Docket 716

Docket No. 50-244

May 17, 1977

Rochester Gas & Electric Corporation
ATTN: Mr. Leon D. White, Jr.
Vice President
Electric & Steam Production
89 East Avenue
Rochester, New York 14604

Gentlemen:

The Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. 13 to Provisional Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment revises the provisions of the Technical Specifications in response to your request of February 27, 1976, by replacing the existing inservice inspection requirements in Section 4.2 with requirements in conformance with 10 CFR 50, Section 50.55a(g).

We have also completed our review of your inservice inspection program description (Appendix B to the Ginna Station Quality Assurance Manual) submitted as an attachment to your letter dated February 27, 1976 and Revisions 1 and 2 thereto submitted by letters dated September 23, 1976 and February 8, 1977, as well as supplemental information provided in your letter dated April 13, 1977. Your proposed inservice inspection program includes a request for relief from certain ASME Code requirements. This inservice inspection program is applicable to the 40 month period which began on September 1, 1976. The first inspections under this program are being conducted during the present refueling outage for Cycle 7.

Based on our review, we have concluded that your proposed inservice inspection program description conforms with the 1974 Edition of the ASME Code, Section XI and Addenda through Summer 1975 to the extent practical for your facility within the limitations of design, geometry and the materials of construction of the components; and thus is acceptable. In addition, pursuant to 10 CFR 50.55a(g)(6)(i), we hereby grant relief from the ASME Code requirements that are identified in Enclosure 1. We are granting this relief based on our review of the information you submitted to support your determinations that these ASME Code requirements would be impractical for your facility, and determination that the granting of this relief is authorized by law and will not endanger life or property or the common defense and security and will otherwise be in the public interest. In making this determination we have given due consideration to the burden that could result if these requirements were imposed on the facility. We have also determined that the granting of this relief does not involve a significant increase in the probability or consequences of accidents previously considered nor a decrease in safety margin; and thus, does not involve a significant hazards consideration.

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SURNAME➤						
DATE➤						

Furthermore, we have determined that the granting of this relief from ASME Code requirements does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. We have concluded that the granting of this relief is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with this action.

The relief from ASME Code requirements granted by Items 1, 2, 3 and 5 of Enclosure 1 to this letter shall remain in effect until specifically revoked by the NRC or until the end of the 120-month period beginning January 1, 1970 for ASME Code Class 1 (Quality Group A) components and May 1, 1973 for ASME Code Class 2 and 3 (Quality Groups B & C) components. If, at that time, you wish this relief to be reinstated, you must submit information to support the continued impracticality of the requirements for your facility at least 90 days before the end of the 120-month period. The NRC will evaluate the basis for the determination that the requirements are still impractical, pursuant to 50.55a(g)(5)(iv). This reevaluation will take into account any advances in the state-of-the-art of inservice inspection techniques that may have occurred between now and the end of the 120-month period. The relief granted in Item 4 of Enclosure 1 shall remain in effect until December 31, 1978. At that time we will reassess the suitability of the UT evaluation criterion.

Finally, in regard to that portion of your inservice inspection program that deals with the pump and valve testing requirements of Section XI of the ASME Boiler and Pressure Vessel Code, we requested, by letter dated December 17, 1976, additional information to evaluate the relief you are seeking for certain pumps and valves. We asked that you supply the detailed information within 120 days. We have received your response dated April 6, 1977.

We conclude that the surveillance programs now in effect in the Ginna Technical Specifications concerning pump and valve testing provide adequate interim control to assure pump and valve reliability and operational readiness prior to implementation of the test program in conformance with 10 CFR 50.55a(g). Accordingly, an exemption from the initial implementation date for the pump and valve testing requirement of 10 CFR Section 50.55a(g) is hereby granted until September 1, 1977.

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SURNAME➤						
DATE➤						

Based on the foregoing, we have determined that, pursuant to 10 CFR Section 50.12, a specific exemption as discussed above can be granted without endangering life or property or the common defense and security and is otherwise in the public interest. In making this determination we have given due consideration to the burden that could result if these requirements were imposed on the facility. We have also determined that the granting of this exemption does not involve a significant increase in the probability or consequences of accidents previously considered nor a decrease in safety margin; and thus, does not involve a significant hazards consideration.

Furthermore, we have determined that the granting of this exemption does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. We have concluded that the granting of this exemption is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(d)(4) that an environmental impact statement, or negative declaration and environmental impact appraisal, need not be prepared in connection with this action.

Copies of the Safety Evaluation supporting the Amendment and the Notice of Issuance relating to these actions are also enclosed.

Sincerely,

Original Signed by
Victor Stello, Jr., Director
Division of Operating Reactors
Office of Nuclear Reactor Regulation

Enclosures:
1. List of ASME Code Requirements for Which Relief is Granted
2. Amendment No. to DPR-18
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
See next page

FOR DISTRIBUTION & CONCURRENCES SEE ATTACHED YELLOW.

OFFICE➤						
SURNAME➤						

requirements were imposed on the facility. We have also determined that the granting of this exemption does not involve a significant increase in the probability or consequences of accidents previously considered nor a decrease in safety margin; and thus, does not involve a significant hazards consideration.

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DATE	4/14/77	4/21/77	4/29/77	5/4/77	5/4/77	/77

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 17, 1977

Docket No. 50-244

Rochester Gas & Electric Corporation
ATTN: Mr. Leon D. White, Jr.
Vice President
Electric & Steam Production
89 East Avenue
Rochester, New York 14604

Gentlemen:

The Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. 13 to Provisional Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment revises the provisions of the Technical Specifications in response to your request of February 27, 1976, by replacing the existing inservice inspection requirements in Section 4.2 with requirements in conformance with 10 CFR 50, Section 50.55a(g).

We have also completed our review of your inservice inspection program description (Appendix B to the Ginna Station Quality Assurance Manual) submitted as an attachment to your letter dated February 27, 1976 and Revisions 1 and 2 thereto submitted by letters dated September 23, 1976 and February 8, 1977, as well as supplemental information provided in your letter dated April 13, 1977. Your proposed inservice inspection program includes a request for relief from certain ASME Code requirements. This inservice inspection program is applicable to the 40 month period which began on September 1, 1976. The first inspections under this program are being conducted during the present refueling outage for Cycle 7.

Based on our review, we have concluded that your proposed inservice inspection program description conforms with the 1974 Edition of the ASME Code, Section XI and Addenda through Summer 1975 to the extent practical for your facility within the limitations of design, geometry and the materials of construction of the components; and thus is acceptable. In addition, pursuant to 10 CFR 50.55a(g)(6)(i), we hereby grant relief from the ASME Code requirements that are identified in Enclosure 1. We are granting this relief based on our review of the information you submitted to support your determinations that these ASME Code requirements would be impractical for your facility, and determination that the granting of this relief is authorized by law and will not endanger life or property or the common defense and security and will otherwise be in the public interest. In making this determination we have given due consideration to the burden that could result if these requirements were imposed on the facility. We have also determined that the granting of this relief does not involve a significant increase in the probability or consequences of accidents previously considered nor a decrease in safety margin; and thus, does not involve a significant hazards consideration.

Furthermore, we have determined that the granting of this relief from ASME Code requirements does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. We have concluded that the granting of this relief is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with this action.

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Finally, in regard to that portion of your inservice inspection program that deals with the pump and valve testing requirements of Section XI of the ASME Boiler and Pressure Vessel Code, we requested, by letter dated December 17, 1976, additional information to evaluate the relief you are seeking for certain pumps and valves. We asked that you supply the detailed information within 120 days. We have received your response dated April 6, 1977.

We conclude that the surveillance programs now in effect in the Ginna Technical Specifications concerning pump and valve testing provide adequate interim control to assure pump and valve reliability and operational readiness prior to implementation of the test program in conformance with 10 CFR 50.55a(g). Accordingly, an exemption from the initial implementation date for the pump and valve testing requirement of 10 CFR Section 50.55a(g) is hereby granted until September 1, 1977.

Based on the foregoing, we have determined that, pursuant to 10 CFR Section 50.12, a specific exemption as discussed above can be granted without endangering life or property or the common defense and security and is otherwise in the public interest. In making this determination we have given due consideration to the burden that could result if these requirements were imposed on the facility. We have also determined that the granting of this exemption does not involve a significant increase in the probability or consequences of accidents previously considered nor a decrease in safety margin; and thus, does not involve a significant hazards consideration.

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Copies of the Safety Evaluation supporting the Amendment and the Notice of Issuance relating to these actions are also enclosed.

Sincerely,

Victor Stello, Jr., Director
Division of Operating Reactors
Office of Nuclear Reactor Regulation

Enclosures:
1. List of ASME Code Requirements for Which Relief is Granted
2. Amendment No. 13 to DPR-18
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
See next page

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Chief, Energy Systems
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Washington, D.C. 20460

U.S. Environmental Protection Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, New York 10007

ENCLOSURE NO. 1

LIST OF ASME CODE REQUIREMENTS FOR WHICH RELIEF IS GRANTED PURSUANT TO 10 CFR 50.55a(g)(6)(i) FOR R. E. GINNA NUCLEAR PLANT

Based on the information submitted by Rochester Gas and Electric Corporation and our review of the design, geometry, and materials of construction of the R. E. Ginna Nuclear Power Plant, certain requirements of the ASME Boiler and Pressure Vessel Code, Section XI, have been determined to be impractical.

Relief from those requirements is, therefore, granted as follows:

1. Substitute dye penetrant test for volumetric examination for reactor coolant pump casing welds. (Item B5.6, Examination Category B-L-1, Table IWB-2600).

   Bases

   a. No practical techniques are presently available for volumetric examination of the Ginna reactor coolant pump casing welds.

      1. Ultrasonic testing cannot be used because of the large variation in UT response from the heavy walled casing and thick weld.

      2. Meaningful radiographic testing cannot be performed because of the high background radiation field (e.g. 15 to 17 R/hr on internal surfaces and 400 mr/hr field external to the pump). Not only is the resolution of the radiograph adversely affected but the exposure to test personnel would be unacceptably high if disassembly of the pump for decontamination was attempted for this purpose.

   b. RG&E has committed to incorporate a new technique into the inservice inspection program whenever it has been developed. Extensive research and development work is going on and it is expected that a reliable technique will be available soon.

   c. Based on design, fabrication, and accessibility considerations, it is judged as an interim measure that a dye penetrant test for these pump casing welds can provide an adequate level of assurance that the integrity of this component will be maintained throughout this inspection period.

2. Substitute surface examination for volumetric examination for piping integrally welded supports (Item B4.9, examination category B-K-1 Table IWB-2600).

   Bases

   These welds are not full penetration welds by design and therefore are difficult to be examined meaningfully by the volumetric method. Further, because of the loading condition, the flaw would most likely generate at the surface. Thus, surface examination is considered to provide a comparable level of assurance of this support integrity and is therefore acceptable.

3. Repair Quality Groups A, B and C (Code Class 1, 2 and 3) components in accordance with applicable requirements of Section III of the ASME B&PV Code except for the N-Stamp requirement instead of those of Section XI Code.

   Bases

   a. Considering the design basis for current fabrication and installation requirements, the repair requirements specified in Section 6.0 of Appendix B are considered to provide a comparable level of assurance of the repaired component integrity and are therefore acceptable.

   b. The repair requirements for Section XI are generally comparable to that for Section III. In fact, Section III acceptance standards for volumetric examination are generally more restrictive than Section XI. Section III does not accept cracks of any size whereas Section XI accepts a crack as long as it does not exceed the allowable standards specified in the Section XI Code.

4. Use 100% of the reference level as the evaluation criterion for indications detected during ultrasonic examination of components other than ferritic vessels and piping systems instead of 20% of the reference level recommended in Article 5 of Section V.

   Bases

   a. Using 20% of the reference level as the evaluation criterion for indications is judged to be impractical since the inspection history at Ginna has shown that the level of "noise" or "hash" in the UT response from the Class 1 system inspections has typically been 20% to 30% and up to 40% from the Class 2 system inspections.

b. The NRC is currently reassessing the effectiveness of the code UT procedures and is intending to issue a regulatory guide to further improve the reliability of UT technique. This relief will remain in effect until December 31, 1978. By that time we expect to have established new criteria and will require those to be met for the next 10 year interval.

c. The 100% of the reference level evaluation criterion as an interim measure is judged acceptable because:

1. Indications of this level found during examination have been sufficiently reliable to detect flaws.

2. 50% reference level recording criterion committed by RG&E establishes a permanent history which can be examined later.

5. Use the Ginna Station Quality Assurance Program to provide the administrative control requirements in lieu of those administrative functions that would be performed by the "Enforcement Authority", and "Authorized Inspector" defined in Section XI of the ASME Boiler and Pressure Vessel Code.

Bases

a. The Ginna Plant is located in New York, a State which has not endorsed Section XI and therefore the administrative organization and controls such as "Enforcement Authority", "Authorized Inspector", and "Reporting Systems" are not provided by the State.

b. RG&E's program for the inservice inspection, governed by the R. E. Ginna Station Quality Assurance Manual, contains the requirements and responsibilities for the implementation of the program in procedures. The procedures have been prepared and approved by the responsible organizations within RG&E, i.e. Ginna Station, Engineering, General Maintenance, Electric Meter and Laboratory and Purchasing.

c. The functions of the ASME authorized inspector, viz. their reviews and verifications, will be performed by personnel of the Hartford Steam Boiler Inspection and Insurance Company. The qualifications of the inspectors, inspection specialists and inspection agency are in compliance with the Code.

d. Examination techniques have been established in accordance with written requirements and incorporated into written procedures. Qualifications for non-destructive test personnel are in compliance with Regulatory Guide 1.58 "Qualification of Nuclear Power Plant Inspection, Examination and Testing Personnel".

e. Procedures will be implemented to control the standards for examination evaluation. These procedures include the indentifications of the organization performing the inspection, description of the method of inspection to be used, acceptance and rejection criteria and requirements for providing evidence of completion and certification of the inspection activity.

   Procedures will be developed by the Ginna Engineering Organization to prescribe the disposition of non-conformances. The procedures implemented for the repairs, the retest procedures and the test results will be reviewed by the Nuclear Safety Audit and Review Board. The members of this Board include technically qualified RG&E personnel, Ginna Plant staff members and qualified consultants.

f. Records and reports of the inservice inspection will be developed and maintained by RG&E and are to include such items as examination plans and schedules, examination results and reports, examination methods and procedures, evaluation of results and corrective actions.

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 13
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated February 27, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility License No. DPR-18 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 13, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 17, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 13

PROVISIONAL OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A as follows:

Replace pages 4.2-1 and 4.2-2 with the enclosed pages 4.2-1 and 4.2-2.

4.2 **Inservice Inspection**

Applicability

Applies to the inservice inspection of Quality Groups A, B and C Components, High Energy Piping Outside of Containment and Steam Generator Tubes.

Objectives

To provide assurance of the continuing structural and operational integrity of the structures, components and systems in accordance with the requirements of 10 CFR 50.55a(g).

Specification

4.2.1 The inservice inspection program for Quality Groups A, B and C Components, High Energy Piping Outside of Containment and Steam Generator tubes shall be in accordance with Appendix B of the Ginna Station Quality Assurance Manual. This inservice inspection program shall define the specific requirements of the edition and Addenda of the ASME Boiler and Pressure Vessel Code, Section XI, which are applicable for the forty month period of the ten year inspection interval. The program ten year inspection intervals shall be based on the following commencing dates.

4.2.1.1 The inspection interval for Quality Group A components shall be ten year intervals of service commencing on January 1, 1970.

4.2.1.2 The inspection intervals for Quality Group B and C Components shall be ten year intervals of service commencing with May 1, 1973.

4.2.1.3 The inspection intervals for the High Energy Piping Outside of Containment shall be ten year intervals of service commencing with May 1, 1973. The inspection program during each third of the first inspection interval provides for examination of all welds at design basis break locations and one-third of all welds at locations where a weld failure would result in unacceptable consequences. During each succeeding inspection interval, the program shall provide for an examination of each of the design basis break location welds, and each of the welds at locations where a weld failure would result in unacceptable consequences.

4.2.1.4 The inspection intervals for Steam Generator Tubes shall be specified in the "Inservice Inspection Program" for the applicable forty month period commencing with May 1, 1973.

4.2.1.5 Inservice Inspection of ASME Code Class 1, Class 2 and Class 3 components (Quality Groups A, B and C) shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).

Basis:

The inservice inspection program provides assurance for the continued structural integrity of the structures, components and systems of Ginna Station. The program complies with the ASME Boiler and Pressure Vessel Code Section XI "Rules for Inservice Inspection of Nuclear Power Plant Components" as practicable, with due consideration to the design and physical access of the structures, components and systems as manufactured and constructed. This compliance will constitute an acceptable basis for satisfying the requirements of General Design Criterion 32, Appendix A of 10 CFR Part 50 and the requirements of Section 50.55a, paragraph g of 10 CFR Part 50.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 13 TO PROVISIONAL OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

Introduction

Paragraph 50.55a(g) contains provisions that require inservice inspection and testing of ASME Code Class 1, 2 and 3 nuclear power plant components (including supports) to be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda. On February 27, 1976, Rochester Gas and Electric Corporation (RG&E) submitted an application for an amendment to Provisional Operating License DPR-18 for the R. E. Ginna Nuclear Plant (Ginna) to change the Technical Specifications to conform to the revised Regulation. The proposed inservice inspection program description for Ginna was submitted with the application. During the course of our review of this inservice inspection program description (Appendix B to the Ginna Station Quality Assurance Manual), RG&E made two revisions to accommodate our comments. Revision 1 was submitted by letter dated September 23, 1976 and Revision 2 was submitted by letter dated February 8, 1977. In addition, information regarding the use of independent inspection personnel was submitted by letter dated April 13, 1977.

Evaluation

10 CFR 50.55a(g) requires that the Technical Specifications for a facility must be revised to conform to the requirements of the Regulation. The change being issued accomplishes this requirement by specifying that the inservice inspection program for Ginna shall be performed in accordance with the requirements of 10 CFR 50, Section 50.55a(g). For those items in the existing inservice inspection program for Ginna that have more stringent inspection requirements than required by 10 CFR 50, Section 50.55a(g), we have retained the existing requirements in the Technical Specifications being issued. These involve the augmented inservice inspection of high energy piping welds outside of containment and steam generator tubes. For the present 40 month inspection interval which commenced on September 1, 1976, we have granted relief from selected ASME Code requirements that are impractical for the facility. These items of relief are identified and justified in Enclosure 1 to the letter transmitting the amendment to RG&E. We have concluded that the Ginna

Inservice Inspection Program (Appendix B to the Quality Assurance Manual, Rev. 2) meets the requirements set forth in 10 CFR 50.55a, Paragraph (g) and therefore constitutes an acceptable basis for satisfying the requirements of NRC General Design Criterion 32, Appendix A of 10 CFR Part 50.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: May 17, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-244

ROCHESTER GAS AND ELECTRIC CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL OPERATING LICENSE

NOTICE OF GRANTING OF RELIEF FROM ASME SECTION XI
INSERVICE INSPECTION (TESTING) REQUIREMENTS

NOTICE OF TEMORARY EXEMPTION FROM 10 CFR 50.55a(g)
TESTING REQUIREMENTS FOR PUMPS AND VALVES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 13 to Provisional Operating License No. DPR-18, issued to Rochester Gas and Electric Corporation (the licensee) for the R. E. Ginna Nuclear Power Plant (the facility) located in Wayne County, New York, which revised the Technical Specifications by replacing the existing inservice inspection requirements in Section 4.2 with requirements in conformance with 10 CFR 50, Section 50.55a(g). The amendment is effective as of the day of its issuance.

The Commission has granted relief from certain requirements of the ASME Code, Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components" to the licensee. The relief relates to the inservice inspection program for the facility. The relief consists of the substitution of visual or surface examination for certain reactor coolant system components, the allowance of repair requirements of Section III of the ASME B&PV Code, a change in the reference level used as the evaluation criterion in Section V of the ASME B&PV Code and the substitution of the Ginna Station QA Program for the Section XI administrative control requirements. The ASME Code requirements are incorporated by reference into the Commission's rules and regulations in 10 CFR Part 50.

The Commission has granted an exemption to the licensee deferring the date for commencement of the testing program conforming to the requirements of the ASME Code, Section XI for certain pumps and valves until September 1, 1977.

The application for the amendment, request for relief, and the exemption comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment and the letter granting the relief and exemption. Prior public notice of these actions was not required since the actions do not involve a singificant hazards consideration.

The Commission has determined that the issuance of the amendment granting of certain relief and temporary exemption will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with these actions.

For further details with respect to these actions, see (1) the application for amendment and relief dated February 27, 1976, as supplemented September 23, 1976 , February 8, 1977,and April 13, 1977 (2) Amendment No. 13 to License No. DPR-18, (3) the Commission's related Safety Evaluation, and (4) the Commission's letter to the licensee dated May 17, 1977. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D.C., and at the Lyons Public Library, 67 Canal Street, Lyons, New York 14489, and at the Rochester Public Library, 115 South Avenue, Rochester, New York 14627. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 17th day of May 1977.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors