MAY 15 1973

Docket No. 50-244

Rochester Gas and Electric Corporation
ATTN: Edward J. Nelson, President
89 East Avenue
Rochester, New York 14604

Change No. 8
License No. DPR-18

Gentlemen:

By letter dated December 7, 1972, you submitted proposed changes to the Technical Specifications appended to Provisional Operating License No. DPR-18, as amended, for Unit No. 1 at the R. E. Ginna Nuclear Power Plant. The proposed changes would revise the reporting requirements section of the Technical Specifications in its entirety.

During our review of your request, we informed your staff that certain changes were necessary to meet regulatory requirements. These changes have been made.

We have reviewed your proposed changes, as modified, and have determined that they conform to Regulatory Guides No. 16 and 21. We conclude that Change No. 8 does not present significant hazards consideration and that there is reasonable assurance that the health and safety of the public will not be endangered.

Pursuant to 10 CFR Part 50, Section 50.59, the Technical Specifications appended to Provisional Operating License No. DPR-18 are changed as shown in Attachment A.

Sincerely,

Donald J. Skovholt
Assistant Director for
Operating Reactors
Directorate of Licensing

Enclosure:
Attachment A - Change No. 8
to the Technical Specifications

cc: see next page

cc w/enclosure:
Arvin E. Upton, Esquire
LeBoeuf, Lamb, Leiby & MacRae
1821 Jefferson Place, N. W.
Washington, D. C. 20036

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Notified licensee that change was signed 4:15 5/15/73 PBE

OFFICE ▶	L:ORB#1	L:ORB#1	L:ORB #1	L:ORB#1	L:OR	
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ATTACHMENT A

CHANGE NO. 8 TO THE TECHNICAL SPECIFICATIONS

ROCHESTER GAS AND ELECTRIC CORPORATION

DOCKET NO. 50-244

Delete Section 1.9 and replace with the following:

1.9 Abnormal Occurrence

Any of the following:

a. A safety system setting less conservative than the limiting setting established in the technical specifications,

b. Violation of a limiting condition for operation established in the technical specifications,

c. An uncontrolled or unplanned release of radioactive material from the site in an amount of 10% of the limits prescribed in the technical specifications,

d. Failure of one or more components of an engineered safety feature or plant protection system that causes or threatens to cause the feature or system to be incapable of performing its intended function,

e. Abnormal degradation of one of the several boundaries designed to contain the radioactive materials resulting from the fission process,

f. Uncontrolled or unanticipated changes in reactivity of greater than 1% $\Delta k/k$,

g. Observed inadequacies in the implementation of administrative or procedural controls such that the inadequacy causes or threatens to cause the existence or development of an unsafe condition in connection with the operation of the plant, and

h. Conditions arising from natural or offsite manmade events that affect or threaten to affect the safe operation of the plant.

Paragraph 6.3.3 - Last line, change to read:

"specified in Section 6.6.2.a, "Abnormal Occurrence Reports."

Delete paragraph 6.6 in its entirety and replace with the following:

6.6 PLANT REPORTING REQUIREMENTS

The following information shall be submitted to the USAEC in addition to the reports required by Title 10, Code of Federal Regulations.

6.6.1 Semiannual Operating Report

A report covering a six-month period shall be submitted in writing to the Director of Licensing, USAEC, Washington, D. C. 20545 within 60 days after January 1 and July 1 of each year and shall include the following:

a. Operations Summary

A summary of operating experience occurring during the reporting period that relates to safe operation of the facility, including a summary of:

1. changes in facility design,

2. performance characteristics (e.g., equipment and fuel performance),

3. changes in operating methods which were necessitated by 1 and 2 or which otherwise were required to improve the safety of facility operations,

4. results of surveillance tests and inspection required by these technical specifications,

5. the results of any periodic containment leak rate tests performed during the reporting period,

6. a brief summary of those changes, tests, and experiments requiring authorization from the Commission pursuant to 10 CFR Part 50, Section 50.59(a), and

7. changes in the plant operating staff for those positions which are designated as key supervisory or technical personnel on Figure 6.1-2.

b. Power Generation

A summary of power generated during the reporting period, including:

1. gross thermal power generated (in MWh),

2. gross electrical power generated (in MWh),

3. net electrical power generated (in MWh),

4. number of hours the reactor was critical,

5. number of hours the generator was on-line, and

6. histogram of thermal power vs time.

c. Shutdowns

Descriptive material covering all outages occurring during the reporting period. For each outage, information shall be provided on:

1. the cause of the outage,

2. the method of shutting down the reactor; e.g., trip, automatic rundown, or manually controlled deliberate shutdown,

3. duration of the outage (in hours),

4. unit status during the outage, e.g., cold shutdown or hot shutdown, and

5. corrective action taken to prevent repetition, if appropriate.

d. Maintenance

A discussion of corrective maintenance (excluding preventative maintenance) performed during the reporting period on safety related systems and components [Safety related is defined in ANSI 18.7-1972 (ANS-3.2, Nov. 2, 1972)] and on systems and components that reduce or prevent the release of radioactive material to the environs. For any malfunctions for which corrective maintenance was required, information shall be provided on:

1. the system or component involved,

2. the cause of the malfunction,

3. the results and effect on safe operation,

4. corrective action taken to prevent repetition, and

5. special precautions taken to provide for reactor safety during repair.

e. Changes, Tests, and Experiments

A brief description and the summary of the safety evaluation for those changes, tests, and experiments which were carried out without prior Commission approval, pursuant to the requirements of 10 CFR Part 50, Section 50.59(b) of the Commission's regulations.

f. Radioactive Effluent Releases

A statement of the quantities of radioactive effluents released from the plant with data summarized on a monthly basis following the format of USAEC Regulatory Guide 1.21.

1. Gaseous Effluents

(a) Gross Radioactivity Releases

(1) Total gross radioactivity (in curies), primarily noble and activation gases.

(2) Maximum gross radioactivity release rate during any one-hour period.

(3) Total gross radioactivity (in curies) by nuclide released based on representative isotopic analyses performed.

(4) Percent of technical specification limit.

(b) Iodine Releases

(1) Total iodine radioactivity (in curies) by nuclide released based on representative isotopic analyses performed.

(2) Percent of technical specification limit for I-131 released.

(c) Particulate Releases

(1) Total gross radioactivity (β,γ) released (in curies) excluding background radioactivity.

(2) Gross alpha radioactivity released (in curies) excluding background radioactivity.

(3) Total gross radioactivity (in curies) of nuclides with half-lives greater than eight days.

(4) Percent of technical specification limit for particulate radioactivity with half-lives greater than eight days.

2. Liquid Effluents

(a) Total gross radioactivity (β,γ) released (in curies) excluding tritium and average concentration released to the unrestricted area.

(b) The maximum concentration of gross radioactivity (β,γ) released to the unrestricted area (averaged over the period of release).

(c) Total tritium and total alpha radioactivity (in curies) released and average concentration released to the unrestricted area.

(d) Total dissolved gas radioactivity (in curies) and average concentration released to the unrestricted area.

(e) Total volume (in liters) of liquid waste released.

(f) Total volume (in liters) of dilution water used prior to release from the restricted area.

(g) Total gross radioactivity (in curies) by nuclide released based on representative isotopic analyses performed.

(h) Percent of technical specification limit for total radioactivity.

g. Solid Waste

1. The total amount of solid waste shipped (in cubic feet).

2. The total estimated radioactivity (in curies) involved.

3. Disposition including date and destination.

h. Environmental Monitoring

1. For each medium sampled during the reporting period, e.g., air, baybottom, surface water, soil, fish, include:

(a) Number of sampling locations,

(b) Total number of samples.

(c) Number of locations at which levels are found to be significantly above local backgrounds, and

(d) Highest, lowest, and the average concentrations or levels of radiation for the sampling point with the highest average and description of the location of that point with respect to the site.

2. If levels of radioactive materials in environmental media as determined by an environmental monitoring program indicate the likelihood of public intakes in excess of 1% of those that could result from continuous exposure to the concentration values listed in Appendix B, Table II, Part 20, estimates of the likely resultant exposure to individuals and to population groups, and assumptions upon which estimates are based shall be provided.

3. If statistically significant variations of offsite environmental concentrations with time are observed, correlation of these results with effluent release shall be provided.

i. Occupational Personnel Radiation Exposure

Tabulate the number of personnel exposures for facility operations personnel (permanent and temporary) in the following exposure increments for the reporting period:

less than 100 mRem, 100 - 500 mRem, 500 - 1250 mRem
1250 - 2500 mRem, above 2500 mRem.

Tabulate the number of personnel receiving more than 500 mRem exposure in the reporting period according to duty function, i.e., routine plant surveillance and inspection (regular duty), routine plant maintenance, special plant maintenance (describe maintenance), routine refueling operations, special refueling operations (describe operation), and other job related exposures. Annually tabulate the number of personnel receiving more than 2500 mRem and report major cause(s).

6.6.2 Non-Routine Reports

a. Abnormal Occurrence Reports

Notification shall be made within 24 hours by telephone and telegraph* to the Director of the Regional Regulatory Operations Office (cc to the Director of Licensing) followed by a written report within 10 days to the Director of Licensing (cc to the Director of the Regional Regulatory Operations Office) in the event of the abnormal occurrences as defined in Section 1.9. The written report on these abnormal occurrences, and to the extent possible, the preliminary telephone and telegraph notification shall: (a) describe, analyze, and evaluate safety implications, (b) outline the measures taken to assure that the cause of the condition is determined, (c) indicate the corrective action (including any changes made to the procedures and to the quality assurance program) taken to prevent repetition of the occurrence and of similar occurrences involving similar components or systems, and (d) evaluate the safety implications of the incident in light of the cumulative experience obtained from the record of previous failures and malfunctions of similar systems and components at the plant.

*Telegraph notification may be sent on the next working day in the event of an abnormal occurrence during a weekend or holiday period.

b. Unusual Events

A written report shall be forwarded within 30 days to the Director of Licensing and to the Director of the Regional Regulatory Operations Office in the event of:

1. Discovery of any substantial errors in the transient or accident analyses or in the methods used for such analyses, as described in the Safety Analysis Report or in the bases for the technical specifications.

2. Discovery of any substantial variance from performance specifications contained in the technical specifications or in the Safety Analysis Report.

3. Discovery of any condition involving a possible single failure which, for a system designed against assumed single failures, could result in a loss of the capability of the system to perform its safety function.

6.6.3 Special Reports

Special reports shall be submitted in writing within 90 days to the Director of Licensing, USAEC, Washington, D. C. 20545.

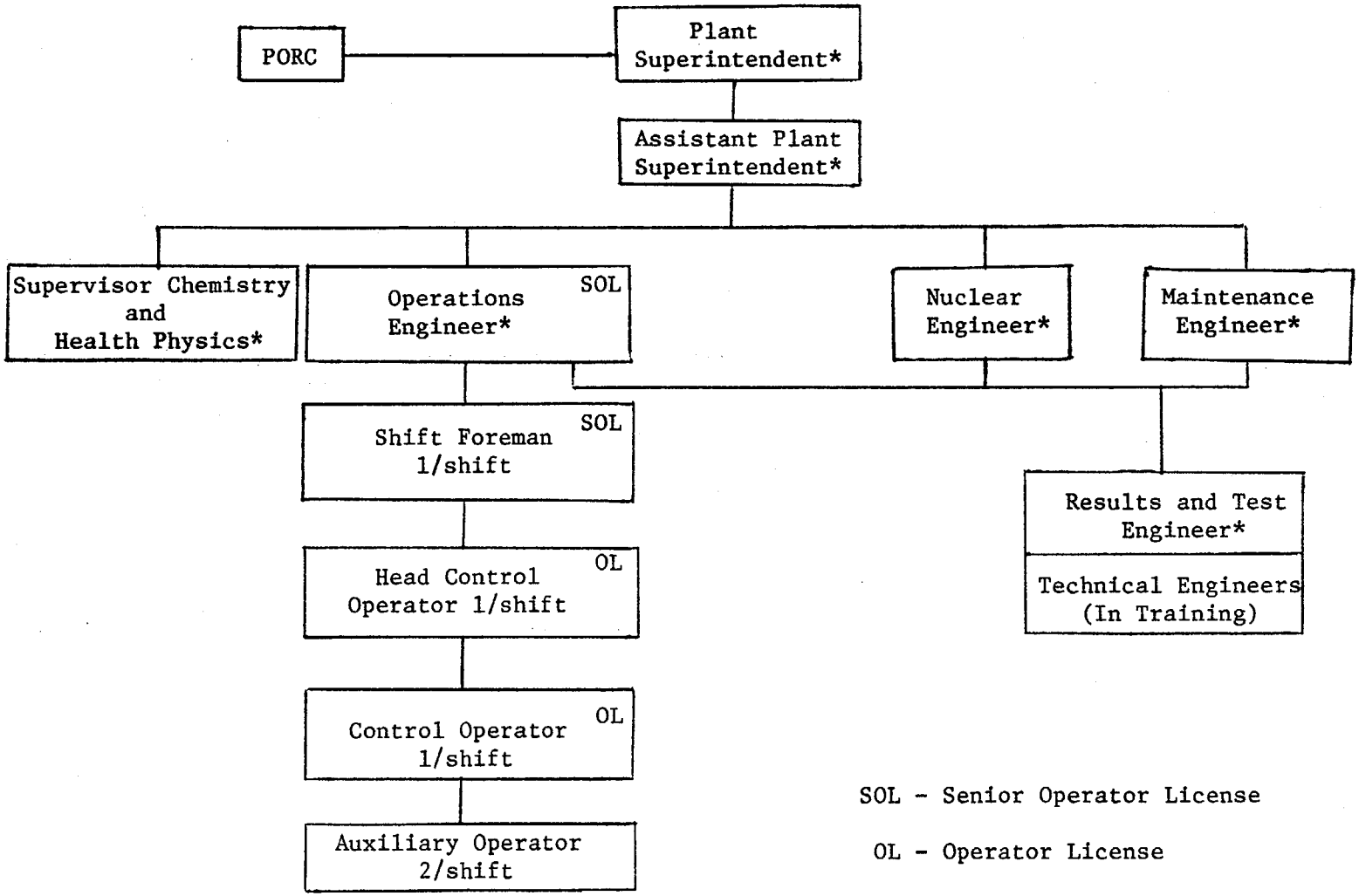
Special reports shall be submitted covering inspections, tests, and maintenance that are appropriate to assure safe operation of the plant. The frequency and content of these special reports are determined on an individual case basis and designated in these technical specifications. Such reports shall include:

a. Containment leak rate tests.

b. Materials radiation surveillance specimens reports.

R. E. GINNA NUCLEAR POWER PLANT

Conduct of Operations Chart



SOL - Senior Operator License

OL - Operator License

*Key Personnel

FIGURE 6.1-2