

April 4, 1989

Dr. Robert C. Mecredy, General Manager
Nuclear Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649-0001

Dear Dr. Mecredy:

SUBJECT: ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE NO. DPR-18

The Commission has issued the enclosed Amendment No. 34 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated, March 10, 1987 as supplemented on January 26, 1988, August 26, 1988, and January 19, 1989.

The amendment revises the requirements of the Technical Specifications which are necessary to clarify some of the requirements and to impose staggered monthly testing of both trains of automatic trip logic contact sets, each set to be tested on alternating months. The Technical Specification requirements for Generic Letter 83-28, item 4.3, were provided in Generic Letter 85-09.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

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Carl Stahle, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

Enclosures:

- 1. Amendment No. 34 to License No. DPR-18
- 2. Safety Evaluation

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

APR 04 1989

Docket No. 50-244

Dr. Robert C. Mecredy, General Manager
Nuclear Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649-0001

Dear Dr. Mecredy:

SUBJECT: ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE NO. DPR-18

The Commission has issued the enclosed Amendment No. 34 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated, March 10, 1987 as supplemented on January 26, 1988, August 26, 1988, and January 17, 1989.

The amendment revises the requirements of the Technical Specifications which are necessary to clarify some of the requirements and to impose staggered monthly testing of both trains of automatic trip logic contact sets, each set to be tested on alternating months. The Technical Specification requirements for Generic Letter 83-28, item 4.3, were provided in Generic Letter 85-09.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "Carl Stahle".

Carl Stahle, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

Enclosures:

1. Amendment No. 34 to License No. DPR-18
2. Safety Evaluation

cc w/enclosures:
See next page

Dr. Robert C. Mecredy
Rochester Gas and Electric Corporation

R. E. Ginna Nuclear Power Plant

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Amendment No. 34 to Facility Operating Licensee DPR-18 - R. E. Ginna
Nuclear Power Plant

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Docket File 50-244 ←

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION
DOCKET NO. 50-244
R. E. GINNA NUCLEAR POWER PLANT
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 34
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated March 10, 1987, January 26, 1988, August 26, 1988 and January 17, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 34, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 30 days from date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: APR 04 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 34

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.5-8
3.5-15
4.1-7

INSERT

3.5-8
3.5-15
4.1-7
4.1-7a*

*Indicates new page

TABLE 3.5-1 (Continued)
PROTECTION SYSTEM INSTRUMENTATION

NO.	FUNCTIONAL UNIT	1	2	3	4	5	6
		TOTAL NO. of CHANNELS	NO. of CHANNELS TO TRIP	MIN. OPERABLE CHANNELS	PERMISSIBLE BYPASS CONDITIONS	OPERATOR ACTION IF CONDITIONS OF COLUMN 1 OR 3 CANNOT BE MET	CHANNEL OPERABLE ABOVE
19.	Degraded Voltage 480V Safeguards Bus	2/bus	2/bus	1/bus		7	T _{max} = 350°F
20.	Automatic Trip Logic Including Reactor Trip Breakers	2	1	2	Note 4	14	Note 5

NOTE 1: When block condition exists, maintain normal operation.

NOTE 2: Channels should be operable at all modes below the bypass condition with the reactor trip system breakers in the closed position and control rod drive system capable of rod withdrawal.

NOTE 3: Channels shall be operable at all modes below the bypass condition except during refueling defined to be when fuel is in the reactor vessel with the vessel head closure bolts less than fully tensioned or with the head removed.

NOTE 4: One reactor trip breaker may be bypassed for surveillance testing provided the other reactor trip breaker is operable.

NOTE 5: Channels shall be operable at all modes above refueling when the control rod drive system is capable of rod withdrawal unless both reactor trip breakers are open.

F.P. = Full Power

12. With the number of operable channels less than the Total Number of Channels, operation may proceed provided the inoperable channel is placed in the tripped condition within 1 hour. Should the next Channel Functional Test require the bypass of an inoperable channel to avoid the generation of an actuation signal, operation may proceed until this Channel Functional Test. At the time of this Channel Functional Test, or if at any time the number of operable channels is less than the Minimum Operable Channels required, be at hot shutdown within 6 hours and at an RCS temperature less than 350°F within 6 hours.
13. With the number of operable channels less than the Minimum Operable Channels required, operation may continue provided the containment purge and exhaust valves are maintained closed.
14. Should one reactor trip breaker or channel of trip logic be inoperable the plant must not be in the operating mode following a six hour time period, and the breaker must be open.

If one of the diverse reactor trip breaker trip features (undervoltage or shunt trip attachment) on one breaker is inoperable, restore it to operable status within 48 hours or declare breaker inoperable. If at the end of the 48 hour period one trip feature is inoperable it must be repaired or the plant must not be in the operating mode, and the reactor trip breaker must be open, following an additional six hour time period. The breaker shall not be bypassed while one of the diverse trip features is inoperable except for the time required for performing maintenance to restore the breaker to operable status.

TABLE 4.1-1 (CONTINUED)

<u>Channel Description</u>	<u>Check</u>	<u>Calibrate</u>	<u>Test</u>	<u>Remarks</u>
25. Containment Pressure	S	R	M	Narrow range containment pressure (-3.0, +3 psig) excluded
26. Steam Generator Pressure	S	R	M	
27. Turbine First Stage Pressure	S	R	M	
28. Emergency Plan Radiation Instruments	M	R	M	
29. Environmental Monitors	M	NA	NA	
30. Loss of Voltage/Degraded Voltage 480 Volt Safeguards Bus	NA	R	M	
31. Trip of Main Feedwater Pumps	NA	NA	R	
32. Steam Flow	S	R	M	
33. T _{avg}	S	R	M	
34. Chlorine Detector, Control Room Air Intake	NA	R	M	
35. Ammonia Detector, Control Room Air Intake	NA	R	M	
36. Radiation Detectors, Control Room Air Intake	NA	R	M	
37. Reactor Vessel Level Indication System	M	R	NA	
38a. Trip Breaker Logic Channel Testing	NA	NA	M	Notes 1, 2 and 3
38b. Trip Breaker Logic Channel Testing	NA	NA	R	Note 1

TABLE 4.1-1 (CONTINUED)

<u>Channel Description</u>	<u>Check</u>	<u>Calibrate</u>	<u>Test</u>	<u>Remarks</u>
39. Reactor Trip Breakers	NA	NA	M	Function test - Includes independent testing of both undervoltage and shunt trip attachment of reactor trip breakers. Each of the two reactor trip breakers will be tested on alternate months. Note 2
40. Manual Reactor Trip	NA	NA	R	Includes independent testing of both undervoltage and shunt trip circuits. The test shall also verify the operability of the bypass breaker.
41a. Reactor Trip Bypass Breaker	NA	NA	M	Using test switches in the reactor protection rack manually trip the reactor trip bypass breaker using the shunt trip coil.
41b. Reactor Trip Bypass Breaker	NA	NA	R	Automatically trip the undervoltage trip attachment.

NOTE 1: Logic trains will be tested on alternate months corresponding to the reactor trip breaker testing. Monthly logic testing will verify the operability of all sets of reactor trip logic actuating contacts on that train (See Note 3). Refueling shutdown testing will verify the operability of all sets of reactor trip actuating contacts on both trains. In testing, operation of one set of contacts will result in a reactor trip breaker trip; the operation of all other sets of contacts will be verified by the use of indication circuitry.

NOTE 2: Testing shall be performed monthly, unless the reactor trip breakers are open, or shall be performed prior to startup if testing has not been performed within the last 30 days.

NOTE 3: The source range trip logic may be excluded from monthly testing provided it is tested within 30 days prior to startup.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 34 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

INTRODUCTION

- 1.0 By letters dated March 10, 1987 and January 26, 1988, the licensee, Rochester Gas & Electric Corporation, proposed changes to the Ginna Technical Specifications which would address changes in the surveillance requirements made necessary by the installation of automatic actuation of the shunt trip attachments of the reactor trip breakers as required by Generic Letter (GL) G.L.83-28, Item 4.3. The changes delineated in G.L.83-28 were revised by the issuance of G.L.85-09. The licensee's proposal was reviewed and additional changes were requested to make the Technical Specifications conform to G.L.85-09 requirements. These changes were incorporated by the licensee's submittal of August 26, 1988. On January 17, 1989 the licensee resubmitted the proposed amendment because the format of the table in the Technical Specification required a change, following the acceptance by NRC of a previous amendment.

2.0 EVALUATION

The licensee's Technical Specifications differ in some respects from the Standard Westinghouse Technical Specifications which were used as the basis for the requirements of G.L.85-09. Consequently, the originally proposed changes to address new surveillance test requirements for the shunt trip attachment of the reactor trip breakers did not fully meet the requirements of G.L.85-09.

This proposed amendment constitutes an agreement by the licensee to incorporate monthly staggered testing of the two trains of Reactor Trip Logic and the Reactor Trip Breakers. In addition, the licensee revised Action Statement 14 of Table 3.5-1 and modified footnote 5 of table 3.5-1 to comply with the G.L.85-09 requirements. All other aspects of Generic Letter 83-28, Item 4.3 are resolved to the satisfaction of the staff.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted areas as defined in 10 CFR Part 20. The staff has determined that the amendment involves

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no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published in the Federal Register a proposed finding that this amendment involves no significant hazards consideration, 53 Fed. Reg. 9513 (March 23, 1988). There has been no public comment on such finding. Subsequent licensee submittals of August 26, 1988 and January 17, 1989 were only minor corrections to the original submittal; therefore, it was determined that it was not necessary to renotice the application. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has evaluated the licensee's request to incorporate on-line reactor trip breaker testing into the Technical Specifications. The change in the test requirements will provide a more reliable reactor trip breaker system.

The staff concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: APR 04 1989

Principal Contributor: D. Lasher