

SEP 25 1969

Docket No. 50-244

Rochester Gas and Electric Corporation  
89 East Avenue  
Rochester, New York 14604

Attention: Francis E. Drake, Jr.  
Chairman of the Board

Gentlemen:

The Commission has issued, effective September 19, 1969, Provisional Operating License No. DPR-18, to Rochester Gas and Electric Corporation authorizing the Company to operate the Robert Emmett Ginna Nuclear Power Plant Unit No. 1, on the Company's site in Wayne County, New York. Some additional copies of the license and the Technical Specifications and a copy of a related notice which has been forwarded to the Office of the Federal Register for filing and publication are enclosed for your use.

Since publication of the Notice of Proposed Issuance in the Federal Register on June 17, 1969, we have reviewed the Class I piping and the outstanding research and development programs. Therefore, the license authorizes Rochester Gas and Electric to operate the facility at power levels not to exceed 1300 megawatts thermal. Five copies of an Addendum to the Safety Evaluation which discusses our evaluation of the Class I piping and the research and development programs are also enclosed.

Two signed copies of Amendment No. 1 to Indemnity Agreement No. B-38, which covers the activities authorized under this license, were transmitted to your attorney on September 19, 1969, for your review and acceptance. Please sign and return one copy of the Amendment to this office.

Sincerely,

Original Signed by  
Peter A. Morris

Peter A. Morris, Director  
Division of Reactor Licensing

Enclosures:

- 1. Lic. No. DPR-18, w/Tech Specs and Attachment A - SEE Rpts Section For Tech. Specs. + Attach. X
- 2. Fed. Reg. Notice

3. Amendment No. 1 to Indemnity Agreement No. B-38				
4. Addendum to Safety Evaluation - SEE Rpts. Section				

cc: DATE Mrvin E. Upton, Esq.

SEP 25 1969

bcc w/TS: G. Hadlock, OGC  
 L. Kornblith, CO (2)  
 N. Dube (3)  
 J. Saltzman, SLR  
 D. Skovholt  
 P. Collins  
 H. J. MacAlduff, ORO  
 S. Robinson, Secy

bcc wo/TS: A. A. Wells, AS&LB  
 E. E. Hall, GMR/H  
 D. Nussbaumer, DML  
 J. A. Harris, PI  
 E. Tremmel, IP  
 R. Leith, OC  
 J. R. Buchanan, ORNL  
 J. Verme, SNM

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- P. A. Morris
- R. S. Boyd
- V. Benaroya
- S. M. Kari

OFFICE ▶	DRL/PPB-5	DRL/PPB-5	OGC	DRL/AD:RP	DRL/DR
SURNAME ▶	SMKari:klis	DFR:klis	WAT	RSBoyd	PAM:klis
DATE ▶	9/ / 1969	9/25/69	9/25/69	9/25/69	9/25/69

UNITED STATES ATOMIC ENERGY COMMISSION

DOCKET NO. 50-244

ROCHESTER GAS AND ELECTRIC CORPORATION

NOTICE OF ISSUANCE OF PROVISIONAL OPERATING LICENSE

No request for a hearing by the applicant or petition to intervene by any interested person having been filed following publication of the notice of proposed action in the Federal Register, on June 17, 1969, (34 F.R. 9466) the Atomic Energy Commission has issued Provisional Operating License No. DPR-18 to Rochester Gas and Electric Corporation (RG&E) authorizing the licensee to possess, use, and operate the Robert Emmett Ginna Nuclear Power Plant Unit No. 1, a pressurized, light water moderated, nuclear reactor, located on RG&E's Brookwood site in Wayne County, New York. The license authorizes RG&E to operate the reactor at power levels not to exceed 1300 megawatts thermal in accordance with the provisions of the license and the Technical Specifications (Appendix A) appended thereto.

The Commission has found that the application for provisional operating license complies with the requirements of the Atomic Energy Act of 1954, as amended, and the Commission's regulations published in 10 CFR, Chapter 1. The Commission has made the findings which were set forth in the proposed license and has concluded that the issuance of the provisional operating license will not be inimical to the common defense and security or to the health and safety of the public. The Commission has inspected the facility and determined that it has been constructed in accordance with the application, as amended, and the provisions of Construction Permit No. CPPR-19, as amended.

The notice of the proposed action as published in the Federal Register provided that the power level would be limited to five megawatts thermal until the Commission reviewed (1) the Class I piping analysis and (2) the results of the research and development programs on heat transfer tests of the cooler tubes, fan motor tests and process instrument transmitters. Since publication of the notice of proposed action, the Commission has reviewed the information relating to these items submitted by RG&E in letters dated June 11, 1969, August 6, 1969, and September 8, 1969. The Commission has determined that the items in question have been satisfactorily resolved and that the restriction in power level is therefore unnecessary. On this basis the Commission has determined that the conclusions previously reached as to the safety of the facility are unchanged.

The license was issued as set forth in the Notice of Proposed Issuance of Provisional Operating License published in the Federal Register on June 17, 1969, 34 F.R. 9466, except for (1) the revision of Paragraph d, of the findings to reflect the satisfactory completion of the Class I piping analysis and the research and development programs, (2) the inclusion in Paragraph 2,B of the authority to possess and use 80 grams of plutonium previously licensed under 10 CFR Part 70, "Special Nuclear Material," (3) the revision of Paragraph 3,A to eliminate the restriction of the power level and to allow operation at 1300 megawatts thermal in accordance with the provisions of the Technical Specifications, and (4) the revision of Paragraph 3,C to include all reporting requirements in the Technical Specifications.

The Technical Specifications have been modified as set forth in Attachment A thereto by (1) revising the definition of an abnormal occurrence to

clarify its usage in subsequent sections under reporting requirements, (2) outlining procedures to be taken in the event of an abnormal occurrence in plant operation, or action necessary if a safety limit is exceeded, (3) consolidating the events requiring reports, written or otherwise, at specified time periods and (4) the inclusion of the items to be covered in the descriptive material covering offsite environmental surveys.

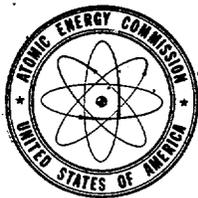
Copies of (1) the letters dated June 11, 1969, August 6, 1969, and September 8, 1969, (2) License No. DPR-18 and Technical Specifications thereto, and (3) the Addendum to the Safety Evaluation are available for public inspection in the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. Copies of items (2) and (3) may be obtained at the Commission's Public Document Room, or upon request addressed to the Atomic Energy Commission, Washington, D. C. 20545, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland this *25<sup>th</sup>* day of September, 1969.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed by  
Peter A. Morris

Peter A. Morris, Director  
Division of Reactor Licensing



UNITED STATES  
ATOMIC ENERGY COMMISSION  
WASHINGTON, D.C. 20545

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

PROVISIONAL OPERATING LICENSE

License No. DPR-18

The Atomic Energy Commission (the Commission) having found that:

- a. The application for provisional operating license (Amendments No. 6 through 19, dated January 18, 1968, April 9, 1968, September 23, 1968, September 30, 1968, October 10, 1968, October 16, 1968, December 2, 1968, December 6, 1968, January 31, 1969, February 3, 1969, February 12, 1969, March 14, 1969, March 28, 1969, and April 16, 1969, respectively) complies with the requirements of the Atomic Energy Act of 1954, as amended, and with the Commission's regulations set forth in Title 10, Chapter 1, CFR;
- b. The facility has been constructed in accordance with the application, as amended, and the provisions of Provisional Construction Permit No. CPPR-19, as amended;
- c. There are involved features, characteristics and components as to which it is desirable to obtain actual operating experience before the issuance of an operating license for the full term requested in the application;
- d. There is reasonable assurance (i) that the facility can be operated at power levels not in excess of 1300 megawatts thermal in accordance with this license without endangering the health and safety of the public and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;
- e. The applicant is technically and financially qualified to engage in the activities authorized by this license, in accordance with the rules and regulations of the Commission;
- f. The applicant has furnished proof of financial protection to satisfy the requirements of 10 CFR Part 140;
- g. The issuance of this license will not be inimical to the common defense and security or to the health and safety of the public;

Provisional Operating License No. DPR-18 is hereby issued to Rochester Gas and Electric Corporation (RG&E), as follows:

1. This license applies to the Robert Emmett Ginna Nuclear Power Plant Unit No. 1, a closed cycle, pressurized, light water moderated and cooled reactor, and electric generating equipment (the facility). The facility is located on the applicant's site on the south shore of Lake Ontario, Wayne County, New York, about 16 miles east of the City of Rochester, and is described in license application Amendment No. 6, "Final Facility Description and Safety Analysis Report," as supplemented and amended (Amendments No. 7 through 19).
2. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses RG&E:
  - A. Pursuant to Section 104b of the Atomic Energy Act of 1954, as amended, (the Act) and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess, use, and operate the facility as a utilization facility at the designated location on RG&E's Brookwood Site;
  - B. Pursuant to the Act and 10 CFR Part 70, "Special Nuclear Material," to receive, possess and use at any one time up to 2300 kilograms of contained uranium-235 in connection with operation of the facility; and to receive, possess and use 80 grams of plutonium contained in encapsulated form as a Pu-Be neutron source;
  - C. Pursuant to the Act and 10 CFR Part 30, "Rules and General Applicability to Licensing of Byproduct Material," to receive, possess and use 72.6 microcuries of neptunium-237 composed of six sealed sources contained in irradiation surveillance capsules; to receive, possess and use 600 curies of polonium-beryllium contained in encapsulated form as primary source rods in neutron source assemblies; and to possess and use 65,000 curies of antimony-beryllium contained in encapsulated form as a secondary source; and
  - D. Pursuant to the Act, and Parts 30 and 70, to possess, but not to separate, such byproduct and special nuclear material as may be produced by operation of the facility.
3. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Part 20, § 30.34 of Part 30, § 40.41 of Part 40, § § 50.54 and 50.59 of Part 50, and § 70.32 of Part 70, and is subject to all applicable

provisions of the Act and rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

A. Maximum Power Level

RG&E is authorized to operate the facility at steady state power levels up to a maximum of 1300 megawatts thermal.

B. Technical Specifications

The Technical Specifications contained in Appendix A attached hereto, and as modified in Attachment A, are hereby incorporated in this license. RG&E shall operate the facility at power levels not in excess of 1300 megawatts thermal in accordance with the Technical Specifications, and may make changes therein only when authorized by the Commission in accordance with the provisions of § 50.59 of 10 CFR Part 50.

C. Reports

In addition to the reports otherwise required under this license and applicable regulations, RG&E shall make certain reports as required in accordance with the provisions of the Technical Specifications, as modified in Attachment A appended thereto.

D. Records

RG&E shall keep facility operating records in accordance with the requirements of the Technical Specifications.

4. Pursuant to § 50.60 of 10 CFR Part 50, the Commission has allocated to RG&E for use in the operation of the facility 14,567 kilograms of uranium-235 contained in uranium in the isotopic ratios specified in the application. Estimated schedules of special nuclear material transfers to RG&E and returns to the Commission are contained in Appendix B which is attached hereto. Transfers by the Commission to RG&E in accordance with Column 2 in Appendix B will be conditioned upon RG&E's return to the Commission of material substantially in accordance with Column 3 (including the subcolumns headed "Scrap" and "Depleted Fuel").
5. This license is effective as of the date of issuance and shall expire eighteen (18) months from said date unless extended for good

cause shown, or upon the earlier issuance of a superseding operating license.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed by  
Peter A. Morris

Peter A. Morris, Director  
Division of Reactor Licensing

Attachments:  
Appendix A, Technical Specifications,  
w/Attachment A  
Appendix B, SNM Transfer Schedule

Date of Issuance:

SEP 19 1969

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APPENDIX B TO DPR-18

Estimated Schedule of Transfers of Special Nuclear Material  
from the Atomic Energy Commission (AEC) to  
Rochester Gas and Electric Corporation (RG&E) and  
to the AEC from RG&E for Robert Emmett Ginna Nuclear Power Plant Unit No. 1

Kg U-235

(1)	(2)	(3)		(4)	(5)
Date of Transfer (Fiscal Year)	Transfer from AEC to RG&E	Return from RG&E to AEC Scrap <sup>1/</sup>	Depleted Fuel <sup>4/</sup>	Net Yearly Distribution Including Cumulative Losses	Cumulative Distribution Including Cumulative Losses
1968	1,342 <sup>2/</sup>	--	--	1,342	1,342
1969	--	108 <sup>3/</sup>	--	(108)	1,234
1970	--	--	--	--	1,234
1971	535	43	--	492	1,726
1972	535	43	168 <sup>5/</sup>	324	2,050
1973	535	43	118 <sup>6/</sup>	374	2,424
1974	535	43	113 <sup>7/</sup>	379	2,803
1975	535	43	146	346	3,149
1976	535	43	146	346	3,495

CONTINUING THE SAME ANNUALLY THROUGHOUT THE

40-YEAR LIFE OF THE PLANT

2007	535	43	146	346	14,221
2008	535	43	146	346	<u>14,567</u>
2009	--	43	146	(189)	14,378
2010	--	--	765	(765)	13,613
	<u>21,672</u>	<u>1,785</u>	<u>6,274</u>	<u>13,613</u>	--

WEIGHT PER CENT U-235

- <sup>1/</sup> All 3.05% except as noted
- <sup>2/</sup> 412 kg at 2.35%; 439 kg at 2.50%; 491 kg at 2.80%
- <sup>3/</sup> 33 kg at 2.35%; 35 kg at 2.50%; 40 kg at 2.80%
- <sup>4/</sup> All 0.98% except as noted
- <sup>5/</sup> 1.11%
- <sup>6/</sup> 0.79%
- <sup>7/</sup> 0.76%