Branch

JUN 23 1972

Docket No. 50-244

Rochester Gas & Electric Corporation
ATTN: Mr. Edward J. Nelson, President
89 East Avenue
Rochester, New York 14604

Gentlemen:

By letter dated June 12, 1972, you informed us of anomalies in some of the fuel bundles of the R. E. Ginna Nuclear Power Plant Unit No. 1 which you discovered during refueling. The letter proposed the resumption of operation with some of these fuel bundles remaining in the core but with a number of restrictions on operation, including an overall reduction in the reactor power level to 83% of full power (1266 MWt). The basis for the proposed operation was included as a proprietary attachment to that letter. Your letters of June 20, 1972, and June 22, 1972, including attachments, presented responses to questions the staff had asked on Friday, June 16, 1972, that were pertinent to interim operation of the reactor.

We have considered your proposal, the basis for your proposal, and the answers which you provided to our questions. Based on our review of the material submitted, our discussions with you, and our re-evaluation of steady-state operation, transients, and accidents presented in the Safety Analysis Report, we have concluded that interim operation of the R. E. Ginna Nuclear Power Plant Unit No. 1 may be safely resumed subject to the conditions you have proposed, as supplemented by the additional conditions specified in Attachment A entitled "Interim Conditions for Operation of the R. E. Ginna Nuclear Power Plant Unit No. 1". Authorization of reactor operation under conditions other than those specified is dependent upon further evaluation of the longer term effects of the fuel anomalies which should be described and submitted for our review as soon as possible, but no later than 60 days from the date of this letter.

Representatives of your Company have agreed to follow the conditions specified above in the resumed operations of the R. E. Ginna Nuclear Power Plant Unit No. 1. Please submit, as appropriate, proposed changes in the Technical Specifications which reflect the changed operating limitations.

Sincerely,

/s/ Paul Collins

Donald J. Skovholt
Assistant Director
for Operating Reactors
Directorate of Licensing

Attachment A - Interim Conditions for Operation

cc: LeBoeuf, Lamb, Leiby & MacRae
1821 Jefferson Place, N. W.
Washington, D. C. 20036

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ATTACHMENT A

INTERIM CONDITIONS FOR OPERATION

OF THE R. E. GINNA NUCLEAR POWER PLANT UNIT NO. 1

DOCKET NO. 50-244

1. The maximum steady-state power level shall not exceed 1266 MWt. This power level shall be considered as 100% rated power for the purpose of interpreting Safety Limits, Limiting Safety System Settings, and Limiting Conditions for Operation given in the Technical Specifications to License No. DPR-18.

2. No more than one unprepressurized fuel bundle shall be subjected to a higher linear power (kW/ft) than it has already experienced. That one bundle shall not exceed the previous experienced power by more than 10%.

3. The peak power of the unprepressurized fuel bundles shall be limited to 16.0 kW/ft.

4. The primary to secondary leakage in a steam generator shall not exceed 0.1 gpm.

5. For cycle 2 startups:

   (a) In-core flux maps shall be measured and evaluated prior to proceeding to higher power at 25 percent, 50 percent, 75 percent and immediately after reaching 100 percent of rated power during startups from a cold condition.

   (b) Primary coolant gross radioactivity shall be measured at least twice per shift.

   (c) Secondary coolant gross radioactivity shall be measured daily to evaluate steam generator leak tightness.

6. For cycle 2 operation other than startup:

   (a) In-core flux maps shall be measured and evaluated at least weekly and after any power increase greater than 10 percent of rated power to assure items 2 and 3 above are met.

(b) Primary coolant gross radioactivity shall be measured at least daily and after any power change greater than 10 percent of rated power.

(c) Secondary coolant gross radioactivity shall be measured at least daily to evaluate steam generator leak tightness. The frequency shall be increased to twice per shift if a significant increase is measured over the previous measurement.

7.0 REPORTING REQUIREMENTS

7.1 Liquid Wastes

If the release of radioactive materials in liquid wastes, when summed over a calendar quarter exceeds 2.5 curies, excluding tritium and dissolved gases:

(a) An investigation shall be made to identify the cause for such release rates,

(b) A program of action to reduce such release rates shall be defined and initiated, and

(c) A report shall be filed with the AEC within 10 days describing the results of the investigation and the corrective actions taken.

7.2 Gaseous Wastes

If the rate of release of noble gases in gaseous wastes, when averaged over a calendar quarter exceeds a total of 2.4 millicuries per second:

(a) An investigation shall be made to identify the causes for such release rates, and

(b) A program to reduce such release rates, including a reduction in power level shall be defined and initiated immediately, and

(c) A report shall be filed with the AEC within 10 days describing the results of the investigation and the corrective actions taken.

7.3 A monthly report of all primary and secondary activity measurements and effluent discharge activity levels shall be made to the Director of Licensing.