since Subsections IWP and IWV of Section XI of the ASME Code have not historically experienced extensive changes over such a short period of time, the change is considered nominal.

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3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change to the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

5.0 ACKNOWLEDGEMENT

G. Johnson, J. Page and C. Miller prepared this Safety Evaluation.

Dated: June 19, 1985.