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Docket No. 50-244

Mr. Leon D. White, Jr.  
 Vice President  
 Electric and Steam Production  
 89 East Avenue  
 Rochester, New York 14649

JUL 26 1979

Dear Mr. White:

The Commission has issued the enclosed Amendment No. 28 to Provisional Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment consists of changes to the Technical Specifications in response to your application dated July 17, 1979 (transmitted by letter dated July 19, 1979).

The amendment allows a change in the overpressure protection system requirements.

Copies of our related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by  
 Dennis L. Ziemann

Dennis L. Ziemann, Chief  
 Operating Reactors Branch #2  
 Division of Operating Reactors

*Crutchfield*

Enclosures:

1. Amendment No. 28 to License No. DPR-18
2. Safety Evaluation
3. Notice

cc w/enclosures:  
 See next page

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*CJB*  
 DOR:RSB  
 C Beringer  
 7/23/79

*Phone copy*  
 ASHB  
 EK  
 7/24/79  
 DOR:AD/SEP  
 RHVolmer  
 7/24/79

OFFICE	DOR:ORB #2	DOR:ORB #2	DOR:RSB	OELD	OELD	DOR:ORB #2
SURNAME	JJShea:ah	HSmith	TMarsh	PCheck	Ketchen	DLZiemann
DATE	7/20/79	7/25/79	7/23/79	7/23/79	7/24/79	7/24/79

Mr. Leon D. White, Jr.

- 2 -

JUL 26 1979

cc w/enclosures:

Lex K. Larson, Esquire  
LeBoeuf, Lamb, Leiby & MacRae  
1757 N Street, N. W.  
Washington, D. C. 20036

Mr. Michael Slade  
1250 Crown Point Drive  
Webster, New York 14580

Rochester Committee for  
Scientific Information  
Robert E. Lee, Ph.D.  
P. O. Box 5236 River Campus  
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Jeffrey Cohen  
New York State Energy Office  
Swan Street Building  
Core 1, Second Floor  
Empire State Plaza  
Albany, New York 12223

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State of New York Energy Office  
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Albany, New York 12223

Rochester Public Library  
115 South Avenue  
Rochester, New York 14604

Supervisor of the Town  
of Ontario  
107 Ridge Road West  
Ontario, New York 14519

\*(w/cy of incoming dtd. 7/19/79 w/application  
dtd. 7/17/79)

Director, Technical Assessment  
Division  
Office of Radiation Programs  
(AW-459)  
U. S. Environmental Protection  
Agency  
Crystal Mall #2  
Arlington, Virginia 20460

U. S. Environmental Protection  
Agency  
Region II Office  
ATTN: EIS COORDINATOR  
26 Federal Plaza  
New York, New York 10007



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

R. E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 28  
License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Rochester Gas and Electric Corporation (the licensee) dated July 17, 1979 (transmitted by letter dated July 19, 1979), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Provisional Operating License No. DPR-18 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 28 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by  
Dennis L. Ziemann

Dennis L. Ziemann, Chief  
Operating Reactors Branch #2  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: JUL 26 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 28

PROVISIONAL OPERATING LICENSE NO. DPR-18

DOCKET NO. 50-244

Revise Appendix A Technical Specifications by removing the page identified below and inserting the enclosed page. The revised page contains the captioned amendment number and a vertical line indicating the area of change.

Page

3.15-1

### 3.15 Overpressure Protection System

#### Applicability

Applies whenever the temperature of one or more of the RCS cold legs is  $\leq 330^{\circ}\text{F}$ .

#### Objective

To prevent overpressurization of the reactor coolant system.

#### Specification

- 3.15.1 Except during secondary side hydrostatic tests in which RCS pressure is to be raised above the PORV setpoint, at least one of the following overpressure protection systems shall be operable:
- a. Two pressurizer power operated relief valves (PORVs) with a lift setting of  $\leq 435$  psig, or
  - b. A reactor coolant system vent of  $\geq 1.1$  square inches.
- 3.15.1.1 With one PORV inoperable, either restore the inoperable PORV to operable status within 7 days or depressurize and vent the RCS through a 1.1 square inch vent(s) within the next 8 hours; maintain the RCS in a vented condition until both PORVs have been restored to operable status.
- 3.15.1.2 With both PORVs inoperable, depressurize and vent the RCS through a 1.1 square inch vent(s) within 8 hours; maintain the RCS in a vented condition until both PORVs have been restored to operable status.
- 3.15.1.3 Use of the overpressure protection system to mitigate a RCS pressure transient shall be reported in accordance with 6.9.3.

#### Basis

The operability of two pressurizer PORVs or an RCS vent opening of greater than 1.1 square inches ensures that the RCS will be protected from pressure transients which could exceed the limits of Appendix G to 10 CFR Part 50 when one or more of the RCS cold legs are  $\leq 330^{\circ}\text{F}$ . Either PORV has adequate relieving capability to protect the RCS from overpressurization when the transient is limited to either (1) the start of an idle RCP with the secondary water temperature of the steam generator  $\leq 50^{\circ}\text{F}$  above the RCS cold leg temperature or (2) the start of a safety injection pump and its injection into a water solid RCS. (1)

#### References:

- (1) L. D. White, Jr. letter to A. Schwencer, NRC, dated July 29, 1977



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 28 TO PROVISIONAL OPERATING LICENSE NO. DPR-18

ROCHESTER GAS AND ELECTRIC CORPORATION

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

1.0 INTRODUCTION

By application dated July 17, 1979 (transmitted by letter dated July 19, 1979), Rochester Gas and Electric Corporation (RG&E) requested an amendment to License No. DPR-18 for the R. E. Ginna Nuclear Power Plant to change reactor overpressure protection system requirements.

2.0 DISCUSSION

Technical Specification Section 3.15.1 requires that at least one overpressure protection system be operable whenever the temperature of one or more of the reactor coolant system cold legs is less than or equal to 330°F. The purpose of this requirement is to protect the reactor coolant system from pressure transients which could exceed the limits of Appendix G to 10 CFR Part 50. RG&E has requested relief from this requirement during secondary system hydrostatic tests. The purpose of the request is to allow performance of required hydrostatic tests at a pressure of at least 1356 psi (125% of design pressure) while maintaining the secondary to primary side pressure differential to 800 psi. RG&E administratively limits the pressure differential between the secondary to primary sides during the tests to prevent separation of cladding from the base metal in the channel head where high radiation levels make repairs difficult:

The pressurizer power operated relief valves used for overpressure protection are required by technical specifications to have a lift setting of less than or equal to 435 psig when one or more of the reactor coolant system cold legs is less than 330°F. This requirement limits the pressure that can be achieved in the secondary steam system to 1235 psig (435 psig + 800 psig) and prevents the conduct of the required hydrostatic test. The hydrostatic test cannot be conducted with the reactor coolant system cold legs at a temperature above 330°F because the steam line spring hangers are pinned during the test while the lines are filled with water and may not be able to accommodate thermal expansion of the steam lines if they are heated.

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During operations at reactor coolant temperatures below 330°F, transients which could cause pressurization are (1) charging pump operation, (2) the start of an idle reactor coolant pump with the secondary water temperature of the steam generators at least 50°F warmer than the reactor coolant system, or (3) start of a safety injection pump. Adherence to technical specification limits prevents charging pump operation in a manner to cause overpressurization. During the hydrostatic tests, the temperature differential between the primary and secondary side is minimized and is maintained below 50°F. Also, during the test no safety injection pumps are allowed to operate. RG&E has proposed to strengthen these administrative controls by supplementing the normal operations shift during the test with an additional licensed operator who will be responsible for assuring that all necessary administrative controls will be observed.

### 3.0 EVALUATION

Our review indicates that the change requested by RG&E is based on prudent plant operating practices. Secondary side hydrostatic tests which require raising RCS pressure to above the PORV setpoint are very infrequent. (RG&E informed us that the test at 125% of design pressure which would be performed upon issuance of the requested amendment would be the first one performed since the plant went into power operation). We also note that the maximum RCS pressure during this test, while above the PORV setpoint, will be well below the heatup and cooldown limit curves of the Technical Specifications and in conformance with the requirements of Appendix G to 10 CFR Part 50. Based on the very low frequency of the 125% secondary side hydrostatic test, the added administrative controls placed in effect during the conduct of the test, and the fact that the maximum RCS pressure is still below the most conservative allowable pressure limit, the proposed technical specification change does not significantly reduce the protection against pressure transients which would exceed the limits of Appendix G to 10 CFR Part 50 and is acceptable. We have revised the proposed change to clarify the intent of the specification. RG&E representatives have agreed to the revision.

### 4.0 ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.



5.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date:

JUL 26 1979

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-244ROCHESTER GAS AND ELECTRIC CORPORATIONNOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 28 to Provisional Operating License No. DPR-18, issued to Rochester Gas and Electric Corporation (the licensee), which revised the Technical Specifications for operation of the R. E. Ginna Nuclear Power Plant (the facility) located in Wayne County, New York. The amendment is effective as of its date of issuance.

The amendment allows a change in the overpressure protection system requirements.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

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- 2 -

For further details with respect to this action, see (1) the application for amendment dated July 17, 1979 (transmitted by letter dated July 19, 1979), (2) Amendment No. 28 to license No. DPR-18, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Rochester Public Library, 115 South Avenue, Rochester, New York 14627. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this JUL 26 1979

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by  
~~Dennis L. Ziemann~~

Dennis L. Ziemann, Chief  
Operating Reactors Branch #2  
Division of Operating Reactors

*Preliminary and final*

William O. Miller, Chief  
License Fee Management Branch, ADM

Date: 7/26/79  
Amended Form Date: \_\_\_\_\_

FACILITY AMENDMENT CLASSIFICATION - DOCKET NO(S). 50-244

Licensee: Rochester Gas

Plant Name and Unit(s): Minna

License No(s): DPR-18 Mail Control No: \_\_\_\_\_

Request Dated: 7/17/79 (and); 7/19/79 (Hr.) Fee Remitted: Yes \_\_\_\_\_ No \_\_\_\_\_

Assigned TAC No: 11932

Licensee's Fee Classification: Class I \_\_\_\_\_, II \_\_\_\_\_, III , IV \_\_\_\_\_, V \_\_\_\_\_, VI \_\_\_\_\_, None

Subject: overpressure protection

Amendment No. 28 Date of Issuance 7/26/79

1. This request has been reviewed by DOR/DPM in accordance with Section 170.22 of Part 170 and is properly categorized.

2. This request is incorrectly classified and should be properly categorized as Class \_\_\_\_\_. Justification for classification or reclassification: \_\_\_\_\_

3. Additional information is required to properly categorize the request: \_\_\_\_\_

4. This request is a Class \_\_\_\_\_ type of action and is exempt from fees because it:

- (a) \_\_\_\_\_ was filed by a nonprofit educational institution,
- (b) \_\_\_\_\_ was filed by a Government agency and is not for a power reactor,
- (c) \_\_\_\_\_ is for a Class \_\_\_\_\_ (can only be a I, II, or III) amendment which results from a written Commission request dated \_\_\_\_\_ for the application and the amendment is to simplify or clarify license or technical specifications, has only minor safety significance, and is being issued for the convenience of the Commission, or
- (d) \_\_\_\_\_ other (state reason therefor): \_\_\_\_\_

*HS 7/20/79*  
*RS*  
*R Silver*

*Dennis R Zemann*

Division of Operating Reactors/Project Management

THE INITIAL FEE DETERMINATION HAS BEEN REASSESSED AND IS HEREBY AFFIRMED \_\_\_\_\_

The above request has been reviewed and is exempt from fees.

*Attached: incoming*  
*LA*  
LFMB 6/78

William O. Miller, Chief  
License Fee Management Branch

\_\_\_\_\_ Date