

September 23, 1988

Docket No. 50-244

Mr. Robert C. Mecredy, Manager,
Nuclear Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Mr. Mecredy:

SUBJECT: ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE NO. DPR-18
(TAC #45137)

The Commission has issued the enclosed Amendment No. 30 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant in response to your application dated September 24, 1987 as supplemented on May 3, 1988.

The amendment revises the Technical Specifications to incorporate the requirements for the Reactor Vessel Level Indication System.

A copy of our related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's, next regular biweekly Federal Register notice.

Sincerely,

IS
Victor Nerses
for Carl Stahle, Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

Enclosures:

1. Amendment No. 30 to License No. DPR-18
2. Safety Evaluation

cc w/enclosures:
See next page

| | | | | | | | |
|------|------------------|------------------|-------------------|------------------------|---|---|---|
| OFC | : PD#1-3 | : PD#I-3 | : OGC <i>Rock</i> | : DIRPD#I-3 <i>200</i> | : | : | : |
| NAME | : MRusbrook | : CStahle:ck | : RBochmann | : RWessman | : | : | : |
| DATE | : <i>9/17/88</i> | : <i>9/19/88</i> | : <i>9/15/88</i> | : <i>9/20/88</i> | : | : | : |

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P PNU

m.m.

CPH

AMENDMENT NO. 30 TO FACILITY OPERATING LICENSE NO. DPR-18 - R. E. Ginna

DISTRIBUTION:

Docket File 50-244 ←

NRC PDR

LPDR

PD I-3 Reading

S. Varga

B. Boger

C. Stahle

M. Rushbrook

RWessman

OGC - 15-B-18 OWFN

D. Hagan

E. Jordan

Brian Grimes

T. Barnhart (4)

Wanda Jones

E. Butcher

ACRS (10) - P-213 Phillips

GPA/PA - 2G-5

ARM/LFMB - 2105 AR III

OFOI
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

September 23, 1988

Docket No. 50-244

Dr. Robert C. Mecredy, General Manager,
Nuclear Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Dr. Mecredy:

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(TAC #45137)

The Commission has issued the enclosed Amendment No. 30 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant in response to your application dated September 24, 1987 as supplemented on May 3, 1988.

The amendment revises the Technical Specifications to incorporate the requirements for the Reactor Vessel Level Indication System.

A copy of our related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's, next regular biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Victor Newses for Carl Stahle".

Carl Stahle, Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

Enclosures:

1. Amendment No. 30 to License No. DPR-18
2. Safety Evaluation

cc w/enclosures:

See next page

Robert C. Mecredy, General Manager, Nuclear Production.
Rochester Gas and Electric Corporation R. E. Ginna Nuclear Power Plant

cc:

Harry H. Voigt, Esquire
LeBoeuf, Lamb, Leiby and MacRae
1333 New Hampshire Avenue, N.W.
Suite 1100
Washington, D.C. 20036

Mr. Bruce A. Snow, Superintendent
Nuclear Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, N.Y. 14649-0001

Ezra Bialik
Assistant Attorney General
Environmental Protection Bureau
New York State Department of Law
2 World Trade Center
New York, New York 10047

Charlie Donaldson, Esq.
Assistant Attorney General
New York Department of Law
120 Broadway
New York, New York 10271

Resident Inspector
R.E. Ginna Plant
c/o U.S. NRC
1503 Lake Road
Ontario, New York 14519

Stanley B. Klimberg, Esquire
General Counsel
New York State Energy Office
Agency Building 2
Empire State Plaza
Albany, New York 12223

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Supervisor of the Town of Ontario
1850 Ridge Road
Ontario, New York 14519

Ms. Donna Ross
Division of Policy Analysis & Planning
New York State Energy Office
Agency Building 2
Empire State Plaza
Albany, New York 12223



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R.E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 30
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated September 24, 1987 as supplemented on May 3, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 30, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 23, 1988

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

15/
Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 30, 1988

| | | | | | | | |
|------|--------------|-----------|------------------------|-------------|---|---|---|
| OFC | : PD#1-3 | : PD#1-3 | : OGC- Rock | : DIRPD#1-3 | : | : | : |
| NAME | : MRushbrook | : CStante | : RBachmann | : RWessman | : | : | : |
| DATE | : 9/27/88 | : 9/19/88 | : 9/15/88 | : 9/20/88 | : | : | : |

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ATTACHMENT TO LICENSE AMENDMENT NO.30

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A as follows:

REMOVE

page 3.5-16
page 4.1-7

INSERT

page 3.5-16
page 4.1-7

TABLE 3.5-3
Accident Monitoring Instrumentation

| | TOTAL REQUIRED NO. OF <u>CHANNELS</u> (7) | MINIMUM CHANNELS <u>OPERABLE</u> (7) |
|---|--|--|
| 1. Pressurizer Water Level (1) | 2 | 1 |
| 2. Auxiliary Feedwater Flow Rate (2)(3) | 2/steam generator | 1/steam generator |
| 3. Steam Generator Water Level - Wide Range (3) | 1/steam generator | 1/steam generator |
| 4. Reactor Coolant System Subcooling Margin Monitor (4) | 2 | 1 |
| 5. Pressurizer PORV Position Indicator (5) | 2/Valve | 1/Valve |
| 6. PORV Block Valve Position Indicator (1) | 1/Valve | 0/Valve |
| 7. Pressurizer Safety Valve Position Indicator (5) | 2/Valve | 1/Valve |
| 8. Containment Pressure (8) | 2 | 1 |
| 9. Containment Water Level (Narrow Range, Sump A) | 1(6) | 1(6) |
| 10. Containment Water Level (Wide Range, Sump B) | 2 | 1 |
| 11. Core-Exit Thermocouples | 4/core quadrant | 2/core quadrant |
| 12. Reactor Vessel Level Indication System | 2 | 1 |

Notes

- (1) Emergency power for pressurizer equipment, NUREG-0737, item II.G.1.
- (2) Auxiliary feedwater system flow indication, NUREG-0737, item II.E.1.2.
- (3) Only 2 out of the 3 indications (two steam generator auxiliary feedwater flow and one wide-range steam generator level) are required to be operable, NUREG-0737, item II.E.1.2.
- (4) Instrumentation for detection of inadequate core cooling, NUREG-0737, item II.F.2.1.
- (5) Direct indication of relief and safety valve position, NUREG-0737, item II.D.3. Two channels include a primary detector and RTD as the backup detector.
- (6) Operation may continue with less than the minimum channels operable provided that the requirements of Technical Specification 3.1.5.1 are met.
- (7) See Specification 3.5.3 for required action.
- (8) Containment pressure monitor, NUREG-0737, item II.F.1.4.

TABLE 4.1-1 (CONTINUED)

| | <u>Channel Description</u> | <u>Check</u> | <u>Calibrate</u> | <u>Test</u> | <u>Remarks</u> |
|-----|--|--------------|------------------|-------------|--|
| 25. | Containment Pressure | S | R | M | Narrow range containment pressure (-3.0, +3 psig) excluded |
| 26. | Steam Generator Pressure | S | R | M | |
| 27. | Turbine First Stage Pressure | S | R | M | |
| 28. | Emergency Plan Radiation Instruments | M | R | M | |
| 29. | Environmental Monitors | M | NA | NA | |
| 30. | Loss of Voltage/Degraded Voltage 480 Volt Safeguards Bus | NA | R | M | |
| 31. | Trip of Main Feedwater Pumps | NA | NA | R | |
| 32. | Steam Flow | S | R | M | |
| 33. | T _{AVG} | S | R | M | |
| 34. | Chlorine Detector, Control Room Air Intake | NA | R | M | |
| 35. | Ammonia Detector, Control Room Air Intake | NA | R | M | |
| 36. | Radiation Detectors, Control Room Air Intake | NA | R | M | |
| 37. | Reactor Vessel Level Indication System | M | R | NA | |



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 30 TO FACILITY OPERATING LICENSE NO. DPR-18
ROCHESTER GAS AND ELECTRIC CORPORATION
R. E. GINNA NUCLEAR POWER PLANT
DOCKET NO. 50-244

1. INTRODUCTION

On September 24, 1987, Rochester Gas and Electric Corporation submitted an application for amending the Technical Specification to incorporate the requirements for the Reactor Vessel Level Indication System (RVLIS). Action on this proposed revision was delayed pending the final approval of RVLIS by the NRC staff. On April 26, 1988, the staff completed the review of RVLIS of GINNA and found it acceptable, provided the licensee committed to comply with certain staff implementation requirements. The licensee complied with such requirements in their letter of May 3, 1988.

2. EVALUATION

In March 1986, the licensee installed the RVLIS to satisfy a NUREG-0737 requirement. The system supplements existing instrumentation in order to insure an unambiguous, easy-to-interpret indication of inadequate reactor core cooling. The proposed Technical Specifications are in accordance with NRC guidelines in that they meet the required number of instrument channels, the frequency for checks, calibration and testing of the system. RVLIS does not actuate any control function of the reactor system. Its purpose is to provide the plant operator additional information on reactor vessel water level, particularly during transient events. Such data is utilized in the GINNA emergency operating procedures. The proposed Technical Specifications on RVLIS do not create any new or different kind of accident because they do not interact with any safety aspects of plant operations. The additional information, provided to the operator, may increase margins of safety for plant operators during unusual events. Early indication on whether or not the water level is responding to corrective measures taken during an event is important information for safe operations.

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3. ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4. CONCLUSION

The staff evaluated the licensee's request to revise the Technical Specifications to incorporate the Reactor Vessel Level Indication System. The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 23, 1988

Principal Contributor: Carl Stahle