October 7, 1982

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Docket No. 50-244
LS05-82-10-018

Mr. John E. Maier
Vice President
Electric & Steam Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649

Dear Mr. Maier:

SUBJECT: CONTAINMENT ISOLATION VALVE TECHNICAL SPECIFICATION CHANGE

R. E. Ginna Nuclear Power Station

The Commission has issued the enclosed Amendment No. 54 to Provisional Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment responds to your application notarized September 28, 1982.

The amendment approves provisions which update the containment isolation valve Table 3.6-1, to reflect plant system modifications.

Copies of our related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,
Original signed by
Dennis M. Crutchfield

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Enclosures:
1. Amendment No. 54 to License No. DPR-18
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

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DATE	10/5/82	10/5/82	10/6/82	10/7/82	10/7/82		

cc
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U. S. Environmental Protection Agency
Region II Office
ATTN: Regional Radiation Representative
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New York, New York 10007

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 54
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated September 28, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Provisional Operating License No. DPR-18 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendix A as revised through Amendment No. 54 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis M. Crutchfield

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 7, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 54

PROVISIONAL OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the page identified below and inserting the enclosed page. The revised page contains the captioned amendment number and a marginal line which indicates the area of change.

REMOVE	INSERT
3.6-6	3.6-6

PENT. NO.	IDENTIFICATION/DESCRIPTION	ISOLATION BOUNDARY	MAXIMUM ISOLATION TIME *(SEC)	ISOLATION BOUNDARY	MAXIMUM ISOLATION TIME *(SEC)
203	Contain. Press. transmitter PT-947 & 948	PT 947 PT 948	NA NA	MV 1819C MV 1819D	NA NA
203	Post accident air sample to "B" fan	MV 1563 MV 1566	NA NA	MV 1565 MV 1568	NA NA
204	Purge Supply Duct	AOV 5870	5	AOV 5869	5
205	Hot leg loop sample	AOV 966C	60	MV 956D(14)	NA
206	Przr. liquid space sample	AOV 966B	60	MV 956E(14)	NA
206	"A" S/G sample	AOV 5735	60	MV 5733(7)	NA
207	Przr. Steam space sample	AOV 966A	60	MV 956F	NA
207	"B" S/G sample	AOV 5736	60	MV 5734(7)	NA
209	Reactor Compart. cooling Units A & B	MV 4758(16) MV 4635(16)	NA NA	(11) (11)	NA NA
210	Oxygen makeup to A & B recombiners	MV 1080A	NA	SOV IV-2A SOV IV-2B	NA Normally Closed NA Normally Closed
300	Purge Exhaust Duct	AOV 5878	5	AOV 5879	5
301	Aux. steam supply to containment	MV 6151	NA	MV 6165(15)	NA
303	Aux. steam condensate return	MV 6175	NA	MV 6152(15)	NA
304	"A" Hydrogen recombiner (pilot and main)	MV 1084A MV 1076A	NA NA	SOV IV-5A SOV IV-3A	NA Normally Closed NA Normally Closed
305	Radiation Monitors R-11, R-12 & R-10A Auto Inlet Isol.	AOV 1597	60	MV 1596	NA
305	R-11, R-12 & R-10A Outlet	AOV 1599	60	AOV 1598	60
305	Post Accident air sample (containment)	MV 1554 MV 1557 MV 1560	NA NA NA	MV 1556 MV 1559 MV 1562	NA NA NA
307	Fire Service Water	CV 9229	NA	AOV 9227	(18)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 54 TO PROVISIONAL OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

By letter dated September 28, 1982, Rochester Gas & Electric Corporation (the licensee) requested an amendment to the Appendix A Technical Specifications appended to its License No. DPR-18, which authorizes operation of the R. E. Ginna Nuclear Power Plant. The amendment would approve a change which would revise the containment isolation valve Table 3.6-1 to reflect a check valve being replaced with an air operated diaphragm valve.

2.0 DISCUSSION

Penetration No. 305 of the Ginna containment building houses the inlet and outlet lines for radiation monitors R-11, R-12 and R-10A as well as the post accident air sample line. The 1" return line from the radiation monitors (designated "outlet" on Table 3.6-1) previously had a check valve inside containment which served as a containment isolation valve. As required by Appendix J of 10 CFR 50, periodic leak tests are required to verify the leak tight integrity of this valve.

Leak testing performed on this check valve reveals a history of poor performance. The licensee has stated that this valve has repeatedly failed leak testing criteria thus questioning the overall leak tight integrity of the entire containment structure. By letter dated September 28, 1982, the licensee has proposed replacing this check valve with an air operated diaphragm valve. The diaphragm valve would serve as a containment isolation valve and would provide a more reliable leak tight isolation boundary.

3.0 EVALUATION

The criteria for establishing containment penetration boundaries for lines that are open to the containment atmosphere are found in General Design Criteria 56 to 10 CFR 50, Appendix A. GDC 56 states that this line should be equipped with two isolation valves - one located inside containment and one located outside containment. The GDC, however, does allow for deviations from the above if the isolation provisions are found acceptable "on some other defined basis."

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The licensee has proposed placing the new air operated valve (AOV) outside containment. This configuration would place both containment isolation valves AOV 1599 and AOV 1598 outside containment. The new valve would be located between the containment and the existing outside isolation valve.

The licensee's basis for this isolation valve configuration is:

(1) As a plant design basis, the piping between the containment and the containment isolation valves is at least equal to containment design pressure (Final Safety Analysis Report [FSAR], Section 5.2.1). Isolation valves are similarly rated.

(2) Piping runs between the containment penetrations and the containment isolation valves have been kept as short as possible and are seismic Category I. The piping between the containment and the new valve is approximately three feet.

(3) All piping penetrations are solidly anchored to the containment wall. External guides, stops, increased pipe thickness, or other means are provided, where required, to limit motion and moments to prevent rupture by making the penetration the strongest part of the system. In addition, all penetrations and anchorages are designed for forces and moments that might result from postulated pipe ruptures (FSAR, Section 5.1).

(4) All piping penetrations, except the main steamlines and the feedwater lines, are located in areas that are protected from tornado missiles.

(5) The NRC PRA study reported in NUREG-0821, Ginna SEP Integrated Plant Safety Assessment, found that the risk resulting from two valves outside containment is about the same as the risk resulting from one valve inside and one valve outside containment.

The new valve to be installed will close on the same isolation signals as other valves in this radiation monitor system. Resetting of the containment isolation signal will not cause the valve to automatically reopen. The new valve will operate and reopen in conjunction with existing valve 1597, another valve in the same system and in a line within the same penetration, which allows passage of air from the containment to the radiation monitor. Electrical independence and single failure protection will be maintained.

4.0 SUMMARY

Due to chronic problems with a one-inch containment isolation check valve, the licensee has proposed replacing it with a more reliable air operated diaphragm valve. The new valve will require a modification to the containment isolation provisions and the technical specifications.

As a result of the modification, both automatic valves for isolation of this line will be outside containment. This type of arrangement has previously been evaluated to be acceptable in the Ginna SEP Integrated Plant Safety Assessment, NUREG-0821. The piping design pressure between the containment and the isolation valves is at least equal to the containment design pressure. The isolation valves are designed to function against containment pressure. The modified piping run between the containment and the new valve is as short as possible, approximately 3 feet, and is Seismic Category I. The piping supports for the new valve and piping between it and the containment wall are designed to subsection NF of the ASME Code.

Based on our review of the licensee's proposal, we conclude that an acceptable defined basis has been established to permit both the proposed plant modification and the technical specification change. Therefore, we find the proposal acceptable.

5.0 ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

6.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

7.0 ACKNOWLEDGEMENT

D. Pickett contributed to this evaluation.

Date: October 7, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-244

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

NOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 54 to Provisional Operating License No. DPR-18, to Rochester Gas and Electric Corporation (the licensee), which revised the Technical Specifications for operation of the R. E. Ginna Nuclear Power Plant (facility) located in Wayne County, New York. This amendment is effective as of its date of issuance.

The amendment approves provisions which update the containment isolation valve Table 3.6-1, to reflect the installation of a new air-operated valve.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

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For further details with respect to this action, see (1) the application for amendment notarized September 28, 1982, (2) Amendment No. 54 to License No. DPR-18, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Rochester Public Library, 115 South Avenue, Rochester, New York 14627. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C., 20555, Attentiсn: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 7th day of October, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis M. Crutchfield

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing