

# REGULATORY DOCKET FILE COPY

April 23, 1981

Docket No. 50-244  
LS05-81-04-050

Mr. John E. Maier  
Vice President  
Electric and Steam Production  
Rochester Gas and Electric Corporation  
89 East Avenue  
Rochester, New York 14649

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Dear Mr. Maier:

SUBJECT: DIESEL GENERATOR SURVEILLANCE AND TESTING - R. E. GINNA NUCLEAR POWER PLANT

The Commission has issued the enclosed Amendment No. 41 to Provisional Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment responds to your application notarized November 16, 1977 (submitted by letter dated November 21, 1977).

The amendment modifies the Technical Specifications for the R. E. Ginna Nuclear Power Plant to remove an inconsistency regarding the electrical load to be added to the generator during testing. The Technical Specifications regarding diesel generator surveillance and testing have also been expanded in conformance with your suggested changes.

Changes have been made to your submittal as mutually agreed upon during telephone conversations with your staff.

Copies of our Safety Evaluation and Notice of Issuance are also enclosed.



Sincerely,

Original signed by  
Dennis M. Crutchfield

Dennis M. Crutchfield, Chief  
Operating Reactors Branch #5  
Division of Licensing

### Enclosures:

1. Amendment No. 41  
to DPR-18
2. Safety Evaluation
3. Notice

cc w/enclosures:

See next page

LA

DL:ORB.#5/

HSmith

4/13/81

DL:ORB.#5/PM

RSnaider:rj

4/15/81

OELD

Ketchen

4/17/81

DL:ORB.#5/C

DCrutchfield

4/21/81

DL:AD/SA

GLamas

4/23/81

OFFICE	See next page	LA	DL:ORB.#5/	DL:ORB.#5/PM	OELD	DL:ORB.#5/C	DL:AD/SA
SURNAME			HSmith	RSnaider:rj	Ketchen	DCrutchfield	GLamas
DATE	P		4/13/81	4/15/81	4/17/81	4/21/81	4/23/81



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

April 23, 1981

Docket No. 50-244  
LS05-81-04-050

Mr. John E. Maier  
Vice President  
Electric and Steam Production  
Rochester Gas and Electric Corporation  
89 East Avenue  
Rochester, New York 14649

Dear Mr. Maier:

SUBJECT: DIESEL GENERATOR SURVEILLANCE AND TESTING - R. E. GINNA NUCLEAR  
POWER PLANT

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Sincerely,

*Dennis M. Crutchfield*  
Dennis M. Crutchfield, Chief  
Operating Reactors Branch #5  
Division of Licensing

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1. Amendment No. 41  
to DPR-18
2. Safety Evaluation
3. Notice

cc w/enclosures:  
See next page

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Mr. John E. Maier

- 2 -

April 23, 1981

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 41  
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Rochester Gas and Electric Company (the licensee) notarized November 16, 1977 (transmitted by letter dated November 21, 1977), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the filing, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Provisional Operating License No. DPR-18 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 41, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Dennis M. Crutchfield, Chief  
Operating Reactors Branch #5  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 23, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 41

PROVISIONAL OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages contain the captioned amendment number and vertical lines which indicate the area of change.

Remove

Insert

4.6-1

4.6-1

- - -

4.6-1a

4.6-2

4.6-2

Applicability

Applies to periodic testing and surveillance requirements of the emergency power system.

Objective

To verify that the emergency power system will respond promptly and properly when required.

Specification

The following tests and surveillance shall be performed as stated:

4.6.1 Diesel Generators

Each diesel generator shall be demonstrated operable:

- a. Except during cold or refueling shutdown at least once per 31 days by:
  1. Verifying the fuel level in the day tank,
  2. Verifying a minimum oil storage of 10,000 gallons is at the station,
  3. Verifying the fuel transfer pump can be started and transfer fuel from the storage system to the day tank,
  4. Verifying the diesel starts from normal standby conditions,
  5. Verifying the generator is synchronized, loaded to at least 1950 kw but less than the 2 hour rating of 2250 kw and operates for at least 60 minutes but less than 120 minutes,
  6. Verifying the diesel generator is aligned to provide standby power to the associated emergency buses.
- b. The tests in specification 4.6.1a will be performed prior to exceeding 5% power if the time since the last test exceeds 31 days.
- c. At least once per 92 days by verifying that a sample of diesel fuel from the fuel storage tank is within the acceptable limits specified in Table 1 of ASTM D975-78 when checked for viscosity, water and sediment.

d. At least once per 18 months during shutdown by:

1. Inspecting the diesel in accordance with the manufacturer's recommendations for this class of standby service,
2. Verifying the generator capability to reject a load of 295 KW without tripping,
3. Simulating a loss of offsite power in conjunction with a safety injection test signal and:

(a) Verifying de-energization of the emergency buses and load shedding from the emergency buses.

(b) Verifying the diesel starts from normal standby condition on the auto-start signal, energizes the emergency buses with permanently connected loads, energizes the automatically connected emergency loads with the following maximum breaker closure times after the initial starting signal for trains A and B not being exceeded

	A	B
Diesel plus Safety Injection Pump	20 sec	22 sec
plus RHR Pump		

All breakers	40 sec	42 sec
--------------	--------	--------

and operates for  $\geq$  five minutes while its generator is loaded with emergency loads.

(c) Verifying that all diesel generator trips, except engine overspeed, low lube oil pressure, and overcrank, are automatically bypassed upon loss of voltage on the emergency bus and/or safety injection actuation signal.



4.6.2 Station Batteries

- a. Every month the voltage of each cell (to the nearest 0.01 volt), the specific gravity and temperature of a pilot cell in each battery shall be measured and recorded.
- b. Every 3 months the specific gravity of each cell, the temperature reading of every fifth cell, the height of electrolyte, and the amount of water added shall be measured and recorded.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 41 TO PROVISIONAL OPERATING LICENSE NO. DPR-18  
ROCHESTER GAS AND ELECTRIC CORPORATION  
R. E. GINNA NUCLEAR POWER PLANT  
DOCKET NO. 50-244

1.0 INTRODUCTION

By application notarized November 16, 1977, (submitted by letter dated November 21, 1977), Rochester Gas and Electric Corporation (RG&E) (the licensee) requested changes to the Technical Specifications for the R. E. Ginna Nuclear Power Plant. The proposed changes would remedy an existing inconsistency in the specifications regarding diesel generator testing.

2.0 EVALUATION

Existing Technical Specification 4.6.1 would require that, during testing, the Ginna diesel generators be loaded to their nameplate rating and that electrical loads not be increased beyond the long term rating of 1950 kW. However, the nameplate rating is 2500 KVA at 80% power factor, the equivalent of 2000 kW, which is in excess of the long-term limit. To remedy this inconsistency, RG&E has proposed that the limits be modified such that each generator is loaded to at least 1950 kW but less than the two-hour rating of 2250 kW.

The NRC staff's present guidance is contained in Regulatory Guide 1.108, "Periodic Testing of Diesel Generator Units Used as Onsite Electric Power Systems at Nuclear Power Plants". The applicable portion of this Guide calls for demonstration of full-load-carrying capability (continuous rating) at an interval of no more than 31 days. Because the proposed change conforms to this guidance and because it will remedy the inconsistency noted above, we have found it to be acceptable.

In addition to the above specification, RG&E proposed the addition of several other specifications related to diesel generator operability. We have reviewed these specifications and have concluded that they add to the assurance that the diesel generators will operate as required. Because they enhance diesel generator operability and because they improve upon the existing specifications, we have found them to be acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

We have determined that the proposed amendment does not authorize a change in effluent types, increase in total amounts of effluents, or an increase in power level, and will not result in any significant environmental impact. Having made this determination, we have concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact, and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

### 4.0 CONCLUSION

We also conclude, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public.

Date: April 23, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-244ROCHESTER GAS AND ELECTRIC CORPORATIONNOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 41 to Provisional Operating License No. DPR-18, issued to Rochester Gas and Electric Corporation (the licensee), which revised the Technical Specifications for operation of the R. E. Ginna Plant (facility) located in Wayne County, New York. This amendment is effective as of its date of issuance.

The amendment incorporates revised Technical Specifications regarding diesel generator surveillance and testing.


The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment notarized November 16, 1977 (transmitted by letter dated November 21, 1977, (2) Amendment No. 41 to License No. DPR-18, and (3) the Commission's related Safety Evaluation. These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and the Rochester Public Library, 115 South Avenue, Rochester, New York 14627. A single copy of items (2) and (3) may be obtained by request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 23rd day of April, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Dennis M. Crutchfield, Chief  
Operating Reactors Branch #5  
Division of Licensing