



Docket No. 50-244

MARCH 1 1978

Rochester Gas & Electric Corporation
ATTN: Mr. Leon D. White, Jr.
Vice President
Electric and Steam Production
89 East Avenue
Rochester, New York 14504

Gentlemen:

The Commission has issued the enclosed Amendment No. 15 to Provisional Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant in response to your application dated July 19, 1977. This amendment implements the proposed Technical Specifications for fire protection forwarded by our letter dated November 25, 1977, and incorporates comments supplied in your letter of December 13, 1977.

The amendment incorporates changes to the Technical Specifications in Appendix A to DPR-18 to provide requirements for fire protection.

These changes to the Ginna Technical Specifications are supported by the Safety Evaluation issued with our letter of November 25, 1977, except for those modifications which are proposed by your letter of December 13, 1977, and other minor modifications. These modifications are discussed herein.

- In order to achieve expeditious implementation of the fire protection Technical Specifications, Specification 6.2.2.f is being issued at this time with the minimum number of on-site fire brigade members specified as three (3) as you proposed. This number is less than the minimum number of five (5) given in the generic staff position, Minimum Fire Brigade Shift Size, which are an attachment to the Safety Evaluation Report issued with our letter to you dated November 25, 1977. However, we are presently evaluating your justification for this smaller brigade size and when our evaluation is complete the minimum number will be increased if we do not agree with your position.

OFFICE >						
SURNAME >						
DATE >						

MARCH 1 1978

- Specification 4.14.1 has been modified to define the type of acceptable test for those fire detectors located in the reactor containment building. These detectors are resistance bulb thermometers and are installed in the charcoal filter housings. The continuity and temperature readout tests that you have proposed are acceptable for these detectors.

A copy of the Notice of Issuance is also enclosed.

Sincerely,

Original Signed By

Dennis L. Ziemann, Chief
 Operating Reactors Branch #2
 Division of Operating Reactors

Enclosures:

- Amendment No. 1 to DPR-61
- Notice of Issuance

cc w/enclosures:
 See next page

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March 1, 1978

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated July 19, 1977, as supplemented December 13, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility License No. DPR-18 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 15, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 1, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 15

TO FACILITY LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A as follows:

1. Remove the following pages and insert identically numbered pages:

i
ii
1-4
6.2-1
6.2-2
6.3-1
6.5-10
6.5-11
6.8-1

2. Add the following pages:

3.14-1 through 3.14-6
4.15-1 through 4.15-3

TABLE OF CONTENTS

	<u>Page</u>
1.0 DEFINITIONS	1-1
2.0 SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS	2.1-1
2.1 Safety Limit, Reactor Core	2.1-1
2.2 Safety Limit, Reactor Coolant System Pressure	2.2-1
2.3 Limiting Safety System Settings, Protective Instrumentation	2.3-1
3.0 LIMITING CONDITIONS FOR OPERATION	3.1-1
3.1 Reactor Coolant System	3.1-1
3.1.1 Operational Components	3.1-1
3.1.2 Heatup and Cooldown	3.1-5
3.1.3 Minimum Conditions for Criticality	3.1-17
3.1.4 Maximum Coolant Activity	3.1-20
3.1.5 Leakage	3.1-25
3.1.6 Maximum Reactor Coolant Oxygen, Fluoride, and Chloride Concentration	3.1-31
3.2 Chemical and Volume Control System	3.2-1
3.3 Emergency Core Cooling System, Auxiliary Cooling Systems, Air Recirculation Fan Coolers, Containment Spray, and Charcoal Filters	3.3-1
3.4 Turbine Cycle	3.4-1
3.5 Instrumentation System	3.5-1
3.6 Containment System	3.6-1
3.7 Auxiliary Electrical Systems	3.7-1
3.8 Refueling	3.8-1
3.9 Plant Effluents	3.9-1
3.10 Control Rod and Power Distribution Limits	3.10-1
3.11 Fuel Handling in the Auxiliary Building	3.11-1
3.12 Movable In-Core Instrumentation	3.12-1
3.13 Shock Suppressors (Snubbers)	3.13-1
3.14 Fire Suppression System	3.14.1

TABLE OF CONTENTS (cont.)

	<u>Page</u>
4.0 SURVEILLANCE REQUIREMENTS	4.1-1
4.1 Operational Safety Review	4.1-1
4.2 Primary Component Tests	4.2-1
4.3 Primary System Testing Following Opening	4.3-1
4.4 Containment Tests	4.4-1
4.5 Safety Injection, Containment Spray and Iodine Removal Systems Tests	4.5-1
4.6 Emergency Power System Periodic Tests	4.6-1
4.7 Main Steam Stop Valves	4.7-1
4.8 Auxiliary Feedwater System	4.8-1
4.9 Reactivity Anomalies	4.9-1
4.10 Environmental Radiation Survey	4.10-1
4.11 Spent Fuel Pit Charcoal Adsorber Testing	4.11-1
4.12 Effluent Surveillance	4.12-1
4.13 Radioactive Material Source Leakage Test	4.13-1
4.14 Shock Suppressors (Snubbers)	4.14-1
4.15 Fire Suppression System Test	4.15-1
5.0 DESIGN FEATURES	
5.1 Site	5.1-1
5.2 Containment Design Features	5.2-1
5.3 Reactor Design Features	5.3-1
5.4 Fuel Storage	5.4-1

1.10 Hot Channel Factors

F_Q , Heat Flux Hot Channel Factor, is defined as the maximum local heat flux on the surface of a fuel rod divided by the average fuel rod heat flux allowing for manufacturing tolerances on fuel pellets and rods.

F_Q^N , Nuclear Heat Flux Hot Channel Factor, is defined as the maximum local fuel rod linear power density divided by the average fuel rod linear power density assuming nominal fuel pellet and rod dimensions.

F_Q^E , Engineering Heat Flux Hot Channel factor, is defined as the ratio between F_Q and F_Q^N and is the allowance on heat flux required for manufacturing tolerances.

$F_{\Delta H}^N$, Nuclear Enthalphy Rise Hot Channel Factor, is defined as the ratio of the integral of linear power along the rod on which minimum DNBR occurs to the average rod power.

1.11 Fire Suppression Water System

The fire suppression water system consists of Lake Ontario water supply, pumps and distribution piping with associated sectionalizing control or isolation valves. Valves include valves between the fire pumps and the first valve ahead of the water flow alarm device on each sprinkler, or spray system riser.

3.14

Fire Suppression System

Applicability

Applies to the operating status of the Fire Suppression System.

Objective

To define those conditions of the Fire Suppression System to provide adequate fire protection.

Specification

- 3.14.1 The fire detection instruments for each fire detection zone shown in Table 3.14-1 shall be operable.
- 3.14.1.1 With the number of operable instruments less than that required by Specification 3.14.1,
- a. Within an hour, establish a fire watch patrol to inspect the zone with the inoperable instrument(s) at least once per hour; except in containment where it will be once every 8 hours and
 - b. Comply with the requirements of Specification 3.14.1 within 14 days, or
 - c. Prepare and submit a Thirty Day Written Report in accordance with Specification 6.9.2 outlining the cause of the inoperability and the plans for restoring the instrument(s) to operable status.
- 3.14.2 The fire suppression water system shall be operable with:
- a. Two fire pumps each with a capacity of 2000 gpm with their discharge aligned to the fire suppression header.
 - b. Automatic initiation logic for each fire pump.
- 3.14.2.1 With an inoperable redundant component,
- a. In the operating mode, restore the component to operable status within 7 days. If the component is not restored to operable status within 7 days, run the remaining pump continuously and prepare and submit a Thirty Day Written Report in accordance with Specification 6.9.2 outlining the cause of the inoperability and plans for restoring the component to operable status.
 - b. In cold or refueling shutdown, restore the component to operable status within 7 days or prepare and submit a Thirty Day Written Report in accordance with

Specification 6.9.2 outlining the cause of inoperability and the plans for restoring the component to operable status.

- 3.14.2.2 With the fire suppression water system inoperable,
- a. Establish within 24 hours a backup fire suppression water system and
 - b. Provide Prompt Notification With Written Followup in accordance with Specification 6.9.2 outlining the actions taken, the cause of the inoperability, and the plans for restoring the components to operable status.
 - c. If a. and b. above cannot be fulfilled, place the reactor in Hot Shutdown within the next six (6) hours and in Cold Shutdown within the following thirty (30) hours.

3.14.3 The spray systems located in the following areas shall be operable when equipment in the area is required to be operable:

- a. "A" Diesel Generator Room
- b. "B" Diesel Generator Room
- c. Turbine Driven Auxiliary Feedwater Pump Oil Reservoir
- d. Cable Tunnel

3.14.3.1 With a spray system inoperable, except for testing, within an hour, establish a continuous fire watch with backup fire suppression equipment in the unprotected area(s) when equipment in the area is required to be operable, and

- a. Restore the system to operable status within 14 days or prepare and submit a Thirty Day Written Report in accordance with Specification 6.9.2 outlining the cause of the inoperability and the plans for restoring the system to operable status.

3.14.4 The Halon systems located in the following areas shall be operable when equipment in the area is required to be operable and the storage tanks shall have at least 95% of the full charge weight and 90% of full charge pressure at 70°F:

- a. Computer Room
 - b. Relay Room
- 3.14.4.1 With a Halon system inoperable, within one hour, establish a continuous fire watch with portable equipment in the unprotected area(s) when equipment in the area is required to be operable, and
- a. Restore the system to operable status within 14 days or prepare and submit a Thirty Day Written Report in accordance with Specification 6.9.2 outlining the cause of inoperability and the plans for restoring the system to operable status.
- 3.14.5 The fire hose stations in Table 3.14-2 shall be operable.
- 3.14.5.1 With a hose station listed in Table 3.14-2 inoperable, route a hose to the unprotected area from an operable hose station within an hour.
- 3.14.6 All fire barrier penetration fire seals protecting safety related areas shall be intact.
- 3.14.6.1 With a fire barrier penetration fire seal which protects a safety related area not intact, a continuous fire watch shall be established on one side of the penetration within one hour.

Basis:

The fire protection system has the capability to extinguish any probable fire which might occur at the station. The system is designed in accordance with the standards of the National Fire Protection Association (1965) and is based generally on the recommendations of the Nuclear Energy Liability Property Insurance Association.

Procedures have been developed for fighting fires in all the plant areas and are contained in the plant's emergency procedures. Fire prevention is controlled by administrative methods to prevent accumulations of combustible materials and to practice good safety methods. Periodic practice exercises will be employed to insure plant personnel are familiar with the proper corrective procedures.

Normal fire protection is provided by a fixed fire-fog system, fixed Halon 1301 system, sprinklers, hose lines, and portable and wheeled extinguishers suitably located in the required areas.

Readily accessible 1-1/2 inch rubber covered hose lines and continuous flow type hose reels are distributed throughout the station so that all areas in the station are within 20 feet of a fog nozzle when attached to not more than 100 foot lengths of hose. All nozzles are 1-1/2 inch variable fog-off nozzles.

Water to the fire system is supplied via the header by two vertical, centrifugal fire pumps of 2000 gpm capacity each. One of these pumps is driven by an electric motor and the other by a combustion engine. Both are automatic starting through fire pump controllers with indication, alarm and manual starting from the central control room fire panel. The combustion engine local fuel supply capacity is designed for 8 hours of operation.

A fire header is installed of sufficient size to deliver an adequate quantity of water throughout the plant at a pressure of no less than 75 psi at the highest nozzle.

The header system is normally pressurized through the use of a hydro-pneumatic tank using house service air and having an active water capacity of 10,000 gallons. Loss of header pressure and/or opening of any deluge system activates the fire pumps and the alarm system.

A backup fire suppression water system would be used to provide protection in the event the fire suppression water system were inoperable. A backup system could, for example, be comprised of a backup pump, the yard hydrant system supplying water to wall hydrants, or other equipment or measures.

Table 3.14-1

FIRE DETECTION INSTRUMENTS

<u>INSTRUMENT LOCATION</u>	<u>MINIMUM INSTRUMENTS OPERABLE</u>	
	<u>HEAT</u>	<u>SMOKE</u>
1. Containment		
*"A" Post-Accident Charcoal Bank	3	N/A
*"B" Post-Accident Charcoal Bank	3	N/A
*"A" Aux. Filter Charcoal Bank	1	N/A
*"B" Aux. Filter Charcoal Bank	1	N/A
2. "A" Diesel Generator	2	N/A
3. "B" Diesel Generator	2	N/A
4. Auxiliary FW Pump Oil Reservoir	1	N/A
5. Cable Tunnel	8	6
6. Relay Room	3	6
7. Computer Room	1	2
8. Battery Rooms		
"A" Battery Room	N/A	1
"B" Battery Room	N/A	1
9. Control Room	N/A	1
10. Aux. Bldg. General	N/A	8

* Resistance Temperature Detectors only.

TABLE 3.14-2
FIRE SERVICE WATER HOSE REEL LOCATION

<u>BUILDING</u>	<u>FLOOR</u>	<u>LOCATION</u>
Turbine	Basement	Battery Room
Turbine	Intermediate	4160 Bus
Intermediate	Level Four	West
Intermediate	Level Four	East
Intermediate	Level Three	East
Intermediate	Level Three	West
Intermediate	Level Two	West
Intermediate	Level Two	East
Intermediate	Level One	East
Intermediate	Level One	West
Intermediate	Level One	South
Intermediate	Level Two	Nuclear Sample Room
Auxiliary	Operating	West
Auxiliary	Operating	Center
Auxiliary	Operating	East
Auxiliary	Intermediate	East
Auxiliary	Intermediate	Center
Auxiliary	Intermediate	West
Auxiliary	Basement	West
Auxiliary	Basement	Center
Auxiliary	Basement	East
Screen House	Main	Fire Pumps

3.14-6

4.15 Fire Suppression System Test

Applicability:

Applies to periodic testing and surveillance requirements of the Fire Suppression System.

Objective:

To verify that the Fire Suppression System will respond properly, if required.

Specification:

- 4.15.1 The fire detection instruments listed in Table 3.14-1 shall be demonstrated operable by performance of the manufacturer's recommended tests at least once every six months. The functional test for detectors inside containment will be performed by verifying detector circuit continuity and detector temperature indication in the control room.
- 4.15.1.1 The circuitry associated with the detector alarms in Table 3.14-1 shall be demonstrated operable at least once every six months.
- 4.15.2 The fire suppression water system shall be demonstrated operable:
- a. At least once per 31 days by starting each pump and operating it for at least 15 minutes on recirculation flow.
 - b. At least once per 31 days by verifying that each valve (manual, power operated, or automatic) in the flow paths is in its correct position.
 - c. At least once per 12 months by cycling each motor operated valve through one complete cycle.
 - d. At least once per 18 months by verifying that each pump will develop a flow of at least 2000 gpm at a system head of 210 feet.
 - e. At least once every 3 years a flow test of the fire suppression water system shall be performed. With one fire pump running the static pressure will be recorded at the test connection for the fire suppression water system. The four exterior wall hydrants will be flowed individually with the residual pressure at the test connection and the flow from each hydrant recorded.
- 4.15.3 The spray systems shall be demonstrated to be operable:
- a. At least once per 12 months by verifying the loss of locking pressure manual operation.
 - b. At least once per 18 months:
 - (i) By performing a system functional test which includes simulating actuation of the system and verifying that the valves in the flow path are capable of going to their correct positions.

- (ii) By visual external inspection of spray headers to verify their integrity,
 - (iii) By visual external inspection of each nozzle to verify no blockage.
- c. At least once per 3 years by performing an air flow test through each spray header and verifying each spray nozzle is unobstructed.

4.15.4

The Halon System shall be demonstrated to be operable:

- a. At least once per 6 months by verifying each Halon storage tank pressure.
- b. At least once per 6 months by verifying each Halon storage tank weight.
- c. At least once per 18 months by verifying the system including associated ventilation dampers actuate in response to a simulated actuation signal. An air flow test through headers and nozzles shall be performed to assure no blockage. The operability of the manual initiating system will also be verified.

4.15.5

Each fire hose station listed in Table 3.14-2 shall be verified to be operable:

- a. At least once per month by visual inspection of the station to assure all equipment is available and the fire water header system pressure is recorded.
- b. At least once per 18 months by unrolling the hose for inspection and re-racking and replacing gaskets in the couplings, as required.
- c. At least once per 18 months, partially open hose station valves to verify valve operability and no blockage.
- d. At least every 3 years by pressure testing each hose to 50 psi greater than the Maximum Working Pressure.

4.15.6

Penetration seals in fire barriers shall be verified to be intact by visual inspection:

- a. At least once per 18 months, and
- b. Prior to declaring a penetration seal in a fire barrier intact following repairs or maintenance.

Basis:

Sufficient tests will be made to be certain that fire detection instruments and associated circuitry are operable such that fires in areas which would jeopardize the safe shutdown of the plant are detected.

The fire suppression water system testing will assure the capability of the system to meet its requirements.

The Halon System is used to protect those areas that would be damaged by the use of water. The 90% of full charge pressure is based on a temperature of 70°F. Pressures at temperatures other than 70°F will be corrected by Chart ULE 2671 March 1, 1973 ANSUL 1301 Clean Agent Free Control System Manual P/N 17210-02.

Visual inspections of fire barrier penetration seals will be made to insure the containment of any fire that may start until it can be extinguished either automatically or manually. There are no fire barriers that perform a pressure sealing function.

- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips caused by transients or emergencies.
- d. All core alterations shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- e. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- f. A Fire Brigade of 3 members shall be maintained on site at all times. This excludes the two members of the minimum shift crew necessary for safe shutdown.

6.3 STATION STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI Standard N18.1-1971, "Selection and Training of Nuclear Power Plant Personnel" for comparable positions.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Training Coordinator and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix A of 10 CFR Part 55.

6.4.2 The training program shall meet or exceed NFPA No. 27, 1975 Section 40, except that (1) training for salvage operations need not be provided and (2) the Fire Brigade training sessions shall be held at least quarterly. Drills are considered to be training sessions.

AUDITS (Continued)

- g. The Facility Fire Protection Program and implementing procedures at least once per two years.
- h. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- i. An inspection and audit of the fire protection and loss prevention program shall be performed by non-licensee personnel at least once per 36 months. The personnel may be representatives of NELPIA, an insurance brokerage firm, or other qualified individuals.
- j. Any other area of facility operation considered appropriate by the NSARB or the Vice President, Electric and Steam Production.

AUTHORITY

6.5.2.9

- a. The chairman of the Nuclear Safety Audit and Review Board is responsible to the Corporate Chairman of the Board on all activities for which the review board is responsible.
- b. The NSARB shall report to and advise the Vice President, Electric and Steam Production, on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of NSARB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes shall be recorded of all meetings of this Board. Copies of the minutes shall be forwarded within 14 days following each meeting to the Corporate Chairman of the Board, Vice President, Electric and Steam Production and such others as the Chairman of the NSARB may designate.
- b. Reports of reviews encompassed by Section 6.5.2.7 e, f, g and h above, shall be prepared, approved and forwarded to the Vice President-Electric and Steam Production within 14 days following completion of the review.

PROCEDURES

6.5.2.11 Written administrative procedures for committee operation shall be prepared and maintained describing the method of submission and the content of presentations to the committee, provisions for use of subcommittees, review and approval by members of written committee evaluations and recommendations, distribution of minutes, and such other matters as may be appropriate.

6.5.3 QUALITY ASSURANCE GROUP

6.5.3.1 The organization, qualifications, responsibilities and training of quality assurance personnel responsible for audits of safety related activities are described in the Quality Assurance Program.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November 1972.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the PORC and approved by the Station Superintendent prior to implementation and reviewed periodically as set forth in the applicable procedures.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom is the Shift Foreman who holds a Senior Reactor Operator's License.
- c. The change is documented, reviewed by the PORC, and approved by the Station Superintendent within 10 days of implementation.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-244

ROCHESTER GAS AND ELECTRIC CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 15 to Provisional Operating License No. DPR-18 issued to Rochester Gas and Electric Corporation which revise Technical Specifications for operation of the R. E. Ginna Plant located in Wayne County, New York. The amendment is effective as of the date of issuance.

The amendment incorporates fire protection Technical Specifications on the existing fire protection equipment and adds administrative controls related to fire protection at the facility. This action is being taken pending completion of the Commission's overall fire protection review of the facility.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated July 19, 1977, as supplemented December 13, 1977, (2) Amendment No. 15 to License No. DPR- , and (3) the Commission's related Safety Evaluation dated November 25, 1977. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW, Washington, DC and at the Rochester Public Library, 115 South Avenue, Rochester, New York 14627. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 1st day of March 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors