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1. PRIMARY SYSTEM EVENTS

A. FAILURE OF A PRIMARY RELIEF VALVE

| Condition | | Indication(s) | Emergency Classification |
|--|----|--|---|
| Failure of safety related safety valve, or relief valve, to close following a pressure reduction | 1. | Indication of flow through Pressurizer Reliefs (as indicated on Panel C5798, C5799 and C5705) | Unusual Event RA-EP-01600 All Modes |
| p | | RCS Pressure drop to <1600 psig | |

See Also: Abnormal RCS Leak Rate (2-A-1 thru 4)

NOTE

1. PRIMARY SYSTEM EVENTS

B. CORE FUEL DAMAGE

| Condition | Indication(s) | Emergency Classification |
|---|--|---|
| 1. High reactor coolant activity sample requiring plant shutdown per T.S. 3.4.8 | Confirmed primary coolant activity sample results indicate > T.S. 3.4.8 AND Plant shutdown required and in progress | Unusual Event RA-EP-01600 Modes 1 & 2 |
| 2. Very high coolant activity | Confirmed primary coolant sample results indicate >300 µCi/gram DOSE EQUIVALENT I-131 | Alert RA-EP-01700 All Modes |

NOTE

1. PRIMARY SYSTEM EVENTS

B. CORE FUEL DAMAGE (Cont.)

| | Condition | | Indication(s) | Emergency Classification |
|----|---|--------------------------------|--|--|
| 3. | Core damage with in- adequate core cooling determined | 1. | Confirmed primary coolant sample results indicate: A. DOSE EQUIVALENT I-131 >T.S. 3.4.8 OR B. >100/E µCi/gram specific activity, | Site Area Emergency RA-EP-01800 All Modes |
| | | AND 2. | The incore thermocouples indicate superheated conditions in the core | |
| 4. | Core damage with other plant conditions making a release of large amounts of radioactivity possible | 1. <u>AND</u> 2. <u>AND</u> 3. | Confirmed primary coolant sample results indicate >300 µCi/gram DOSE EQUIV-ALENT I-131 Incore thermocouple temperatures correspond to region 3 or 4 of DB-OP-O2000 Figure 2 A. Containment radiation level is > 10 4 R/hr (RE 4596A/RE 4596B) OR B. SFAS level 4 trip (CTMT pressure 38.4 psia) | General Emergency RA-EP-01900 All Modes |

See Also: Loss of Fission Product Barriers (1/C-1)
Abnormal Containment Atmosphere (1-D-1 thru 3)

NOTE

Emergency

Emergency

TABLE OF EMERGENCY ACTION LEVEL CONDITIONS

1. PRIMARY SYSTEM EVENTS

B. CORE FUEL DAMAGE (Cont.)

| | Со | nditio | n | | Indication(s) | Emergency Classification | ı |
|----|-----------|-------------------|--|---|--|---|-----|
| 5. | Core melt | situa | tions | 1. <u>AND</u> 2. | Any sequence of events has occurred in which severe core damage (such as core melting) has taken place A failure of containment is ready to take place (imminent) | General Emerge RA-EP-01900 All Modes | eno |
| | 1- | | | | | , ; | |
| | - | | | | NOTE | | |
| | | Exa in 1. OR 2. | Either a current to severe A transic feedwater auxiliary period will A transic down syst | small failur e core ent is r syst y feed ith co ent oc tems w | or large LOCA occurs with a ce of the ECCS to perform, le degradation or melting initiated by a loss of the em followed by a failure of water system for an extended re melting resulting curs requiring operation of ith failure to trip which resulting the content of the content is the content of the content | a con- eading main the d shut- | |
| | | OR 4. | a core me A failure with tota capabilit to a core A small I ECCS, how | damage and ma elt e of o il los y occ melt coca occ vever a | , or additional failures of keup systems occur which lead fished and onsite power along sof auxiliary feedwater makeurs for several hours which cours with initially success a subsequent failure of RCS sover a period of several hours where the cours with initially success and subsequent failure of RCS sover a period of several hours which we have a period of several hours which have been supported by the course of the course which lead the course which | core ad to ag ag ag ag ag ag ag a | |

See Also: Loss of Fission Product Barriers (1-C-1) Abnormal Containment Atmosphere (1-D-1 thru 3) Abnormal RCS Leak Rate (2-A-1 thru 4)

leads to a core melt

loss of Pission

1. PRIMARY SYSTEM EVENTS

C. LOSS OF FISSION PRODUCT BARRIERS

Condition

Indication(s)

Emergency Classification

 Loss of 2 of 3 fission product barriers with a potential loss of the 3rd barrier

٠:

Any TWO of the following conditions exist and the third is ready to take place (imminent):

- 1. Fuel clad rupture as indicated by confirmed primary coolant sample results indicating >300 µCi/gm DOSE EQUIVALENT I-131
- A rupture of the RCS has been confirmed with the leak rate >50 gpm.
 (Makeup tank level decreasing at a rate greater than 2 inches per minute)
- 3. Containment integrity has been breached and cannot be restored. Refer to T.S. 3.6.1.3 and T.S. 3.6.3.1.

General Emergency RA-EP-01900 All Modes

See Also: Abnormal RCS Leak Rate (2-A-1 thru 4)
Major Steam Leak (5-A-1 thru 3)

NOTE

1. PRIMARY SYSTEM EVENTS

D. ABNORMAL CONTAINMENT ATMOSPHERE

| | Condition | | Indication(s) | Emergency Classification |
|----|--|------------------------|---|--|
| 1. | Abnormal containment radiation and temperature | Bot 1. AND 2. | h of the following: Containment radiation level corresponds to an Alert as determined from the Containment Radiation EAL Plot on pages 20 or 21 Containment average air temperature indicates >170°F (TI1356, 1357, 1358) | Alert RA-EP-01700 All Modes |
| 2. | High containment radia- tion, pressure and temperature | 1. AND 2. | Containment radiation levels correspond to a Site Area Emergency as determined from the Containment Radiation EAL Plot on pages 20 or 21 A. Containment average air temperature indicates >200°F (TI1356, 1357, 1358) OR B. Safety Features Actuation System (SFAS) level 2 has activated | Site Area Emergency RA-EP-01800 All Modes |

See Also: Abnormal RCS Leak Rate (2-A-1 thru 4)
Loss of Fission Product Barriers (1-C-1)
Core Fuel Damage (1-B-1 thru 5)

NOTE

1. PRIMARY SYSTEM EVENTS

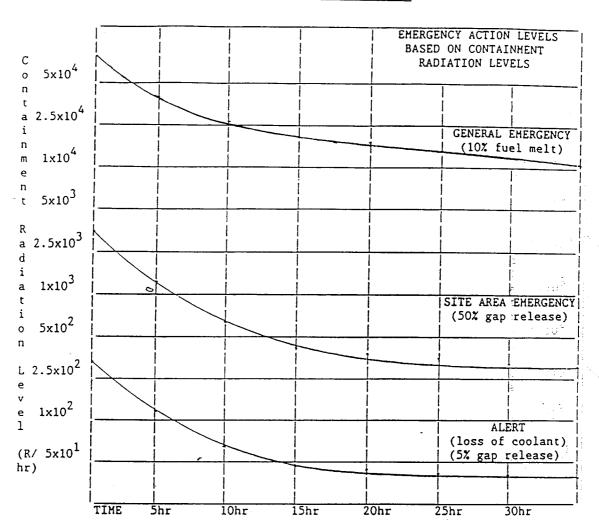
D. ABNORMAL CONTAINMENT ATMOSPHERE (Cont.)

| | Condition | | Indication(s) | Emergency Classification |
|----|--|--------------------------|--|---|
| 3. | Very high containment radiation and pressure | 1. | Containment radiation level correlates to a General Emergency as determined from the Containment Radiation EAL Plot on pages 20 or 21 | General Emergency RA-EP-01900 All Modes |
| | | $\frac{\text{AND}}{2}$. | SFAS level 4 actuation (Containment Pressure >38.4 psia) | |

See Also: Abnormal RCS Leak Rate (2-A-1 thru 4)
Loss of Fission Product Barrier (1-C-1)
Core Fuel Damage (1-B-1 thru 5)

NOTE

CONTAINMENT RADIATION EAL PLOT



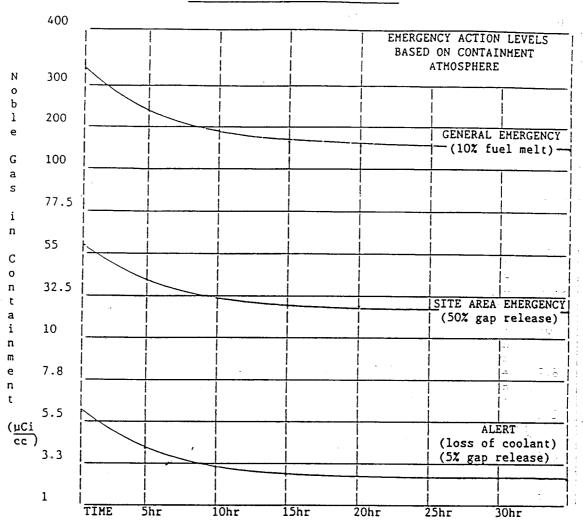
CONTAINMENT RADIATION EAL PLOT INSTRUCTIONS

The curves represent readings for monitors RE 4596A or B, Containment High Range Radiation Detector. The procedure for their use is as follows:

- 1. Determine the time after reactor shutdown
- 2. Determine the RE 4596 Channel A or B radiation reading
- 3. Find the point on the figure where these two numbers intersect
- 4. Read the classification level of the line immediately below this point.

 This is the classification to use in correlation to the "Abnormal Containment Atmosphere" section of the Emergency Action Levels.

CONTAINMENT RADIATION EAL PLOT



CONTAINMENT RADIATION EAL PLOT INSTRUCTIONS

The curves represent readings for monitors RE 4597AB or BB, Containment Atmosphere Radiation Detector. The procedure for their use is as follows:

- 1. Determine the time after reactor shutdown
- 2. Determine the RE 4597AB or BB Channel 1 or 2 radiation reading
- 3. Find the point on the figure where these two numbers intersect
- 4. Read the classification level of the line immediately below this point. This is the classification to use in correlation to the "Abnormal Containment Atmosphere" section of in the Emergency Action Levels.

2. REACTOR COOLANT SYSTEM LEAK RATE

A. ABNORMAL RCS LEAK RATE

| | | Emergency |
|---|--------------------------------------|--------------------|
| Condition | Indication(s) | Classification |
| Reactor Coolant System leak | 1. A. Any leakage occurs from the | Unusual Event |
| requiring shutdown per | pressure boundary | RA-EP-01600 |
| T.S. 3.4.6.2 (includes | <u>OR</u> | Modes 1, 2, 3, & 4 |
| primary leakage, and primary | B. RCS inventory balance indicates | |
| to secondary leakage) | >1 GPM unidentified leakage | • |
| | <u>OR</u> | |
| | C. Primary to Secondary leakage | |
| | through the tubes of any one steam | |
| | generator > 150 GPD | |
| | OR | |
| | D. RCS inventory balance indicates | |
| | >10 GPM identified leakage | |
| | OR | |
| | E. Controlled leakage from Reactor | |
| | Coolant Pump seals is > 10 GPM | |
| | total | |
| | <u>OR</u> | |
| | | |
| | F. Leakage from any RCS pressure | |
| | isolation valve listed in T.S. Table | |
| | 3.4-2 >5 GPM | |
| | AND | |
| | 2. Plant shutdown required and in | |
| | progress | |
| | | |

See Also: Major Steam Leak (5-A-1 through 3)
Loss of Fission Product Barriers (1-C-1)

Abnormal Radiation Levels at Site Boundary (6-D-1 through 7)

NOTE

2. REACTOR COOLANT SYSTEM LEAK RATE

A. ABNORMAL RCS LEAK RATE

| Condition | | Indication(s) | Emergency Classification |
|---|-----------------------|--|--|
| Reactor Coolant System leak rate >50 gpm, but within High Pressure Injection capacity (includes primary leakage, and primary to secondary leakage) | 1. <u>OR</u> 2. | Makeup tank level decreasing at a rate greater than 2 inches per minute, while RCS temperature remains steady RCS inventory balance indicates >50 gpm total leakage | Alert RA-EP-01700 All Modes |
| 3. Reactor Coolant System leak rate >50 gpm, but within High Pressure Injection Capacity (includes primary leak- age, and primary to secondary leakage) AND loss of offsite power | 1. AND 2. | A. Makeup tank level decreasing at a rate greater than 2 inches per minute, while RCS temperature remains steady OR B. RCS inventory balance indicates >50 gpm total leakage The 13.8 KV busses are de-energized | Site Area Emergency RA-EP-01800 All Modes |

See Also: Loss of Fission Product Barrier (1-C-1) Electrical Failures (4-A-1 thru 5) Safety/Relief Valve Failure (1-A-1)

NOTE

2. REACTOR COOLANT SYSTEM LEAK RATE

A. ABNORMAL RCS LEAK RATE (Cont.)

| Condition | Indication(s) | Emergency Classification |
|--|--|--|
| 4. Loss of Coolant Accident > High Pres- sure Injection system capacity | 1. High Pressure Injection system running AND 2. A. RCS pressure/pressurizer level continue to decrease | Site Area Emergency RA-EP-01800 All Modes |
| | OR B. RCS temperature/ pressure reach saturation conditions | |

See Also: Loss of Fission Product Barrier (1-C-1)

Electrical Failures (4-A-1 thru 5)

Failure of a Primary Relief Valve (1-A-1)

NOTE

3. SAFETY SYSTEM FUNCTIONS

A. CRD, RPS

| | Condition | | Indication(s) | Emergency Classification |
|----|---|--------------------------------|---|--|
| 1. | An uncontrolled control rod withdrawal from a subcritical reactor | 1. AND 2. | Outward control rod motion is indicated without a command for such motion The reactor is initially subcritical | Unusual Event RA-EP-01600 Modes 2, 3, 4, 5 |
| 2. | Failure of Reactor Protection System (RPS) to initiate and com- plete a trip which brings the reactor subcritical. | 1. AND 2. | | Alert RA-EP-01700 Modes 1 & 2 |
| 3. | Transient requiring operation of shutdown systems with failure to trip the reactor (continued power generation but no core damage immediately evident). | 1. <u>AND</u> 2. <u>AND</u> 3. | Any time plant parameters meet conditions requiring a trip RPS fails to initiate and complete a trip (automatic or manual) which brings the reactor subcriti Power interruption from the Control Room fails to bring the reactor subcritical. | Site Area Emergency RA-EP-01800 Modes 1 & 2 |

NOTE

3. SAFETY SYSTEM FUNCTIONS

B. SW, DH, CCW, MU, HPI, MFW, AFW

| Condition | Indication(s) | Emergency Classification |
|---|---|---|
| Complete loss of any functions needed for plant cold shut- down | Loss of the Low Pressure Injection/Decay Heat System (BOTH TRAINS) | Alert RA-EP-01700 Modes 1, 2, 3 & 4 |
| 2. Inability to maintain plant in cold shutdown | 1. Loss of any cooling system function needed to maintain cold shutdown (Decay Heat, Component Cooling Water, Service Water) (BOTH TRAINS). AND 2. a. An operational mode change due to temperature increase. OR b. A 30°F rise in RCS temperature. OR c. Core cooling by feed and bleed has been initiated. | Alert RA-EP-01700 Modes 5 & 6 |
| | | |

NOTE

3. SAFETY SYSTEM FUNCTIONS

B. SW, DH, CCW, MU, HPI, MFW, AFW (Continued)

| _ | Condition | | Indication(s) | Emergency Classification |
|----|--|--------|--|--|
| 3. | Complete loss of any function needed for plant hot shut-down | | s of any of the following tems: Service Water System (BOTH TRAINS) Component Cooling Water (BOTH TRAINS) A. Makeup System AND B. High Pressure Injection System (BOTH TRAINS) A. Main Feedwater System AND B. Auxiliary Feedwater- System AND C. Motor Driven Feed Pump | Site Area Emergency RA-EP-01800 Modes 1,2,3 & 4 |
| 4. | Loss of water level in the reactor vessel that has or will uncover fuel in the reactor vessel | AND 2. | Loss of any cooling system function needed to maintain cold shutdown (Decay Heat, Component Cooling Water, Service Water) (BOTH TRAINS). Indication that the core is uncovered (e.g. incores indicate superheat, containment radiation levels increasing, source range detectors increasing, etc.). | Site Area Emergency RA-EP-01800 Modes 5 & 6 |

NOTE

3. SAFETY SYSTEM FUNCTIONS

C. LOSS OF CONTROL ROOM ALARMS, INDICATION, OR COMMUNICATIONS

| | Condition | Indication(s) | Emergency Classification |
|----|---|---|--|
| 1. | Communication capability lost to an extent requiring plant shutdown or other significant loss of assessment | Complete loss of the plant telephone system <u>AND</u> Gai-tronics system | Unusual Event RA-EP-01600 All Modes |
| 2. | Most or all alarms (annunciators) lost | Any simultaneous loss of all annunciator alarms <u>AND</u> the station computer | Alert RA-EP-01700 Modes 1 & 2 |
| 3. | Most or all alarms (annunciators) lost and plant transient initiated or in progress | 1. Complete loss of all annunciator alarms AND 2. Loss of the station computer AND 3. Plant transient in progress | Site Area Emergency RA-EP-01800 Modes 1 & 2 |

NOTE

4. ELECTRICAL FAILURES

A. AC

| | Condition | | Indication(s) | Emergency Classification |
|----|---|---------------|---|---|
| 1. | Loss of offsite power 1. or loss of onsite AC power capability | | Loss of power to A and B busses from the following transformers: a. Startup 01 AND b. Startup 02 AND c. Aux 11 | Unusual Event RA-EP-01600 All Modes |
| | | OR 2. | a. Loss of power to C-1 AND D-1 busses from AC AND BD transformers. AND b. Onsite power capability has been degraded to either 4160 VAC vital bus C-1 or D-1 powered from a diesel generator | |
| | | 3. | Loss of all diesel generators. | |
| 2. | AC power capability to vital busses reduced to a single power source for greater than 15 minutes such that any additional | 1. | Loss of power to C-1 AND D-1 busses from AC AND BD transformers for greater than 15 minutes. | Alert RA-EP-01900 Modes 1,2,3 & 4 |
| | single failure would result in a station blackout. | 2. | Onsite power capability has been degraded to either 4160 VAC vital bus C-1 or D-1 powered from a diesel generator. | n fage in næ Stands Marie Silveren |

NOTE

4. ELECTRICAL FAILURES

A. AC (Continued)

| | Condition | Indication(s) | Emergency Classification |
|----|--|--|--|
| 3. | Loss of offsite power and loss of all onsite AC power | 4160 VAC vital busses C-1 AND D-1 de-energized longer than momentarily during transfers (see below for extended loss) | Alert RA-EP-01700 All Modes |
| 4. | Loss of offsite power and loss of onsite AC power for more than 15 minutes | 4160 VAC vital busses C-1 AND D-1 de-energized more than 15 minutes | Site Area Emergency RA-EP-01800 All Modes |
| 5. | Prolonged loss of all offsite power and prolonged loss of all onsite AC power. | 1. Loss of power to A and B busses from the following transformers: A. Startup 01 AND B. Startup 02 AND C. Aux 11 AND 2. 4160 VAC vital busses C-1 AND D1 are de-energized for more than 15 minutes. AND 3. A. Restoration of at least one vital bus within 4 hours is NOT likely. OR B. Indication of continuing degradation of core cooling based on fission product barrier monitoring. | |

See Also: Abnormal RCS Leak Rate With a Loss of Offsite Power (2-A-3)

NOTE

4. ELECTRICAL FAILURES

B. DC

| Inc | Condition | Emergency ation(s) Classificatio | <u>n</u> |
|--|---|--|----------|
| All in pla de-energiz for extend | oss of all onsite C power | (see below RA-EP-01700 | |
| All in pla de-energiz 15 minutes | oss of all vital onsite C power for more than 5 minutes | DC busses Site Area for more than Emergency RA-EP-01800 All Modes | |
| de-energiz | C power for more than | for more than $\begin{array}{c} {\tt Emergency} \\ {\tt RA-EP-0180} \end{array}$ | 0 |

NOTE

5. SECONDARY SYSTEM EVENTS

A. MAJOR STEAM LEAK

| | Condition | | Indication(s) | Emergency Classification |
|----|--|--------------------------------|---|---|
| 1. | Rapid depressurization of secondary side | 1. | A. Increasing containment pressure (if leak is inside containment) OR B. Unusually loud noise | Unusual Event RA-EP-01600 Modes 1, 2, 3 & 4 |
| | | AND | OR C. Visual sighting outside containment | |
| | | 2. | Valid Steam and Feedwater Rupture Control System (SFR automatic initiation on low main steam line pressure | • |
| 2. | Steam line break with >10 gpm primary to secondary leak rate | 1. <u>AND</u> 2. <u>AND</u> 3. | Indication of a major steam leak (see 5.A.1) Main steam line radiation monitor(s) indicate increased activity (RE 600/60) RCS leak rate >10 gpm as indicated by: A. Makeup tank decreasing >1/2 inch per minute OR B. RCS inventory balance indicates >10 gpm leak rate OR C. DB-CH-01814, Steam Generator Tube Leak Determination | Alert RA-EP-01700 Modes 1, 2, 3 & 4 |

NOTE

5. <u>SECONDARY SYSTEM EVENTS</u>

A. MAJOR STEAM LEAK (Cont.)

| Condition | Indication(s) | Emergency Classification |
|---|--|--|
| 3. Steam line break with >50 gpm primary to secondary leakage AND indication of fuel damage | 1. Indication of a major steam leak (see 5.A.1) AND 2. Main steam line radiation Monitor(s) indicate increased activity (RE 600/609 | Site Area Emergency RA-EP-01800 Modes 1, 2, 3 & 4 |
| | AND 3. RCS leak rate >50 gpm as indicated by: A. Makeup tank decreasing >2 inches per minute OR B. RCS inventory balance indicates >50 gpm leak rate | |
| | AND 4. Confirmed primary coolant sample results indicate activity above acceptable limits of T.S. 3.4.8 | |

NOTE

5. SECONDARY SYSTEM EVENTS

B. MAIN STEAM SAFETY VALVE FAILURE

| Condition | Indication(s) | Emergency Classification |
|--|---|---|
| 1. Failure of safety related safety valves, or relief valves, to close following a pressure reduction AP | decrease in steam gen- erator pressure to <500 psig | Unusual Event RA-EP-01600 Modes 1,2,3 & 4 |
| 2. | <u>=</u> | |

See Also: Major Steam Leak (5-A-1 through 5-A-3)

NOTE

6. RADIATION RELEASE EVENTS

A. HIGH RADIATION LEVELS WITHIN

THE PROTECTED AREA

Emergency Condition Indication(s) Classification 1. Radiation levels or air-A. An area radiation Alert borne contamination which survey indicates RA-EP-01700 indicates a severe radiation levels All Modes degradation in the >1000 times normal control of radioactive materials (such as an B. Airborne radioactivity increase of a factor of sample indicates activity 1000 in direct radiation levels >1000 times normal readings) C. If an area of the plant is inaccessable, a radiation monitor reading indicating radiation levels >1000 times normal

NOTE

Emergency

TABLE OF EMERGENCY ACTION LEVEL CONDITIONS

6. RADIATION RELEASE EVENTS

B. FUEL HANDLING ACCIDENT

| Condition | Indication(s) | Classification |
|---|--|--|
| 1. Fuel handling accident which results in the release of radio-activity to containment or fuel handling area | Direct information from fuel handling personnel indicating that an irradiated fuel assembly has been damaged and radioactive gases are escaping | Alert RA-EP-01700 All Modes |
| 2. Major damage to spent fuel in containment or fuel handling area (e.g., large object damages fuel or water loss below fuel level) | 1. Indications of fuel hand- ling accident which results in the release radioactivity to containment or spent fuel pool area AND 2. A. SFAS incident Level 1 actuation on radiation in containment OR B. Isolation of venti- lation in containment or spent fuel pool area based on radiation. | Site Area Emergency RA-EP-01800 All Modes |

NOTE

The USAR analyzed fuel handling accident (Chapter 15) postulates the failure of 56 fuel pins from an assembly at maximum burnup, 72 hours after reactor shutdown.

Fuel repair activities involving the handling of an individual fuel pin are covered under the maintenance exception of Section 6.1.1.

NOTE

6. RADIATION RELEASE EVENTS

C. ABNORMAL EFFLUENT RELEASE

| | Condition | | Indication(s) | Emergency Classification |
|----|--|------------------|--|---|
| 1. | Effluent release > limits allowed by the Offsite Dose Calculation Manual | The 1. | following combination: Any confirmed effluent release exceeding the limits of the ODCM. | Unusual Event RA-EP-01600 All Modes |
| | (ODCM): ODCM Section 2.3.1 ODCM Section 2.4.1 ODCM Section 3.3.1 | $\frac{OR}{2}$. | A high alarm is received on any of the following Radiation Monitoring | |
| | ODCM Section 3.7.1 ODCM Section 3.8.1 | | System monitors for greater than 15 minutes during a release (alarm setpoint established by the Chemistry Department) A. 1878A or B, Miscel- | |
| | | | laneous Waste Outlet OR B. 1770A or B, Clean Waste Outlet OR | |
| | | | C. 1822A or B, Waste Gas Outlet | |
| | | AND 3. | The associated discharge valve fails to close (automatically OR manually) | |
| | | AND 4. | · — | |

NOTE

6. RADIATION RELEASE EVENTS

C. ABNORMAL EFFLUENT RELEASE (Cont.)

| _ | Condition | | Indication(s) | Emergency Classification |
|----|--|------------------|---|--|
| 2. | Effluent release >10 times limits allowed by the Offsite Dose Calculation Manual (ODCM): | | following combination: Any confirmed effluent release exceeding the ODCM Limits by >10 times the limits | Alert RA-EP-01700 All Modes |
| | ODCM Section 2.3.1 ODCM Section 2.4.1 ODCM Section 3.3.1 ODCM Section 3.7.1 | $\frac{OR}{2}$. | A high alarm is received on any of the following Radiation Monitoring | |
| | ODCM Section 3.7.1 | | System monitors at 10 times setpoint (as established by the the Chemistry Department) A. 1878A or B, Miscel- laneous Waste Outlet OR B. 1770A or B, Clean Waste Outlet OR C. 1822A or B, Waste Gas Outlet | e professional de la constanción de la constanci |
| | | AND 3. | The associated discharge valve fails to close (automatically OR manually) | |
| | •• | AND 4. | Chemistry Unit or Radiation Protection Section confirms that an ODCM has been exceed | ded. |

NOTE

. Amai y

TABLE OF EMERGENCY ACTION LEVEL CONDITIONS

6. RADIATION RELEASE EVENTS

D. ABNORMAL RADIATION LEVELS AT SITE BOUNDARY

NOTE 6.D.1

RE 4598 indication is based on average meteorological conditions: stability class D, wind speed 10 mph.

| Condition | Indication(s) | Emergency Classification |
|--|---|-----------------------------------|
| 1. Projected or actual site boundary radiation levels that indicate a potential dose of about 1 mrem at the site boundary if continued over 2 hours. | Station Vent RE 4598 Channel 1 reading ≥3.6E-3 μci/cc (Noble Gas) for 2 hours OR 2. 0.5 mrem/hr measured at the Site Boundary for 2 hours | Alert RA-EP-01700 All Modes |

NOTE 6.D.2

RE 4598 indications are based on adverse meteorological conditions: stability class F, wind speed 2 mph.

| Condition | | Indication(s) | Emergency Classification |
|---|-------|---|---------------------------------------|
| 2. Projected or measured site boundary Total Effective Dose | 1. | Station Vent RE 4598 Channel 1 (Noble Gas) indicates 1.6E-2 µci/cc | Site Area Emergency RA-EP-01800 |
| Equivalent (TEDE) rate ≥50 mrem/hr for ½ hour. | OR 2. | or greater for % hour 50 mrem/hr by direct measurement at the site boundary for % hour | All Modes |

NOTE

6. RADIATION RELEASE EVENTS

D. ABNORMAL RADIATION LEVELS AT SITE BOUNDARY (Cont.)

NOTES 6.D.3, 6.D.4 and 6.D.5

RE 4598 indications are based on adverse meteorological conditions: stability class F, wind speed 2 mph.

| 3. | Projected or measured site boundary TEDE rate >500 mrem/hr for 2 minutes | 1. <u>OR</u> 2. | Station Vent RE 4598 Channel 1 (Noble Gas) indicates 1.6E-1 µci/cc or greater for 2 minutes | Site Area Emergency RA-EP-01800 All Modes |
|----|--|-----------------------|---|--|
| | | 2. | 500 mrem/hr by direct measurement at the site boundary for 2 minutes | |
| 4. | Projected or measured site boundary thyroid dose rate > 250 mrem/hr for % hour | 0R 2. | Station Vent RE 4598 Channel 3 (Iodine) indicates 3.7E-6 µci/cc or greater for ½ hour Radioiodine of 7.9E-8 µci/cc by direct measurement at the Site Boundary for ½ hour | Site Area Emergency RA-EP-01800 All Modes |
| 5. | Projected or measured site boundary thyroid dose rate ≥ 2500 mrem/hr for 2 minutes | 1. <u>OR</u> 2. | Station Vent RE 4598 Channel 3 (Iodine) indicates 3.7E-5 µci/cc or greater for 2 minutes Radioiodine of 7.8E-7 µci/cc by direct measurement at the Site Boundary for 2 minutes | Site Area Emergency RA-EP-01800 All Modes |

NOTE

6. RADIATION RELEASE EVENTS

D. ABNORMAL RADIATION LEVELS AT SITE BOUNDARY (Cont.)

NOTE 6.D.6 and 6.D.7

RE 4598 indications are based on adverse meteorological conditions: stability class D, wind speed 10 mph.

- 6. Projected or measured TEDE rate of 1 rem/hr or greater at the Site Boundary.
- 1. Station Vent RE 4598
 Channel 1 (Noble Gas)
 indicates 6.9E-1 µci/cc
 or greater.

1 rem/hr by direct measurement at the Site Boundary.

General Emergency RA-EP-01900 All Modes

OR

- 7. Projected or measured thyroid dose rate of 5 rem/hr or greater at the Site Boundary
- 1. Station Vent RE 4598
 Channel 3 (Iodine)
 indicates 1.8E-3 µci/cc
 or greater.

General Emergency RA-EP-01900 All Modes

OR
2. Radioiodine of
1.7E-6 μci/cc by
direct measurement
at the Site Boundary.

NOTE

7. HAZARDS TO STATION OPERATIONS

A. FIRE

| | Condition | Indication(s) | Emergency Classification |
|----|---|--|--|
| 1. | Fire within the plant lasting more than 10 minutes | Any fire within the protected area lasting mor than 10 minutes from the initiation of fire suppression (manually or automatically), NO safe systems affected Any fire which requires offsite assistance | e RA-EP-01600 e All Modes ty |
| 2. | Fire potentially affecting safety systems | Any fire at the station tha has the potential to damage or degrade a safety system | |
| 3. | Fire resulting in the loss of redundant trains of a safety system | Any fire that defeats the capability of both trains of a safety system | Site Area Emergency RA-EP-01800 All Modes |

NOTE

7. HAZARDS TO STATION OPERATIONS

B. AIRCRAFT CRASH

| _ | Condition | Indication(s) | Emergency Classification | |
|--|---|--|--|--|
| Aircraft crash onsite or unusual aircraft activity over facility | | Control room informed by station personnel who have made a visual sighting | Unusual Event RA-EP-01600 All Modes | |
| 2. | Aircraft crash affecting plant structures | Control room informed by station personnel who have made a visual sighting | Alert RA-EP-01700 All Modes | |
| 3. | Aircraft crash damaging vital structures by impact or fire. | Control room informed by station personnel who have made a visual sighting AND Instrumentation readings on vital systems indicate equipment problems | Site Area Emergency RA-EP-01800 Modes 1,2,3&4 | |

NOTE

7. HAZARDS TO STATION OPERATIONS

C. TRAIN DERAILMENT

| Condition | Indication(s) | Emergency Classification |
|----------------------------|---|---|
| 1. Train derailment onsite | 1. Control room informed by station personnel who have made a visual sighting AND 2. A. Station structures have been damaged OR B. Danger to station personnel exists | Unusual Event RA-EP-01600 All Modes |

NOTE

7. HAZARDS TO STATION OPERATIONS

D. EXPLOSION

| | Condition | | Indication(s) | Emergency Classification |
|----|--|--|--|--|
| 1. | Near or onsite explosion | Control room informed by station personnel who have made a visual sighting | | Unusual Event RA-EP-01600 All Modes |
| 2. | Onsite explosion affecting plant opera- tions | 1. AND 2. | | All Modes |
| 3. | Explosion causing severe damage to safe shutdown equipment | _ | losion causing loss of: Makeup system <u>AND</u> HPI system Ability to supply feed water to the OTSG's | Site Area Emergency RA-EP-01800 Modes 1,2,3&4 |

NOTE

RA-EP-02840, Explosion, contains further detailed information.

NOTE

7. HAZARDS TO STATION OPERATIONS

E. TOXIC OR FLAMMABLE GAS

Emergency Condition Indication(s) Classification 1. Near or onsite toxic or Report or detection of Unusual Event flammable gas release toxic or flammable gases RA-EP-01600 that could enter within All Modes the Owner Controlled Area in amounts that can affect normal operation of the plant. Report by local, county or State officials for potential evacuation of Owner Controlled Area personnel based on offsite events. 2. Entry into facility 1. Report or detection of Alert environs of uncontrolled toxic or flammable gas RA-EP-01700 toxic or flammable gas within a Protected Area All Modes structure in concentrations that will be threatening to plant personnel. Report or detection of toxic or flammable gases within a Protected Area structure in concentrations that will affect the safe operation of the plant.

NOTE

RA-EP-02850, Hazardous Chemical and Oil Spills, contains further detailed information.

NOTE

- 7. HAZARDS TO STATION OPERATIONS
- E. TOXIC OR FLAMMABLE GAS (Continued)

| Condition | | Indication(s) | Emergency Classification |
|--|----|---|--|
| 3. Entry of uncontrolled flammable gases into vital areas. Entry of uncontrolled toxic gases into vital areas where lack of access to the area constitutes a safety problem. (Plant not in cold shutdown.) | 1. | Report or detection of toxic or flammable gases within Vital Areas where lack of access to the area prevents operation of BOTH TRAINS of a safety system. | Site Area Emergency RA-EP-01800 Modes 1,2,3&4 |

NOTE

RA-EP-02850, Hazardous Chemical and Oil Spills, contains further detailed information.

NOTE

7. HAZARDS TO STATION OPERATIONS

F. TURBINE DAMAGE

| | Condition | Indication(s) | Emergency Classification |
|----|---|--|---|
| 1. | Turbine rotating component failure causing rapid plant shutdown | High turbine vibration trip AND 2. Reactor trip | Unusual Event RA-EP-01600 Modes 1 & 2 |
| 2. | Turbine failure CAS- ing casing penetrate- tion | Control room informed by Station personnel who have made a visual inspection of turbine casing | Alert RA-EP-01700 Modes 1 & 2 |

NOTE

7. HAZARDS TO STATION OPERATIONS

G. MISSILE IMPACT

| _ | Condition | | Indication(s) | Emergency Classification |
|----|---|-----|--|--|
| 1. | Missile impact from whatever source on the facility | Sta | trol room informed by tion personnel of any sile | Alert RA-EP-01700 All Modes |
| 2. | Missile impact causing severe damage to safe shutdown equipment | AND | Control room informed by Station personnel of any missile impact on safe shutdown equipment Instrumentation readings on safe shutdown equipment indicate equipment problems | Site Area Emergency RA-EP-01800 Modes 1,2,3&4 |

NOTE

7. HAZARDS TO STATION OPERATIONS

H. CONTROL ROOM EVACUATION

| Condition | Indication(s) | Emergency Classification |
|---|---|--|
| 1. Evacuation of control room anticipated or required | Any evacuation of the control room anticipated or required with control of shutdown systems established from local stations within 15 minutes | Alert RA-EP-01700 All Modes |
| 2. Evacuation of control room and control of shutdown systems NOT established from local stations in 15 minutes | Any evacuation of the control room with shut-down control NOT established locally within 15 minutes | Site Area Emergency RA-EP-01800 All Modes |

NOTE

7. HAZARDS TO STATION OPERATIONS

I. SECURITY THREAT

| | Condition | e | Indication(s) | Emergency Classification |
|----|--|--------------------------|--|--|
| 1. | Security threat or attempted entry or attempted sabotage | of a property | cort by plant personnel a security threat with cotential for industrial cotage (i.e. attempted cible entry into a vital ea, armed entry into the otected area, discovery of spected bombs or incendiary vices, etc.) | Unusual Event RA-EP-01600 All Modes |
| 2. | Ongoing security compromise | of tha | oort by a member the security force at a security emer- acy is in progress | Alert RA-EP-01700 All Modes |
| 3. | Loss of physical con- trol of the plant is ready to take place (imminent) | pla occ roc sta | rsical attack on the int involving imminent eupancy of the control om <u>OR</u> local shutdown tions that control eal equipment | Site Area Emergency RA-EP-01800 All Modes |
| 4. | Loss of physical control of the facility | 1. <u>OR</u> 2. | Physical attack on the plant which has resulted in occupation of the control room Unauthorized personnel in control of vital plant equipment | General Emergency RA-EP-01900 All Modes |

NOTE

8. NATURAL EVENTS (WITHIN OTTAWA COUNTY)

A. EARTHQUAKE

| | Condition | Ind | ication(s) | Emergency Classification |
|----|---|-------------------------------|---|--|
| 1. | Any earthquake felt in- plant or detected on station seismic instru- mentation | | uake felt in-plant d by station seismic ation | Unusual Event RA-EP-01600 All Modes |
| 2. | Earthquake > Operating Basis Earthquake (OBE) levels | AND 2. OBE al | motion felt arm on seismic panel C5764A | Alert RA-EP-01700 All Modes |
| 3. | Earthquake >Safe Shutdown Earthquake (SSE) levels | $\frac{\text{AND}}{2}$ SSE al | motion felt arm on seismic panel C5764A | Site Area Emergency RA-EP-01800 Modes 1,2,3&4 |

NOTE

 $\ensuremath{\text{RA-EP-02820}}\xspace$, Earthquake, contains further detailed information.

NOTE

8. NATURAL EVENTS (WITHIN OTTAWA COUNTY)

B. TORNADO

| Condition | Indication(s) | Emergency Classification |
|----------------------------------|--|---|
| 1. Any tornado onsite | Control room informed by station personnel who have made a visual sighting of a tornado crossing the site boundary | Unusual Event RA-EP-01600 All Modes |
| 2. Any tornado striking facility | Control room informed by station personnel who have made a visual sighting of a tornado striking the facility | Alert RA-EP-01700 All Modes |

NOTE

RA-EP-02810, Tornado, contains further detailed information.

NOTE

8. NATURAL EVENTS (WITHIN OTTAWA COUNTY)

C. HURRICANE FORCE WINDS

| 1. | Condition Hurricane force winds (greater than 74 mph) | of | Indication(s) trol room informed hurricane force winds ecast for Ottawa | Emergency Classification Unusual Event RA-EP-01600 All Modes |
|----|--|------------------|--|--|
| 2. | Hurricane force winds near design basis levels (greater than 74 mph, but less than 90 mph) | 1. ANE 2. | | Alert RA-EP-01700 All Modes |
| 3. | Hurricane force winds > design basis levels (greater than 90 mph) | 1. <u>AND</u> 2. | | Site Area Emergency RA-EP-01800 Modes 1,2,3&4 |

NOTE

8. NATURAL EVENTS (WITHIN OTTAWA COUNTY)

D. FLOODING, LOW WATER

| _ | Condition | Indication(s) | Emergency Classification |
|----|--|--|---|
| 1. | 50 year flood or low water, surge or seiche | Forebay level observed to be: 1. High (>580 feet IGLD) OR 2. Low (<562 feet IGLD) | Unusual Event RA-EP-01600 All Modes |
| 2. | Flood, low water, surge or seiche near design levels | Forebay level observed to be: 1. High (584 feet IGLD) OR 2. Low (<560 feet IGLD) | Alert RA-EP-01700 All Modes |
| 3. | Flood, low water, surge or seiche > design levels with plant not in cold shutdown | Forebay level observed to be: 1. High (>584 feet IGLD) OR 2. Low (<558 feet IGLD) | Site Area Emergency RA-EP-01800 Modes 1,2,3,&4 |

NOTE

 $\mbox{RA-EP-}02830,$ Flooding, contains further detailed information on high water situations.

NOTE

9. MISCELLANEOUS

| Condition | Indication(s) | Emergency Classification |
|--|--|--|
| Inability to reach required shutdown within technical specification limi | <pre>a required operating mode within a Technical</pre> | Unusual Event RA-EP-01600 Modes 1,2,3,&4 |
| 2. Miscellaneous | Other plant conditions exist that warrant increased awareness on the part of the plant operations staff or State and/or local offsite authorities which are not covered under any other existing station procedures. | Unusual Event RA-EP-01600 All Modes |
| 3. Miscellaneous | Other plant conditions exist that warrant precautionary activation of the Technical Support Center and Emergency Control Center and placing other key emergency personnel on standby. | Alert RA-EP-01700 All Modes |
| 4. Miscellaneous | Other plant conditions exist that warrant activation of emergency centers and monitoring teams or a precautionary notification to the public near the site. | Site Area Emergency RA-EP-01800 All Modes |

NOTE

9. MISCELLANEOUS (Cont.)

| Condition | Indication(s) | Emergency Classification |
|------------------|---|---|
| 5. Miscellaneous | Other plant conditions exist, from whatever source, that make release of large amounts of radioactivity in a short time period possible, e.g., any core melt situation. | General Emergency RA-EP-01900 All Modes |

NOTE

10. DOWNGRADING GUIDELINES

- A. Existing conditions no longer meet the emergency criteria $\frac{AND}{i\,t}$ appears unlikely that conditions will deteriorate further.
- B. Nonroutine releases of radioactive material to the environment are under control or terminated.
- C. Any fire, flood, earthquake, or similar emergency conditions are controlled or have ceased.
- D. All specified corrective actions have occurred $\frac{OR}{the} \ \ plant \ has \ been \ placed \ in \ the \ appropriate \ operational \ mode.$
- E. All required notifications have been completed.
- F. Agreement between the Technical Support Center and the Emergency Control Center that downgrading is appropriate (if they were activated).
- G. After issuance of offsite protective actions has occurred, State and County officials must concur with the downgrading.

11. TERMINATING GUIDELINES

- A. The conditions which caused the emergency have stabilized, are under control, and are unlikely to deteriorate further.
- B. No surveillance relative to offsite protective actions is needed, except for the control of foodstuffs, water, and offsite contamination, or environmental assessment activities.
- C. Radiation levels in affected plant areas are acceptable, and/or are stable or decreasing.
- D. Releases of radioactive material to the environment greater than Offsite Dose Calculation Manual are under control or have ceased.
- E. The potential for an uncontrolled release of radioactive material is at an acceptable, low level.
- F. Containment pressure is within Technical Specification requirements related to the existing mode of operation.
- G. The reactor is in a stable, safe shutdown condition and long-term core cooling is available, as required.
- H. Any fire, flood, earthquake, or similar emergency conditions no longer exist.
- I. All offsite notifications are complete.
- J. Offsite conditions will not limit access of personnel and resources.
- K. Discussions have been held with those federal, state and local organizations that have mobilized in support of the emergency, and that are in direct communication with DBNPS.
- L. The Technical Support Center (TSC) staff, if activated, has evaluated plant status with respect to Technical Specifications and concurs with termination of the emergency.

7.0 FINAL CONDITIONS

Abnormal plant conditions have been terminated and an Emergency Director is no longer required.

8.0 RECORDS

- 8.1 The following quality assurance records are completed by this procedure and shall be listed on the Nuclear Records List, captured, and submitted to Nuclear Records Management in accordance with NG-NA-00106:
 - 8.1.1 None
- 8.2 The following non-quality assurance records are completed by this procedure and may be captured and submitted to Nuclear Records Management in accordance with NG-NA-00106:
 - 8.2.1 None

ATTACHMENT 13: CONDITIONS AFFECTING EDG OPERABILITY

| | PARAMETER | CONDITION | RESULT |
|-----|--|--|---|
| 1. | EDG Room Temperature | Greater than 120°F | Qualification issue. Contact Engineer in accordance with DB-OP-02037. |
| 2. | EDG Room TIC | Loss of power or not functioning (Removing any damper actuator from service causes a loss of control signal to all actuators) | EDG Inoperable. EDG Ventilation will not maintain EDG Room Temperature less than 120°F. |
| 3. | EDG Ventilation | Outside air temperature greater than 68°F with less than both Supply Air Fans OPERABLE. | EDG Inoperable |
| | - | Exception: With one Supply Air Fan inoperable, if outside air temperature is below 68°F and the inoperable fan is blanked off. | EDG OPERABLE |
| 4. | Lube Oil Temperature | Less than 85°F as indicated on TI 20173 (EDG 1) TI 20174 (EDG 2) | EDG Inoperable |
| 5. | Circulating (Soak Back) Oil Pump | Not operating when the EDG is in Standby | EDG Inoperable |
| 6. | Turbo Oil Pumps | Both AC and DC pumps not running | EDG Inoperable |
| 7. | Lube Oil Return Line from Camshaft Area | Qil Level not visible in lower sightglass | EDG OPERABLE but requires attention. |
| | • | Oil Level not visible in upper sightglass while running | EDG OPERABLE but requires attention. |
| 8. | Governor Oil Level | Low out of the sightglass | EDG Inoperable |
| 9. | Lube Oil Sump Level | Less than the LOW mark on the dipstick while in Standby Mode | EDG Inoperable |
| 10. | Immersion Heater | Breaker open or heater not functioning | EDG OPERABLE unless lube oil temperature drops below 85°F. |
| 11. | Cooling Water Expansion Tank Level | Less than the operating level bottom of sightglass | EDG Inoperable |
| 12. | Starting Air Receivers | Both less than 210 PSIG for an EDG | EDG Inoperable |

ATTACHMENT 13: CONDITIONS AFFECTING EDG OPERABILITY (Continued)

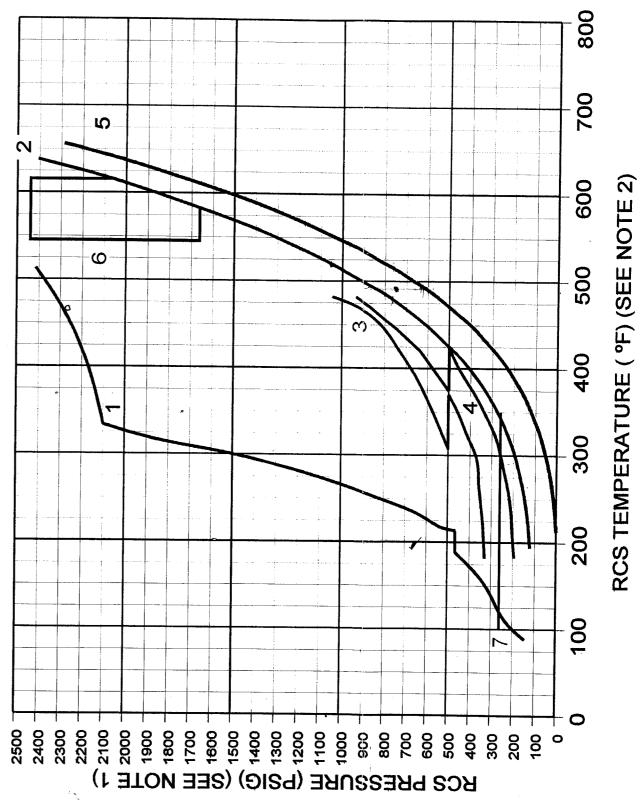
| | <u>PARAMETER</u> | CONDITION | RESULT |
|-----|--|--|---|
| 13. | Starting Air Regulator Outlet Pressure | Both less than 140 PSIG for an EDG as indicated on PI 2987 - DA 30 air start side PI 2988 - DA 44 air start side PI 2989 - DA 31 air start side PI 2994 - DA 45 air start side | EDG Inoperable |
| 14. | DC Control Power | Loss of power | EDG Inoperable |
| 15. | AC Control Power | Loss of power | EDG Inoperable. Power is lost to the EDG Room TIC. |
| 16. | EDG Fuel Oil Storage Tank Level | Less than 32,000 gallons | EDG Inoperable |
| 17. | EDG Fuel Oil Transfer Pump | Loss of power or not functioning | EDG Inoperable |
| 18. | EDG Fuel Oil Day Tank Level | Less than 4,000 gallons | EDG Inoperable |
| 19. | DC Motor Driven Fuel Oil Pump | Loss of power or not functioning | EDG Inoperable if not running. The diesel can be started but may not meet the fast start criteria (<10 sec). Operable if running. |
| 20. | Ambient Air Temperature (Computer Point M012 or | Use the flowchart on the next page. | en e |

^{*}Equivalent is defined as a direct reading at the 10 meter elevation at the Meteorological Tower or the Toledo temperature provided on www.weather.com. Computer Point M012 reads only to 100°F.

equivalent)*

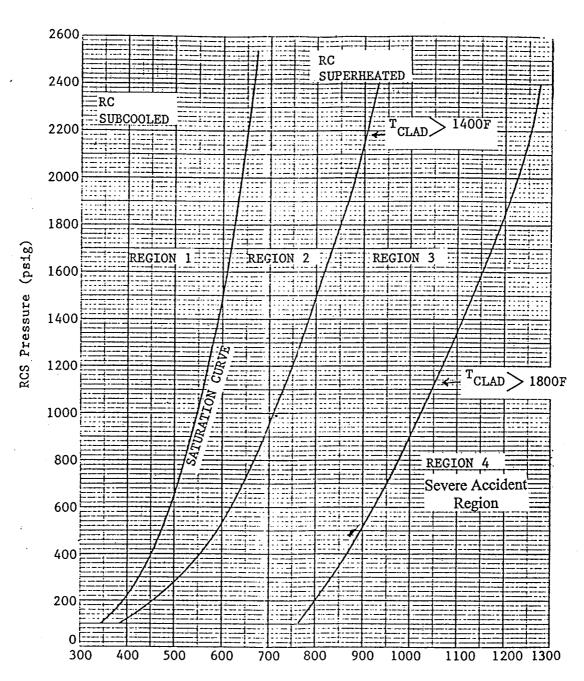
| Curve | e Title |
|---|---|
| 1 | Maximum RCS P/T for Cooldown |
| 2 | Minimum RCS P/T to Maintain RCS Subcooled |
| 3 | Minimum RCS P/T to provide NPSH with one |
| | RCP operating in a Loop |
| 4 | Minimum RCS P/T to provide NPSH with two |
| | RCP's operating in a Loop |
| 5 | Saturation Curve |
| 6 | Abnormal Transient Envelope |
| 7 | Maximum RCS P/T during simultaneous operation of |
| | DH Pumps and normal combination of RC Pumps |
| Note 1 - | Curves 1, 2, and 6 |
| | RCS Pressure indicated on RCS Wide Range Pressure Indicator PI-RC2A4 (P732) or PI-RC2B4 (P724). |
| · | Curves 3 and 4 |
| | With RCS Pressure greater than 500 psig, use RCS |
| | Wide Range Pressure Indicators PI-RC2A4 (P732), or PRS-RC2A1, or PI RC2B4 (P724) |
| | With RCS Pressure less than 500 psig, use RCS Hot Leg Low Range Pressure Indication from PI RC2A6 |
| | |
| Note 2 - | Curves 1, 3, 4, and 7 |
| | RCS Temperature indicated on RCS Cold Leg Temperature Indicator TI RC4A2 or TI RC4B2 |
| | Curves 2 and 6 |
| | RCS Temperature based on average of Core Exit Thermocouple readings |
| Note 3 - | Curves 1, 2, 3, 4, and 7 are corrected for instrument error to ensure limits are not exceeded. |
| Andrew | Curve 5 is not instrument error corrected since it is a water saturation curve only. |
| | Curve 6 is not instrument error corrected as it is guidance only, not a limit. |

RCS PRESSURE/TEMPERATURE LIMITS



DB-OP-02000 Revision 05 C-1

FIGURE 2
Incore T\C
Temperature vs
RCS Pressure
for ICC



Incore Thermocouple Temperature (F)

Figure 2 Sheet 1 of 1