UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET SW SUITE 23T85
ATLANTA, GEORGIA 30303-8931

February 22, 2001

Carolina Power and Light Company
ATTN: Mr. J. S. Keenan
Vice President
Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461

SUBJECT: NOTIFICATION OF CONDUCT OF A TRIENNIAL FIRE PROTECTION BASELINE INSPECTION, (NRC INSPECTION REPORT NO. 50-325/01-06, 50-324/01-06)

Dear Mr. Keenan:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a triennial fire protection baseline inspection at Brunswick Steam Electric Plant in May 2001. The inspection team will be lead by Mr. M. Thomas, a senior reactor inspector from the NRC Region II Office. The team will be composed of personnel from NRC Region II staff. The inspection will be conducted in accordance with the NRC's baseline fire protection inspection procedure 71111.05.

The inspection objective will be to evaluate your fire protection program implementation with emphasis on post-fire safe shutdown capability and the fire protection features provided to ensure at least one post-fire safe shutdown success path is maintained free of fire damage. The inspection team will focus their review of the separation of systems and equipment necessary to achieve and maintain safe shutdown and fire protection features of selected fire areas.

On February 20, 2001, during a telephone conversation, our respective staffs confirmed arrangements for a three-day information gathering site visit and a one-week onsite inspection. The schedule is as follows:

- Information gathering visit - April 10-12, 2001.
- Week of onsite inspection - May 7-11, 2001.

The purposes of the information gathering visit are to obtain information and documentation needed to support the inspection; to become familiar with the Brunswick fire protection program, fire protection features, and post-fire safe shutdown capabilities and plant layout; and, as necessary, obtain plant specific site access training and badging for unescorted site access. The types of documents the team will be interested in reviewing, and possibly obtaining, are listed in the Enclosure. Please do not copy in advance all of the material in the Enclosure. The inspection team will try to minimize your administrative burden by specifically identifying those documents required for inspection preparation.

During the information gathering visit, the team will also discuss the following inspection support administrative details: office space; specific documents requested to be made available to the team in their office space; arrangements for reactor site access (including radiation protection training, security, safety and fitness for duty requirements); and the availability of knowledgeable plant engineering and licensing organization personnel to serve as points of contact during the inspection.

We request that during the onsite inspection week you ensure that copies of analyses, evaluations or documentation regarding the implementation and maintenance of the Brunswick Steam Electric Plant fire protection program, including post-fire safe shutdown capability, be readily accessible to the team for their review. Of specific interest are those documents which establish that your fire protection program satisfies NRC regulatory requirements and conforms to applicable NRC and industry fire protection guidance. Also, personnel should be available at the site during the inspection who are knowledgeable regarding those plant systems required to achieve and maintain safe shutdown conditions from inside and outside the control room (including the electrical aspects of the relevant post-fire safe shutdown analyses), reactor plant fire protection systems and features, and the fire protection program and its implementation.

Your cooperation and support during this inspection will be appreciated. If you have questions concerning this inspection, or the inspection team's information or logistical needs, please contact me at (404) 562-4605, or Mr. M. Thomas at (404) 562-4673.

Sincerely,

/RA/

Charles R. Ogle, Chief
Engineering Branch
Division of Reactor Safety

Docket Nos. 50-325, 50-324
License Nos. DPR-71, DPR-62

Enclosure: Supporting Documentation Requested for the
Reactor Fire Protection Program

cc w/encl:
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E-MAIL COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO

OFFICIAL RECORD COPY DOCUMENT NAME: C:\brunswick fp letter.wpd

Reactor Fire Protection Program Supporting Documentation

[Note: This is a broad list of the documents the NRC inspection team may be interested in reviewing, and possibly obtaining, during the information gathering site visit. Electronic media is preferred if readily available (i.e., on CD-ROM or computer disc)]

1. The current version of the Fire Protection Program and Fire Hazards Analysis.

2. Current versions of the fire protection program implementing procedures (e.g., administrative controls, surveillance testing, fire brigade).

3. Fire brigade training program and pre-fire plans.

4. Post-fire safe shutdown systems and separation analysis.

5. Post-fire alternative shutdown analysis.

6. Piping and instrumentation (flow) diagrams showing the components used to achieve and maintain hot standby and cold shutdown for fires outside the control room and those components used for those areas requiring alternative shutdown capability.

7. Plant layout and equipment drawings which identify the physical plant locations of hot standby and cold shutdown equipment.

8. Plant layout drawings which identify plant fire area delineation, areas protected by automatic fire suppression and detection, and the locations of fire protection equipment.

9. Plant layout drawings which identify the general location of the post-fire emergency lighting units.

10. Plant operating procedures which would be used and describe shutdown from inside the control room with a postulated fire occurring in any plant area outside the control room, procedures which would be used to implement alternative shutdown capability in the event of a fire in either the control or cable spreading room.

11. Maintenance and surveillance testing procedures for alternative shutdown capability and fire barriers, detectors, pumps and suppression systems.

12. Maintenance procedures which routinely verify fuse breaker coordination in accordance with the post-fire safe shutdown coordination analysis.

13. A sample of significant fire protection and post-fire safe shutdown related design change packages (including their associated 10 CFR 50.59 evaluations) and Generic Letter 86-10 evaluations.

Enclosure

14. The reactor plant's IPEEE, results of any post-IPEEE reviews, and listings of actions taken/plant modifications conducted in response to IPEEE information.

15. Temporary modification procedures.

16. Organization charts of site personnel down to the level of fire protection staff personnel.

17. If applicable, layout/arrangement drawings of potential reactor coolant/recirculation pump lube oil system leakage points and associated lube oil collection systems.

18. A listing of the SERs and actual copies of the 50.59 reviews which form the licensing basis for the reactor plant's post-fire safe shutdown configuration.

19. Procedures/instructions that control the configuration of the reactor plant's fire protection program, features, and post-fire safe shutdown methodology and system design.

20. A list of applicable codes and standards related to the design of plant fire protection features and evaluations of code deviations.

21. Procedures/instructions that govern the implementation of plant modifications, maintenance, and special operations, and their impact on fire protection.

22. The three most recent fire protection QA audits and/or fire protection self-assessments.

23. Recent QA surveillances of fire protection activities.

24. A listing of open and closed fire protection condition reports (problem reports/NCRs/EARs/problem identification and resolution reports).

25. Listing of plant fire protection licensing basis documents.

26. A listing of the NFPA code versions committed to (NFPA codes of record).

27. A listing of plant deviations from code commitments.

28. Actual copies of Generic Letter 86-10 evaluations.