NOV 3 1975

Electric Corporation
D. White, Jr.
President
Electric and Steam Production
89 East Avenue
Rochester, New York 14604

Gentlemen:

The Commission has issued the enclosed Amendment No. 8 to Provisional Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment includes Change No. 17 to the Technical Specifications and is in response to your request dated December 2, 1974.

The amendment incorporates into the Ginna Technical Specifications changes to the Administrative Controls. Changes to your proposal were necessary to meet our requirements. These have been discussed with your staff. The technical specifications are based on the regulatory positions described in guides 1.8, "Personnel Selection and Training", 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4, and 1.33, "Quality Assurance Program Requirements".

We request that you use the formats presented in the Appendices to Regulatory Guide 1.16, Revision 4, for reporting operating information and that you report events of the type described under the section "Events of Potential Public Interest". Instructions for using these reporting formats are contained in Regulatory Guide 1.16 (a copy is enclosed for your use), and AEC report OCE-SS-CC1 titled "Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File" (a copy of which was provided you previously). This report is modified by updated instructions dated August 21, 1975 which are enclosed. Copy requirements are summarized in Regulatory Guide 10.1, "Compilation of Reporting Requirements for Persons Subject to NRC Regulations", a copy of which is also enclosed. This Guide will assist you in identifying reports that are required by the Commission's regulations set forth in Title 10 Code of Federal Regulations but are not contained in your technical specifications. Reports that are required by the regulations have not been repeated in your technical specifications.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

Original signed by
R. A. Purple

Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Reactor Licensing

Enclosures: See next page

OFFICE ➤						
SURNAME ➤						
DATE ➤						

Form AEC-318 (Rev. 9-53) AECM 0240 ☆ U. S. GOVERNMENT PRINTING OFFICE: 1974-526-166

Enclosures:
1. Amendment No. 8
2. Regulatory Guide 1.16
3. Updated Instructions
4. Regulatory Guide 10.1
5. Safety Evaluation
6. Federal Register Notice

cc w/enclosures:
See next page

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NDube
BJones(4)
BScharf(15)
OI&E(3)
OELD
ACRS(16)
TBAbernathy, TIC
RHVollmer
FAllenspach

OFFICE ►	RL:ORB#1 WMHiggins:dc	RL:QAB RHVollmer	OELD	RL:ORB#1 RAPurple		
SURNAME ►	TVWambach:dc					
DATE ►	10/15/75	10/23/75	10/23/75	10/23/75		

Form AEC-318 (Rev. 9-53) AECM 0240 ☆ U.S. GOVERNMENT PRINTING OFFICE: 1974 — 555-338

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ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 8
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated December 2, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Provisional Operating License No. DPR-18 is hereby amended to read as follows:

OFFICE ➤						
SURNAME ➤						
DATE ➤						

"(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 17."

3. This license amendment is effective 30 days after the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by
R. A. Purple

Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Reactor Licensing

Attachment:
Change No. 17 to the Technical Specifications

Date of Issuance: NOV 3 1975

OFFICE ➤						
SURNAME ➤						
DATE ➤						

ATTACHMENT TO LICENSE AMENDMENT NO. 8

CHANGE NO. 17 TO THE TECHNICAL SPECIFICATIONS

PROVISIONAL OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A as follows:

1. Remove Page ii.
2. Remove Section 6 in its entirety.

Remove Pages
6.1-1 through 6.1-15
6.2-1
6.3-1
6.4-1 through 6.4-12
6.5-1 and 6.5-2
6.6-1 through 6.6-10

1. Insert new Pages ii and iii.
2. Insert Section 6 in its entirety.

Insert Pages
6.1-1
6.2-1 through 6.2-3
6.3-1
6.5-1 through 6.5-11
6.6-1
6.8-1
6.9-1 through 6.9-10
6.10-1 and 6.10-2
6.11-1
6.12-1 through 6.12-8
6.13-1

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17

TABLE OF CONTENTS (cont.)

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6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Station Superintendent shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence. 17

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1. 17

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and: 17

a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Figure 6.2-2.

b. At least once licensed Operator shall be in the control room when fuel is in the reactor.

c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.

d. All core alterations shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation. 17

e. **An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.**

R.E. GINNA NUCLEAR POWER PLANT
MANAGEMENT ORGANIZATION CHART

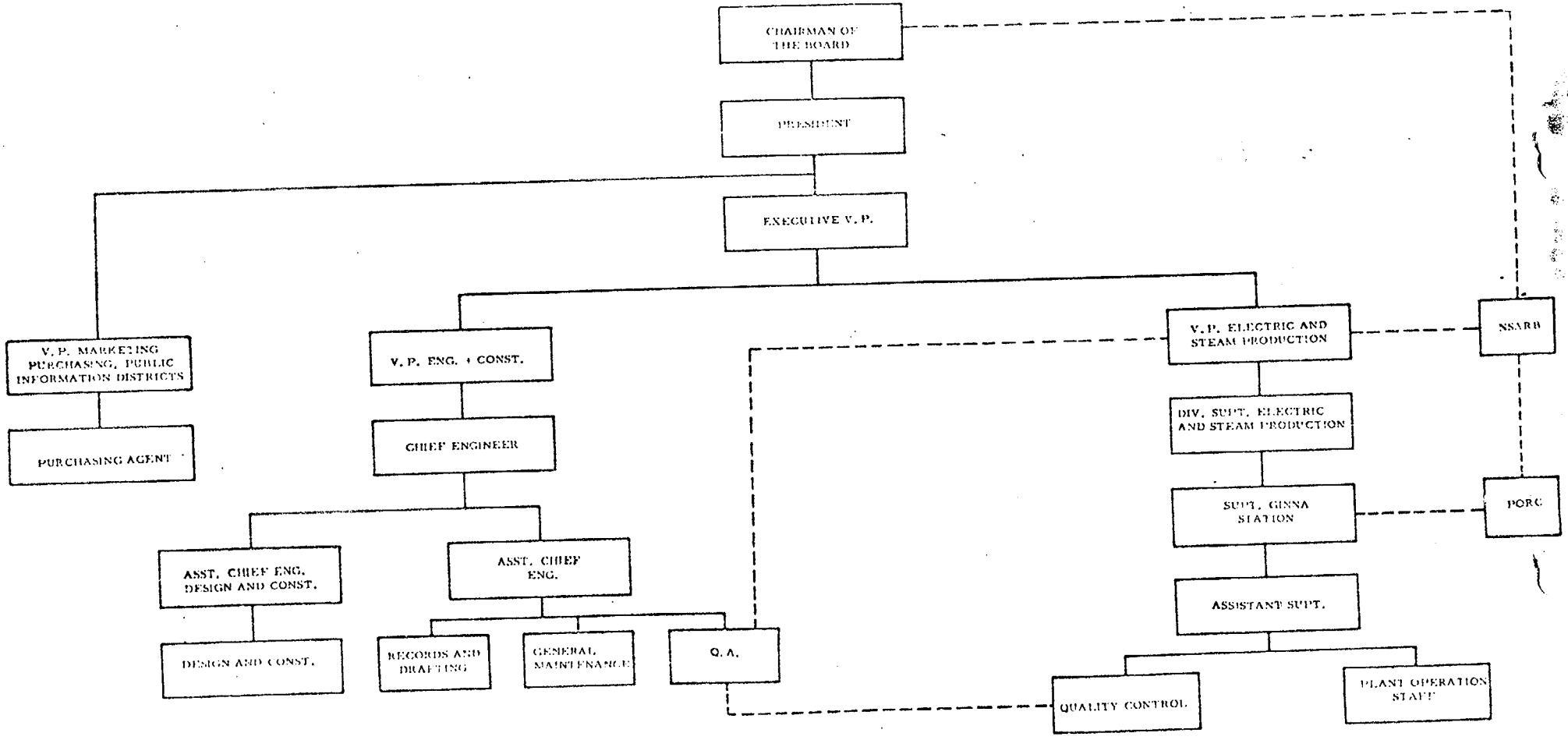
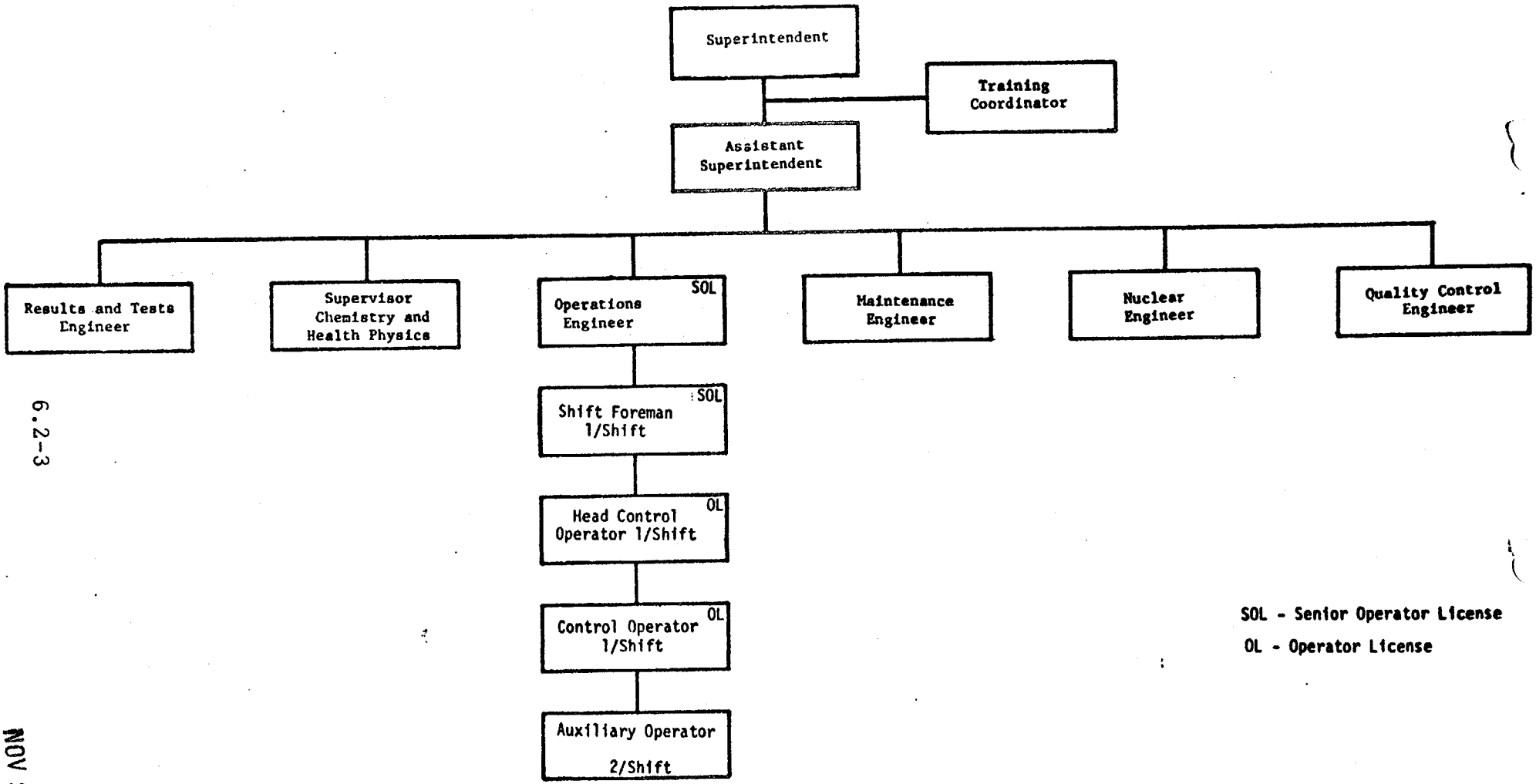


FIGURE 6.2-1

6.2-2

NOV 3 1975

:OCHESTER GAS AND ELECTRIC CORPORATION
GINNA STATION ORGANIZATION



SOL - Senior Operator License

OL - Operator License

Figure 6.2-2

6.2-3

NOV 3 1975

6.3 STATION STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI Standard N18.1-1971, "Selection and Training of Nuclear Power Plant Personnel" for comparable positions.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Training Coordinator and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix A of 10 CFR Part 55.

17

6.5 REVIEW AND AUDIT

Three separate organizational units shall be established for the purpose of review and audit of plant operations and safety related matters. One of these will be an on site operations review group, the Plant Operations Review Committee (PORC). A second is the Quality Assurance (Q.A.) group, responsible for the audit of safety related activities associated with plant operations. A third is the independent audit and review group, the Nuclear Safety Audit and Review Board (NSARB). This group is responsible for the periodic review of the activities of the Plant Operations Review Committee, for directing audits and evaluating their results, and for the management evaluation of the status and adequacy of the quality assurance program. 17

6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC)

FUNCTION

6.5.1.1 The Plant Operations Review Committee shall function to advise the Station Superintendent on all matters related to nuclear safety and for referral of appropriate matters to the Nuclear Safety Audit and Review Board.

COMPOSITION

6.5.1.2 The PORC shall be composed of the:

Chairman:	Station Superintendent	1 ?
Member:	Assistant Superintendent	
Member:	Operations Engineer	
Member:	Maintenance Engineer	
Member:	Results and Test Engineer	
Member:	Nuclear Engineer	
Member:	Supervisor Chemistry and Health Physics	
Member:	Quality Control Engineer	
Member:	**Instrument and Control Supervisor**	1 7

ALTERNATES

6.5.1.3 Alternate members shall be designated by name, in writing, by the Chairman.

MEETING FREQUENCY

6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairman.

QUORUM

6.5.1.5 A quorum of the PORC shall consist of the Chairman and four members including alternates. No more than two shall be alternates.

RESPONSIBILITIES

6.5.1.6 The PORC shall be responsible for:

a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Station Superintendent to affect nuclear safety. 17

b. Review of all proposed tests and experiments that **affect** nuclear safety.

c. Review of all proposed changes to the Technical Specifications.

d. Review of all proposed changes or modifications to plant systems or equipment that **affect** nuclear safety. 17

e. Investigation of all violations of the **Technical** Specifications and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Vice President, Electric and Steam Production, and to the Chairman of the Nuclear Safety Audit and Review Board.

f. Review of facility operations to detect potential safety hazards.

g. Performance of special reviews and investigations and reports thereon as requested by the Chairman of the Nuclear Safety Audit and Review Board.

h. Review of the Plant Security Plan and shall submit recommended changes to the Chairman of the Nuclear Safety Audit and Review Board.

i. Review of the Radiation Emergency Plan and shall submit recommended changes to the Chairman of the Nuclear Safety Audit and Review Board.

j. Review of implementing procedures for the Plant Security Plan and the Radiation Emergency Plan and proposed changes thereto.

RESPONSIBILITIES (Continued)

k. **Review of all events which are required by regulations or Technical Specifications to be reported to the NRC in writing within 24 hours.**

AUTHORITY

6.5.1.7 The PORC shall:

a. Recommend **in writing to the Station Superintendent approval or** disapproval of items considered under 6.5.1.6(a) through (d) above. 17

b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (d) above constitutes an unreviewed safety question as defined in 10 CFR Section 50.59. 17

c. Provide immediate written notification to the Vice President, Electric and Steam Production, and the Nuclear Safety Audit and Review Board of disagreement between the PCRC and the Station Superintendent; however, the Station Superintendent shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above. 17

RECORDS

6.5.1.8 The PCRC shall maintain written minutes of each meeting and copies shall be provided to the Vice President, Electric and Steam Production, the Chairman of the Nuclear Safety Audit and Review Board, and such others as the Chairman may designate.

6.5.2 NUCLEAR SAFETY AUDIT AND REVIEW BOARD (NSARB)

FUNCTION

6.5.2.1 The NSARB shall function to provide independent review and audit of designated activities in the areas of:

a. nuclear power plant operations

b. nuclear engineering

c. chemistry and radiochemistry

d. metallurgy

e. instrumentation and control

f. radiological safety

g. mechanical and electrical engineering

h. quality assurance practices

17

COMPOSITION

6.5.2.2 The composition of the NSARB shall be established as follows:

a. Chairman and Vice Chairman appointed by name by the Chairman of the Board and Chief Executive Officer of the Corporation or officer of his designation.

b. At least four technically qualified persons who are not members of the plant staff to provide expertise in the functional areas described in 6.5.2.1.

17

COMPOSITION (Continued)

c. At least one qualified non-company affiliated technical consultant and others as required. Duly appointed consultant members shall have equal vote with company affiliated members of the Board.

d. Two members from the supervisory staff of the R. E. Ginna Nuclear Power Plant staff. These two members shall serve as nonvoting members of the Nuclear Safety Audit and Review Board.

e. Members in (b) and (d) above to be designated by the Chairman of the Board and Chief Executive Officer.

ALTERNATES

6.5.2.3 Alternate members shall be appointed in writing by the NSARB Chairman to serve on a temporary basis; however, no more than two alternates shall participate in NSARB activities at any one time. 17

QUALIFICATIONS

6.5.2.4 The minimum qualifications of the Nuclear Safety Audit and Review Board **with regards to the individual members shall be maintained at a level equal to or higher than the following:**

(a) Reactor Engineering

Engineering graduate or equivalent with over eight years experience in the nuclear power field and over four years responsible engineering management.

QUALIFICATIONS (Continued)

(b) Utility Operations

Engineering graduate or equivalent with over eight years experience in utility operations and with over four years responsible engineering management.

(c) Reactor Physics

Physics graduate or equivalent with over five years experience in reactor physics work.

(d) Heat and Fluid Flow

Engineering or Physics graduate or equivalent with four years experience in heat and fluid flow analysis.

(e) Environmental Analysis

Engineering graduate or equivalent with over five years experience in environmental hazard analysis.

(f) Reactor Control and Instrumentation

Engineering graduate or equivalent with over five years experience in nuclear engineering.

(g) Power Plant Operations

Engineering graduate or equivalent with over five years experience in power plant operations.

(h) Safety Analysis

Engineering graduate or equivalent with over five years experience in nuclear engineering.

(i) Chemistry and Radiochemistry

Engineering graduate or equivalent with over five years experience in nuclear engineering

(j) Radiological Safety

Engineer graduate or equivalent with over five years experience in health physics and/or radiological safety.

17

MEETING FREQUENCY

6.5.2.5 **At least semi-annually and as required on call of the Chairman.**

QUORUM

6.5.2.6 A quorum shall consist of five members including the Chairman or Vice Chairman. At least one of the quorum shall be a non-company affiliated technical consultant.

REVIEW

6.5.2.7 The NSARB shall review:

a. **The safety evaluations for 1) changes to procedures, equipment or systems as described in the safety analysis report and 2) tests or experiments completed under the provision of 10 CFR Section 50.59 to verify that such actions did not constitute an unreviewed safety question.**

b. Proposed changes to procedures, equipment or systems which have been determined by the PORC to involve an unreviewed safety question as defined in 10 CFR Section 50.59.

c. Proposed tests or experiments which have been determined by the PORC to involve an unreviewed safety question as defined in 10 CFR Section 50.59. 17

d. Proposed changes in Technical Specifications or licenses.

e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.

f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.

g. **All events which are required by regulations or Technical Specifications to be reported to the NRC in writing within 24 hours.**

REVIEW (Continued)

h. Any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.

i. Reports and meeting minutes of the Plant Operations Review Committee.

AUDITS

6.5.2.8 The NSARB shall direct the establishment of an audit program and evaluate audits performed to ensure safe facility operation. Audits shall encompass:

a. The conformance of facility operation to all provisions contained within the Technical Specifications and applicable license conditions at least once per year. 17

b. **The performance, training and qualifications of the operating and technical staff at least once a year.**

c. The results of all actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.

d. The performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix B, 10 CFR 50, at least once per year.

e. The Radiation Emergency Plan and implementing procedures at least once per two years.

f. The Station Security Plan and implementing procedures at least once per two years.

AUDITS (Continued)

g. Any other area of facility operation considered appropriate by the NSARB or the Vice President, Electric and Steam Production.

AUTHORITY

6.5.2.9 17

a. The chairman of the Nuclear Safety Audit and Review Board is responsible to the Corporate Chairman of the Board on all activities for which the review board is responsible.

b. The NSARB shall report to and advise the Vice President, Electric and Steam Production, on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of NSARB activities shall be prepared, approved and distributed as indicated below:

a. Minutes shall be recorded of all meetings of this Board. Copies of the minutes shall be forwarded within 14 days following each meeting to the Corporate Chairman of the Board, Vice President, Electric and Steam Production and such others as the Chairman of the NSARB may designate. 17

b. Reports of reviews encompassed by Section 6.5.2.7 e, f, g and h above, shall be prepared, approved and forwarded to the (Vice President-Electric and Steam Production) within 14 days following completion of the review.

c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Corporate Chairman of the Board and to the management positions responsible for the areas audited within 30 days after completion of the audit. 17

NOV 3 1975

6.5.2.11 Written administrative procedures for committee operation shall be prepared and maintained describing the method of submission and the content of presentations to the committee, provisions for use of subcommittees, review and approval by members of written committee evaluations and recommendations, distribution of minutes, and such other matters as may be appropriate.

6.5.3 QUALITY ASSURANCE GROUP

6.5.3.1 The organization, qualifications, responsibilities and training of quality assurance personnel responsible for audits of safety related activities are described in the Quality Assurance Program.

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6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken in the event of a **Reportable Occurrence** as defined by Section 6.9.2:

a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.

b. Each Reportable Occurrence (as in 6.9.2.a) submitted to the Commission shall be reviewed by the PORC and submitted to the NSARB and the Vice President, Electric and Steam Production.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

a. The provisions of 10 CFR Section 50.36(c)(1)(i) shall be complied with immediately.

b. The Safety Limit violation shall be reported to the Vice President, Electric and Steam Production, to the NSARB and to the **Commission immediately.**

c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

d. The Safety Limit Violation Report shall be submitted to the Commission, the NSARB and the Vice President, Electric and Steam Production within two weeks of the violation.

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November 1972.

b. Refueling operations.

c. Surveillance and test activities of safety related equipment.

d. Security Plan implementation.

e. Emergency Plan implementation.

17

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the PORC and approved by the Station Superintendent prior to implementation and reviewed periodically as set forth in the applicable procedures.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

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a. The intent of the original procedure is not altered.

b. The change is approved by two members of the plant management staff, at least one of whom is the Shift Foreman who holds a Senior Reactor Operator's License.

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c. The change is documented, reviewed by the PORC, and approved by the Station Superintendent within 10 days of implementation.

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6.9 Reporting Requirements

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the Director of the appropriate Regional Office of Inspection and Enforcement unless otherwise noted.

6.9.1. Routine Reports

a. Startup Report. A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

b. Annual Operating Report.[1/] Routine operating reports covering the operation of the unit during the previous calendar year shall be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

The annual operating reports made by licensees shall provide a comprehensive summary of the operating experience gained during the year, even though some repetition of

previously reported information may be involved. References in the annual operating report to previously submitted reports shall be clear.

Each annual operating report shall include:

(1) A narrative summary of operating experience during the report period relating to safe operation of the facility, including safety-related maintenance not covered in item 1.b.(2)(e) below.

(2) For each outage or forced reduction in power[2/] of over twenty percent of design power level where the reduction extends for greater than four hours:

(a) the proximate cause and the system and major component involved (if the outage or forced reduction in power involved equipment malfunction);

(b) a brief discussion of (or reference to reports of) any reportable occurrences pertaining to the outage or power reduction;

(c) corrective action taken to reduce the probability of recurrence, if appropriate;

(d) operating time lost as a result of the outage or power reduction (for scheduled or forced outages,[3/] use the generator off-line hours; for forced reductions in power, use the approximate duration of operation at reduced power);

(e) a description of major safety-related corrective maintenance performed during the outage or power reduction, including the system and component involved and identification of the critical path activity dictating the length of the outage or power reduction; and

(f) a report of any single release of radioactivity or radiation exposure specifically associated with the outage which accounts for more than 10% of the allowable annual values.

(3) A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,[4/] e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

(4) Indications of failed fuel resulting from irradiated fuel examinations, including eddy current tests, ultrasonic tests, or visual examinations completed during the report period.

c. Monthly Operating Report. Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Office of Inspection and Enforcement U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the appropriate Regional Office, to arrive no later than the tenth of each month following the calendar month covered by the report.

6.9.2. Reportable Occurrences

Reportable occurrences, including corrective actions and measures to prevent reoccurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

a. Prompt Notification With Written Followup. The types of events listed below shall be reported as expeditiously as possible, but within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the appropriate Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

(1) Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.

Note: Instrument drift discovered as a result of testing need not be reported under this item but may be reportable under items 2.a(5), 2.a(6), or 2.b(1) below.

(2) Operation of the unit or affected systems when any parameter or operation subject to a limiting condition is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.

Note: If specified action is taken when a system is found to be operating between the most conservative and the least conservative aspects of a limiting condition for operation listed in the technical specifications, the limiting condition for operation is not considered to have been violated and need not be reported under this item, but it may be reportable under item 2.b(2) below.

(3) Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

Note: Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

(4) Reactivity anomalies, involving disagreement with the predicted value of reactivity balance under steady state conditions during power operation, greater than or equal to 1% $\Delta k/k$; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if sub-critical, an unplanned reactivity insertion of more than 0.5% $\Delta k/k$; or occurrence of any unplanned criticality.

(5) Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.

(6) Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.

Note: For items 2.a(5) and 2.a(6) reduced redundancy that does not result in a loss of system function need not be reported under this section but may be reportable under items 2.b(2) and 2.b(3) below.

(7) Conditions arising from natural or man-made events that, as a direct result of the event require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.

(8) Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

(9) Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

Note: This item is intended to provide for reporting of potentially generic problems.

b. Thirty Day Written Reports. The reportable occurrences discussed below shall be the subject of written reports to the Director of the appropriate Regional Office within thirty days of occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

(1) Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

(2) Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

Note: Routine surveillance testing, instrument calibration, or preventative maintenance which require system configurations as described in items 2.b(1) and 2.b(2) need not be reported except where test results themselves reveal a degraded mode as described above.

(3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.

(4) Abnormal degradation of systems other than those specified in item 2.a(3) above designed to contain radioactive material resulting from the fission process.

Note: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

6.9.3. Unique Reporting Requirements

a. Environmental Monitoring

An Annual Environmental Operating Report covering the results of the environmental monitoring program during the previous calendar year shall be submitted prior to March 1 of each year. Each Annual Environmental Operating Report shall include:

(1) Descriptive material covering the off-site environmental surveys performed during the reporting period including information on:

(a) The number and types of samples taken; e.g., air, lake bottom, surface water, milk, soil, biota.

(b) The number and types of measurements made: e.g., dosimetry.

(c) Locations of the sample points and monitoring stations.

(d) The frequency of the surveys.

(e) A summary of survey results.

(2) If a particular sample or measurement indicates statistically significant levels of radioactivity above established or concurrent backgrounds, the following information shall be provided:

(a) The type of analysis performed; e.g., alpha, beta, gamma, and/or isotopic.

(b) The minimum sensitivity of the monitoring system.

(c) The measured radiation level or sample concentration.

(d) The applicable established limit.

(e) The specific times when samples were taken and measurements were made.

(f) An estimate of the likely resultant exposure to the public.

b. Semi-annual Effluent Release Report

Within 60 days after January 1 and July 1 of each year a report shall be submitted covering the radioactive content of effluents released to unrestricted areas during the previous six months of operation. The data shall be summarized on a monthly basis and include as a minimum:

(1) Radioactive Liquid Waste

(a) Gross radioactivity (β,γ) released (in curies) and average concentration released to the unrestricted area.

(b) Total tritium and alpha radioactivity (in curies) released and average concentration released to the unrestricted area.

(c) Total dissolved gas radioactivity (in curies) and average concentration released to the unrestricted area.

(d) Total volume (in liters) of liquid waste released.

(e) Total volume (in liters) of dilution water used prior to release from the restricted area.

(f) The maximum concentration of gross radioactivity (β,γ) released to the unrestricted area (averaged over the period of release).

(g) Total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.

(h) Percent of Technical Specification limit for total activity released.

(2) Gaseous Waste

(a) Total radioactivity (in curies) releases of noble and activation gases.

(b) Maximum noble gas release rate during any one-hour period.

(c) Total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.

(d) Percent of Technical Specification limit.

(3) Iodine Releases

(a) Total (I-131, I-133, I-135) radioactivity (in curies) released.

(b) Total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.

(c) Percent of Technical Specification limit.

(4) Particulate Releases

(a) Gross radioactivity (β,γ) released (in curies) excluding background radioactivity.

(b) Gross alpha radioactivity released (in curies) excluding background radioactivity.

(c) Total radioactivity released (in curies) of nuclides with half-lives greater than eight days.

(d) Percent of Technical Specification limit.

(5) Solid Radioactive Waste

(a) Total volume (in cubic feet) of solid waste generated.

(b) Gross curie activity involved.

(c) Dates and disposition of the material if shipped offsite.

(c) Total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.

(d) Percent of Technical Specification limit.

(3) Iodine Releases

(a) Total (I-131, I-133, I-135) radioactivity (in curies) released.

(b) Total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.

(c) Percent of Technical Specification limit.

(4) Particulate Releases

(a) Gross radioactivity (β,γ) released (in curies) excluding background radioactivity.

(b) Gross alpha radioactivity released (in curies) excluding background radioactivity.

(c) Total radioactivity released (in curies) of nuclides with half-lives greater than eight days.

(d) Percent of Technical Specification limit.

(5) Solid Radioactive Waste

(a) Total volume (in cubic feet) of solid waste generated.

(b) Gross curie activity involved.

(c) Dates and disposition of the material if shipped offsite.

FOOTNOTES

1. A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

2. The term "forced reduction in power" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the load on the unit be reduced for corrective action immediately or up to and including the very next weekend. Note that routine preventive maintenance, surveillance and calibration activities requiring power reductions are not covered by this section.

3. The term "forced outage" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the unit be removed from service for corrective action immediately or up to and including the very next weekend.

4. This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

6.10 RECORD RETENTION

In accordance with Rochester Gas and Electric Corporation policy, operating charts for the first year's operation will be permanently stored.

6.10.1 The following records shall be retained for at least five years:

a. Records and logs of facility operation, including power levels and periods of operation at each power level. | 17

b. Records and logs of principal maintenance activities, including inspection, repair, substitution or replacement of principal items of equipment pertaining to nuclear safety. | 17

c. Reportable Occurrence Reports. | 17

d. Records of surveillance activities, inspections, and calibrations required by these Technical Specifications. | 17

e. Records of reactor tests or experiments. | 17

f. Records of changes made in the Operating Procedures.

g. Records of sealed source leak tests and results.

h. Records of annual physical inventory of all sealed source material of record. | 17

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report; changes shall also be periodically incorporated into the as-built file.

b. Records of new and irradiated fuel inventory, fuel transfers, and assembly burnup histories. | 17

c. Records of plant radiation and contamination surveys.

d. Records of off-site environmental monitoring surveys.

e. Records of radiation exposure of all plant personnel, including all contractors and visitors to the plant who enter radiation control areas.

f. Records of radioactivity in liquid and gaseous **material released** to the environment and radioactive waste shipments.

g. Records of transient or operational cycles for those facility components designed for a limited number of transients or cycles.

h. Records of training and qualification for current station technical and operations staff members.

i. Records of in-service inspections performed pursuant to these Technical Specifications.

j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR Section 50.59

k. Records of meetings of the PORC and the NSARB.

l. Records of Quality Assurance activities as required by the QA Manual.

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6.11 RADIATION PROTECTION PROGRAM

Radiation control procedures shall be prepared and made available to all station personnel or other persons who may be subject to radiation exposure at the station. These procedures shall show permissible radiation exposure, and shall be consistent with the requirements of 10 CFR Part 20. The radiation protection program shall be organized and maintained to meet the requirements of 10 CFR Part 20, with exceptions set forth in Sections 6.12 and 6.13 of these Technical Specifications. The program shall be adhered to for all operations involving personnel radiation exposure. 17

6.12 RESPIRATORY PROTECTION PROGRAM

ALLOWANCE

6.12.1 Pursuant to 10CFR20. 103(c) (1) and (3), allowance may be made for the use of respiratory protective equipment in conjunction with activities authorized by the operating license for this plant in determining whether individuals in restricted areas are exposed to concentrations in excess of the limits specified in Appendix B, Table I, Column 1 of 10CFR20, subject to the following conditions and limitations:

a. The limits provided in Section 20.103 (a) and (b) are not exceeded.

b. If the radioactive material is of such form that intake through the skin or other additional route is likely, individual exposures to radioactive material shall be controlled so that the radioactive content of any critical organ from all routes of intake averaged over 7 consecutive days does not exceed that which would result from inhaling such radioactive material for 40 hours at the pertinent concentration values provided in Appendix B, Table I, Column I of 10CFR20.

c. For radioactive materials designated "Sub" in the "Isotope" column of Appendix B, Table I, Column I of 10CFR20, the concentration value specified is based upon exposure to the material as an external radiation source. Individual exposures to these materials shall be accounted for as part of the limitation on individual dose in Part 20.101. These materials shall be subject to applicable process and other engineering controls. | 17

| 17

6.12.2 In all operations in which adequate limitation of the inhalation of radioactive material by the use of process or other engineering controls is impracticable, the licensee may permit an individual in a restricted area to use respiratory protective equipment to limit the inhalation of airborne radioactive material provided:

a. The limits specified in Section 6.12.1 are not exceeded. | 17

b. Respiratory protective equipment is selected and used so that the peak concentrations of airborne radioactive material inhaled by an individual wearing the equipment does not exceed the pertinent concentration values specified in Appendix B, Table I, Column 1 of 10CFR20. For the purposes of this subparagraph, the concentration of radioactive material that is inhaled when respirators are worn may be determined by dividing the ambient airborne concentration by the protection factor specified in Table 6.12-1 appended to this specification for the respiratory protective equipment worn. If the intake of radioactivity is later determined by other measure- | 17

| 17

ments have been different from that initially estimated, the later quantity shall be used in evaluating the exposures.

c The licensee shall advise each respirator user that he may leave the area at any time for relief from respirator use in case of equipment malfunction, physical or psychological discomfort, or any other condition that might cause reduction in the protection afforded the wearer.

d The licensee shall maintain a respiratory protective program adequate to assure that the previously stated requirements are met. Such a program shall include the following practices recommended by the American National Standards Institute (ANSI-Z88.2-1969):

1. Air sampling and other surveys sufficient to identify the hazard, to evaluate individual exposure, and to permit proper selection of respiratory protective equipment.
2. Written procedures to assure proper selection, supervision, and training of personnel using such protective equipment.
3. Written procedures to assure the adequate fitting of respirators and the testing of respiratory protective equipment for operability **immediately prior to use.**
4. Written procedures for maintenance to assure full effectiveness of respiratory protective equipment, including issuance, cleaning and decontamination, inspection, repair, and storage.

5. Written operational and administrative procedures for proper use of respiratory protective equipment including provisions for planned limitations on working times as necessitated by operational conditions.

6. Bioassays and/or whole body counts (and other surveys, as appropriate) of individuals to evaluate exposures and to assess protection actually provided.

e. The licensee shall use equipment approved by the U.S. Bureau of Mines under its appropriate Approval Schedules as set forth in Table 6.12-1. Equipment not approved under U.S. Bureau of Mines Approval Schedules may be used only if the licensee has evaluated the equipment and can demonstrate by testing, or on the basis of reliable test information, that the material and performance characteristics of the equipment are at least equal to those afforded by U.S. Bureau of Mines approved equipment of the same type, as specified in Table 6.12-1.

f. The licensee shall not assign protection factors in excess of those specified in Table 6.12-1 in selecting and using respiratory equipment, unless otherwise authorized by the Commission.

REVOCATION

6.12.3 The specifications of Section 6.12 shall be revoked in their entirety upon adoption of the proposed change to 10 CFR Section 20.103, which would make such provisions unnecessary.

TABLE 6.12-1
PROTECTION FACTORS FOR RESPIRATORS

Description	MODES[1/]	Protection Factors [2/] Particulates and Vapors and Gases Except Tritium Oxide [3/]	Guides to Selection of Equipment Bureau of Mines/National Institute For Occupational Safety and Health Approvals
I. Air-Purifying Respirators			
Facepiece, half-mask [4/] [7/]	NP	5	30 CFR Part 11 Subpart K
Facepiece, full [7/]	NP	100	30 CFR Part 11 Subpart K
II. Atmosphere-Supplying Respirator			
1. Air line respirator			
Facepiece, half-mask	CF	100	30 CFR Part 11 Subpart J
Facepiece, full	CF	1,000	30 CFR Part 11 Subpart J
Facepiece, full [7/]	D	100	30 CFR Part 11 Subpart J
Facepiece, full	PD	1,000	30 CFR Part 11 Subpart J
Hood	CF	[5/]	30 CFR Part 11 Subpart J
Suit	CF	[5/]	[6/]
2. Self-contained breathing apparatus (SCBA)			
Facepiece, full [7/]	D	100	30 CFR Part 11 Subpart H
Facepiece, full	PD	1,000	30 CFR Part 11 Subpart H
Facepiece, full	R	100	30 CFR Part 11 Subpart H
III. Combination Respirator Any combination of air-purifying and atmosphere-supplying respirator		Protection factor for type and mode of operation as listed above.	30 CFR Part 11 §11.63(b)

[1/], [2/], [3/], [4/], [5/], [6/], [7/]. (These notes are on the following pages)

1/ See the following symbols:

CF: continuous flow

D : demand

NP: negative pressure (i.e., negative phase during inhalation)

PD: pressure demand (i.e., always positive pressure)

R : recirculating (closed circuit)

2/ (a) For purposes of this specification the protection factor is a measure of the degree of protection afforded by a respirator, defined as the ratio of the concentration of airborne radioactive material outside the respiratory protective equipment to that inside the equipment (usually inside the facepiece) under conditions of use. It is applied to the ambient airborne concentration to estimate the concentration inhaled by the wearer according to the following formula:

$$\text{Concentration Inhaled} = \frac{\text{Ambient Airborne Concentration}}{\text{Protection Factor}}$$

(b) The protection factors apply:

(i) only for trained individuals wearing properly fitted respirators used and maintained under supervision in a well-planned respiratory protective program.

(ii) for air-purifying respirators only when high efficiency [above 99.9% removal efficiency by U.S. Bureau of Mines type dioctyl phthalate (DOP) test] particulate

filters and/or sorbents appropriate to the hazard are used in atmospheres not deficient in oxygen.

(iii) for atmosphere-supplying respirators only when supplied with adequate respirable air.

3/ Excluding radioactive contaminants that present an absorption or submersion hazard. For tritium oxide approximately half of the intake occurs by absorption through the skin so that an overall protection factor of not more than approximately 2 is appropriate when atmosphere-supplying respirators are used to protect against tritium oxide. Air-purifying respirators are not recommended for use against tritium oxide. See also footnote 5/ below, concerning supplied-air suits and hoods.

4/ Under chin type only. Not recommended for use where it might be possible for the ambient airborne concentration to reach instantaneous values greater than 50 times the pertinent values in Appendix B, Table I, Column 1 of 10CFR, Part 20.

5/ Appropriate protection factor must be determined taking account of the design of the suit or hood and its permeability to the contaminant under conditions of use. No protection factor greater than 1,000 shall be used except as authorized by the Commission.

6/ No approval schedules currently available for this equipment. Equipment must be evaluated by testing or on basis of available test information.

7/ Only for shaven faces.

Note 1: Protection factors for respirators, as may be approved by the U.S. Bureau of Mines and/or NIOSH according to approval schedules for respirators to protect against airborne radionuclides, may be used to the extent that they do not exceed the protection factors listed in this Table. The protection factors in this Table may not be appropriate to circumstances where chemical or other respiratory hazards exist in addition to radioactive hazards. The selection and use of respirators for such circumstances should take into account approvals of the U.S. Bureau of Mines and/or NIOSH in accordance with its applicable schedules. | 17

Note 2: Radioactive contaminants for which the concentration values in Appendix B, Table I, 10 CFR Part 20 are based on internal dose due to inhalation may, in addition, present external exposure hazards at higher concentrations. Under such circumstances, limitations on occupancy may have to be governed by external dose limits. | 17

6.13 HIGH RADIATION AREA

6.13.1

In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20: 17

a. Each High Radiation Area in which the intensity of radiation is 1000 mrem/hr or less shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit, and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.

b. Each High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.13.1 a. above, and in addition locked doors shall be provided to 17 prevent unauthorized entry into these areas and the keys to these locked doors shall be maintained under the administrative control of the Shift Foreman on duty.

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 8 TO PROVISIONAL LICENSE NO. DPR-18

CHANGE NO. 17 TO TECHNICAL SPECIFICATIONS

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

Introduction

By letter dated December 2, 1974, Rochester Gas and Electric Corporation proposed changes to the Technical Specifications appended to Provisional Operating License No. DPR-18, for the R. E. Ginna Nuclear Power Plant. The proposed changes involve changes to the administrative controls including changes to the reporting requirements.

Discussion

The proposed changes would be administrative in nature and would affect the conduct of operation. The proposed changes are intended to provide uniform license requirements. Areas covered by the proposed uniform specifications include licensee staffing qualifications and management procedures involved with operating the reactor, reporting requirements, abnormal occurrence definition change, and a respiratory protection program modification.

Members of the facility staff should meet the requirements set forth in Guide 1.8, "Personnel Selection and Training" which endorses proposed ANSI N18.1, which was subsequently issued as ANSI N18.1-1971. Provisions for independent review of facility operations should be in accord with Guide 1.33, "Quality Assurance Program Requirements" which endorses proposed standard ANS 3.2, which was subsequently issued as ANSI 18.7-1972.

In Section 208 of the Energy Reorganization Act of 1974 "abnormal occurrences" is defined as an unscheduled incident or event which the Commission determines is significant from the standpoint of public health or safety. The term "abnormal occurrence" is reserved for usage by NRC. Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4, enumerates required

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reports consistent with Section 208. The proposed change to required reports identifies the reports required of all licensees not already identified by the regulations and those unique to this facility. The proposal would formalize present reporting and would delete any reports no longer needed for assessment of safety related activities. In addition, a radiation protection program delineates use of respiratory equipment in the event personnel are to be exposed to concentrations in excess of Part 20 concentrations.

Evaluation

The new guidance for reporting operating information does not identify any event as an "abnormal occurrence". The proposed reporting requirements also delete reporting of information no longer required and duplication of reported information. The standardization of required reports and desired format for the information will permit more rapid recognition of potential problems.

Identifying minimum acceptable qualifications for facility personnel should assure capable performance from the facility staff. Other administrative requirements also restated by the specifications assure uniformity and conformance to the desired features in the review, staffing, and procedures. Modification to the currently accepted respiratory protection program at this time assures that a consistent method of using respiratory equipment is immediately available whenever needed. Similar changes are being approved for all power reactor licensees, so all licensees will have the same requirements presented in a uniform manner.

During our review of the proposed changes, we found that certain modifications to the proposal were necessary to have conformance with the desired regulatory position. These changes were discussed with the licensee's staff and have been incorporated into the proposal.

We have concluded that the proposal as modified improves the licensee's program for evaluating plant performance and the reporting of the operating information needed by the Commission to assess safety related activities and is acceptable. The facility staff qualifications and training program conform to Guide 1.8 and therefore are acceptable. The administrative procedures and facility review and audit are consistent with Guide 1.33 and are acceptable. The modified reporting program is consistent with the guidance provided by Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4. The administrative controls are consistent with requirements being incorporated in Technical Specifications for new licensed facilities.

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Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

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UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-244

ROCHESTER GAS AND ELECTRIC CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL OPERATING LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 8 to Provisional Operating License No. DPR-18 issued to Rochester Gas and Electric Corporation which revised Technical Specifications for operation of the R. E. Ginna Nuclear Power Plant located in Wayne County, New York. The amendment becomes effective 30 days after the date of issuance.

This amendment modifies the Administrative Controls, Reporting Requirements, and Respiratory Protection Program of the Technical Specifications for the R. E. Ginna Nuclear Power Plant.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

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For further details with respect to this action, see (1) the application for amendment dated December 2, 1974, (2) Amendment No. 8 to License No. DPR-18, with Change No. 17, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C., and at the Lyons Public Library, 67 Canal Street, Lyons, New York and the Rochester Public Library, 115 South Avenue, Rochester, New York.

A copy ot items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this NOV 3 1975

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by
R. A. Purple

Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Reactor Licensing

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