

April 24, 1989

Docket No. 50-244

Dr. Robert C. Mecredy, General Manager
Nuclear Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Dr. Mecredy:

SUBJECT: ISSUANCE OF AMENDMENT NO. 35 TO DPR-18 - R. E. GINNA NUCLEAR
POWER PLANT (TAC NO. 71575)

The Commission has issued the enclosed Amendment No. 35 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant in response to your application dated February 16, 1989.

The amendment revises the Technical Specifications to reflect the addition of steam generator sleeving and plugging criteria.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

15/
Carl Stahle, Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

Enclosures:

- 1. Amendment No. 35 to License No. DPR-18
- 2. Safety Evaluation

cc w/enclosure:
See next page

RMS

OFC: PDI-3	PDI-3	: OGC	DD	: DIR/PDI-3:	:	:
NAME: MRushbrook	: Stahle	: N. Romero	: RWessman	:	:	:
DATE: 3/15/89	: 3/15/89	: 3/23/89	: 3/30	:	:	:

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 35, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

15/
Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 24, 1989

OFC: PDI-3	: PDI-3	: DIR/PDI-3:	OGC <i>LR</i>	:	:
NAME: MR <i>Brook</i> : CStahle	: RWessman	:	S H Lewis	:	:
DATE: 3/15/89	: 3/15/89	: 3/31/89	: 3/28/89:	:	:



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 24, 1989

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Rochester Gas & Electric Corporation
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The amendment revises the Technical Specifications to reflect the addition of steam generator sleeving and plugging criteria.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Carl Stahle".

Carl Stahle, Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

Enclosures:

1. Amendment No. 35 to License No. DPR-18
2. Safety Evaluation

cc w/enclosure:
See next page

Dr. Robert C. Mecredy
Rochester Gas and Electric Corporation

R. E. Ginna Nuclear Power Plant

cc:

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DATE April 24, 1989

AMENDMENT NO. 35 TO DPR-18 - R. E. GINNA NUCLEAR POWER PLANT

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cc: Plant Service List



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 35
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Rochester Gas and Electric Corporation (the licensee), dated February 16, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-18 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 35, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 24, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 35

FACILITY OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A as follows:

Remove Pages

4.2-2

Insert Pages

4.2-2
4.2-3*

*Denotes new page

and 2,000 respectively. The inspection program during each third of the first inspection interval provides for examination of all welds at design basis break locations and one-third of all welds at locations where a weld failure would result in unacceptable consequences. During each succeeding inspection interval, the program shall provide for an examination of each of the design basis break location welds, and each of the welds at locations where a weld failure would result in unacceptable consequences.

- 4.2.1.4 The inspection intervals for Steam Generator Tubes shall be specified in the "Inservice Inspection Program" for the applicable forty month period commencing with May 1, 1973.
- 4.2.1.4.a Steam generator tubes that have imperfections greater than 40% through wall, as indicated by eddy current, shall be repaired by plugging or sleeving.
- 4.2.1.4.b Steam generator sleeves that have imperfections greater than 30% through wall, as indicated by eddy current, shall be repaired by plugging.
- 4.2.1.5 Inservice Inspection of ASME Code Class 1, Class 2 and Class 3 components (Quality Groups A, B and C) shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).
- 4.2.1.6 The inspection interval for the Inservice Pump and Valve Testing Program shall be ten year intervals commencing with January 1, 1981, 1990 and 2000.

Basis:

The inservice inspection programs provide assurance for the continued structural integrity of the structures, components and systems of Ginna Station. The programs comply with the ASME Boiler and Pressure Vessel Code Section XI "Rules for Inservice Inspection of Nuclear Power Plant Components" as practicable, with due consideration to the design and physical access of the structures,

Basis (Cont'd)

components and systems as manufactured and constructed. This compliance will constitute an acceptable basis for satisfying the requirements of General Design Criterion 32, Appendix A of 10 CFR Part 50 and the requirements of Section 50.55a, paragraph g of 10 CFR Part 50.

The repair criteria of 4.2.1.4.a and 4.2.1.4.b are based on the requirements of USNRC Regulatory Guide 1.121, "Bases for Plugging Degraded PWR Steam Generator Tubes" as implemented by RG&E (Reference 1). This guide describes a method acceptable to the NRC staff for establishing the limiting safe conditions of tube degradation of steam generator tubing. The repair criteria is based on structural allowances, an allowance for eddy current measurement error and an allowance for degradation during the operating period. These allowances are added together to determine the repair criteria which is typically 40% for steam generator tubes. Based on calculations the appropriate sleeve plugging limit is a 42% thru wall defect. In order to allow for conservatism, a 30% plugging limit for sleeves will be utilized.

Reference 1: "Steam Generator Rapid Sleaving Program Design Verification Report", R.E. Ginna Nuclear Power Plant, August 1982.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 35 TO FACILITY OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

By letter dated February 16, 1989, Rochester Gas & Electric Corporation proposed changes to Technical Specifications (TS) 4.2-2 for the Ginna Nuclear Power Plant. The proposed changes would add quantitative requirements for resleeving and plugging of steam generator tubes. These criteria are presently required in the plant Quality Assurance (QA) manual.

2.0 BACKGROUND

On July 19, 1988, the NRC staff met with representatives from Rochester Gas & Electric Corporation (RG&E) and its consultants from Combustion Engineering to discuss steam generator sleeving utilizing a new technique for sleeve installation. (See NRC letter to RG&E dated December 1, 1988).

The purpose of the meeting was for RG&E to describe the development program which would result in a sleeve design and necessary tooling to service currently inaccessible areas of the steam generators such that as much as 95% of the steam generators could be resleeved for extended service life.

Additionally, the NRC staff suggested the benefits of including steam generator tube sleeving and plugging requirements in the plant TS. The licensee agreed and submitted an amendment request to their license.

3.0 EVALUATION

The proposed additions to the technical specifications are as follows:

1. 4.2.1.4.a - Steam generator tubes that have imperfections greater than 40% throughwall, as indicated by eddy current, shall be repaired by plugging or sleeving.
2. 4.2.1.4.b - Steam generator sleeves that have imperfections greater than 30% throughwall, as indicated by eddy current, shall be repaired by plugging.
3. The repair criteria of 4.2.1.4.a and 4.2.1.4.b are based on the requirements of USNRC Regulatory Guide 1.121, "Bases for Plugging

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Degraded PWR Steam Generator Tubes," as implemented by RG&E (Reference 1). This guide describes a method acceptable to the NRC staff for establishing the limiting safe conditions of tube degradation of steam generator tubing. The repair criteria is based on structural allowances, an allowance for eddy current measurement error and an allowance for degradation during the operating period. These allowances are added together to determine the repair criteria which is typically 40% for steam generator tubes. Based on calculations the appropriate sleeve plugging limit is a 42% throughwall defect. In order to allow for conservatism, a 30% plugging limit for sleeves will be utilized.

The licensee's proposed changes to its technical specification are administrative in nature and are consistent with other PWR TS. The change would add the same requirements that currently exist in the plant Quality Assurance manual to the plant technical specifications.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, or administrative procedures or requirements because the same requirements that currently exist in the plant Quality Assurance Manual are being added to the plant Technical Specifications. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration, which was published in the Federal Register (54FR11942) on March 22, 1989. The Commission consulted with the State of New York. No public comments were received, and the State of New York did not have any comments.

On the basis of the consideration discussed above, the staff concludes that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: C. Stahle, D. Solorio

Dated: April 24, 1989