



W. R. McCollum, Jr.
Vice President

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February 15, 2001

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555-0001

Subject: Duke Energy Corporation
Oconee Nuclear Station, Units 1, 2 and 3
Docket Numbers 50-269, 50-270 and 50-287
Supplemental Information - Proposed Technical Specification Amendment
Generic Letter 96-04 - Spent Fuel Storage Racks (TSCR 2000-01)

On December 28, 2000, Duke Energy Corporation (Duke) submitted a License Amendment Request (LAR) for Facility Operating License Nos. DPR-38, DPR-47, and DPR-55 for Oconee Nuclear Station Units 1, 2, and 3, respectively, pursuant to 10 CFR 50.90. The LAR proposed revised Technical Specification (TS) requirements for fuel storage due to degradation of the Boraflex panels in the Oconee spent fuel pool storage racks.

During performance of crosschecks of data sources for the basis of the LAR, the minimum burnups for Unrestricted fuel storage in the Oconee spent fuel pools were determined to have been incorrectly stated for 2.00 wt % U-235 fuel. This occurred as a result of typographic errors identified in the governing criticality analysis. This letter provides replacement pages to the LAR to correct these minor non-conservatisms.

Enclosure 1 to this letter provides replacement mark-up pages for insertion in LAR Attachment 1. Enclosure 2 provides replacement TS pages to the original LAR Attachment 2. Enclosure 3 provides replacement pages for LAR Attachment 6. Each Enclosure's cover sheet provides page change instructions.

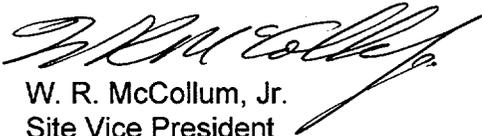
This revision does not affect the No Significant Hazards Consideration Evaluation and Environmental Assessment/Impact Statement for the LAR.

Pursuant to 10CFR50.91, a copy of this revision is being provided to the State of South Carolina.

AD68

Please contact Robert C. Douglas at 864-885-3073 with any questions regarding this submittal.

Very truly yours,



W. R. McCollum, Jr.
Site Vice President
Oconee Nuclear Station

Enclosures

c: (w/attachments)

L.A. Reyes
Administrator, Region II

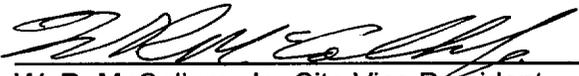
M. C. Shannon
NRC Senior Resident Inspector
Oconee Nuclear Station

D. E. LaBarge
ONRR, Senior Project Manager

V.R. Autry, Director
DHEC

AFFIDAVIT

W. R. McCollum, Jr., being duly sworn, states that he is Site Vice President of Duke Energy Corporation; that he is authorized on the part of said corporation to sign and file with the Nuclear Regulatory Commission this revision to the Oconee Nuclear Station License Nos. DPR-38, DPR-47, and DPR-55; and that all statements and matters set forth therein are true and correct to the best of his knowledge.



W. R. McCollum, Jr., Site Vice President

Subscribed and sworn to me: FEBRUARY 15, 2001
Date

Notary Public: Robert C. Douglas

My Commission Expires: August 13, 2009
Date

Seal

U. S. Nuclear Regulatory Commission
February 15, 2001

ENCLOSURE 1

REPLACEMENT MARK-UP PAGES

**FOR ATTACHMENT 1 TO DECEMBER 28, 2000
LICENSE AMENDMENT APPLICATION**

Replace the following listed Attachment 1 pages with the enclosed replacement pages:

Insert 3 - Page 3.7.13-3

Insert 6 - Page 3.7.13-6

NOTE:

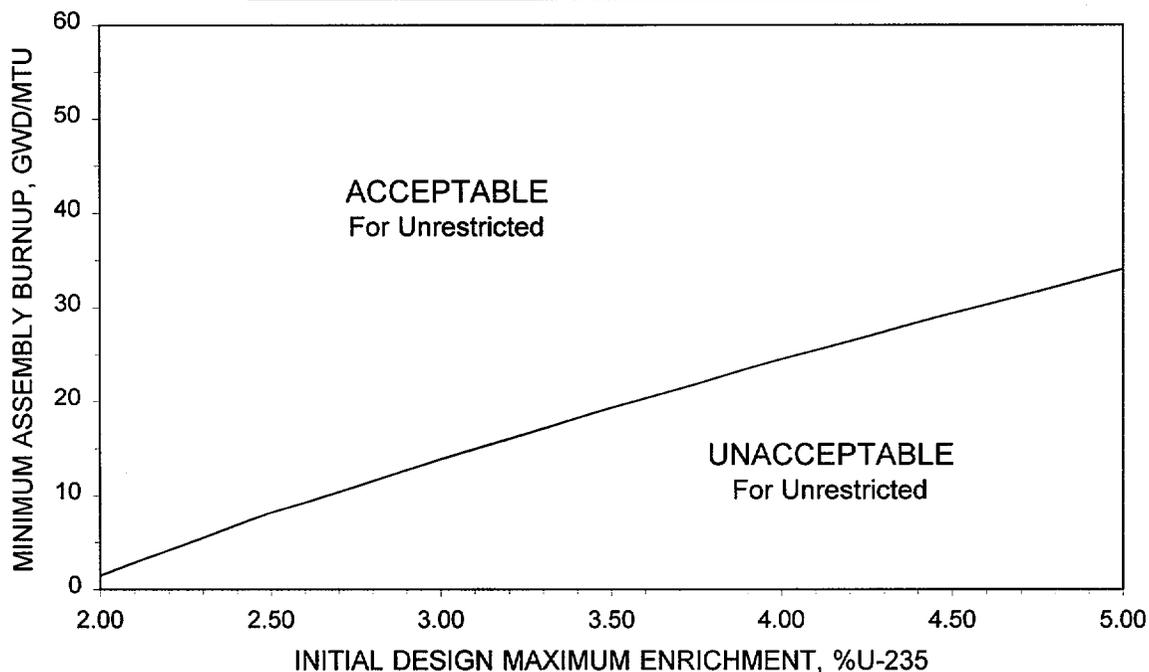
Changes from the original mark-up pages are identified by a vertical line in the right margin.

3.7.13 Fuel Assembly Storage - **INSERTS**

Insert 3 – Page 3.7.13-3 (replaces Figure 3.7.13-1)

Table 3.7.13-1 (page 1 of 1)
 Minimum Qualifying Burnup versus Design Maximum Enrichment
 for Unrestricted Storage in the Units 1 and 2 Spent Fuel Pool

Initial Design Maximum Enrichment (Weight% U-235)	Minimum Assembly Burnup (GWD/MTU)
1.91 (or less)	0
2.00	1.43
2.50	8.08
3.00	13.85
3.50	19.30
4.00	24.47
4.50	29.35
5.00	34.07



NOTES:

The Design Maximum enrichment indicated above is the nominal maximum enrichment of any fuel pin in the fuel assembly being considered. The as-built enrichment of a fuel assembly may exceed its specified Design Maximum by up to 0.05 wt % U-235 and still be stored in accordance with the above burnup limits for that Design Maximum enrichment.

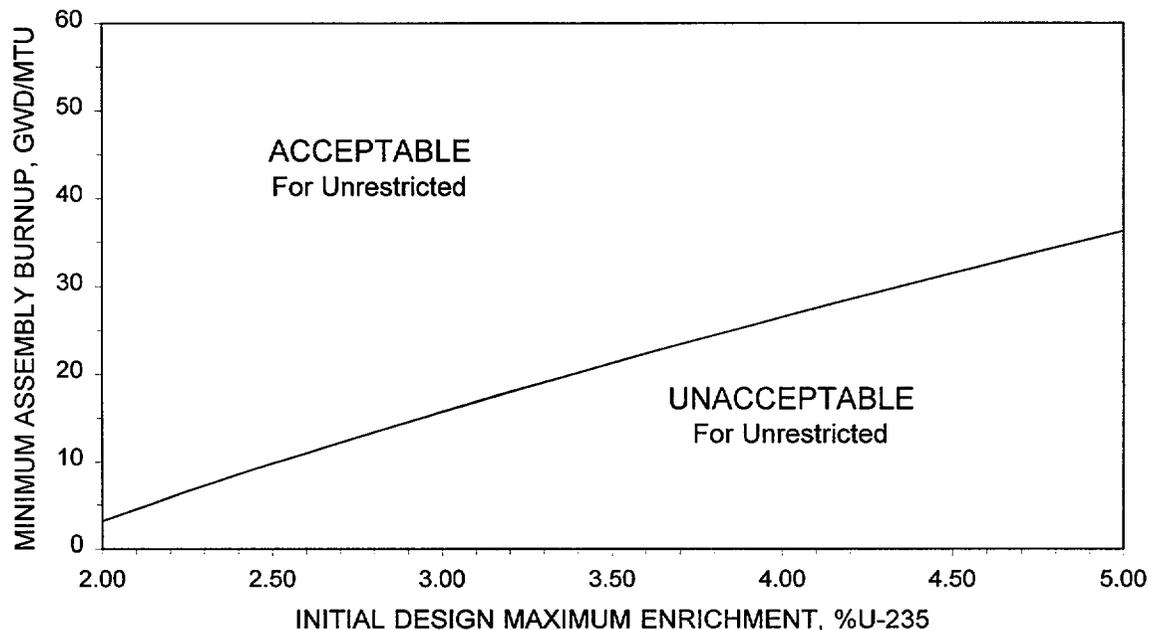
Fuel which differs from those designs used to determine the requirements of Table 3.7.13-1 may be qualified for Unrestricted storage by means of an analysis using NRC approved methodology to assure that k_{eff} is less than 1.0 with no boron and less than or equal to 0.95 with credit for soluble boron.

3.7.13 Fuel Assembly Storage - INSERTS

Insert 6 – Page 3.7.13-6 (replaces Figure 3.7.13-4)

Table 3.7.13-4 (page 1 of 1)
Minimum Qualifying Burnup versus Design Maximum Enrichment
for Unrestricted Storage in the Unit 3 Spent Fuel Pool

Initial Design Maximum Enrichment (Weight% U-235)	Minimum Assembly Burnup (GWD/MTU)
1.81 (or less)	0
2.00	3.16
2.50	9.79
3.00	15.72
3.50	21.30
4.00	26.54
4.50	31.50
5.00	36.30



NOTES:

The Design Maximum enrichment indicated above is the nominal maximum enrichment of any fuel pin in the fuel assembly being considered. The as-built enrichment of a fuel assembly may exceed its specified Design Maximum by up to 0.05 wt % U-235 and still be stored in accordance with the above burnup limits for that Design Maximum enrichment.

Fuel which differs from those designs used to determine the requirements of Table 3.7.13-4 may be qualified for Unrestricted storage by means of an analysis using NRC approved methodology to assure that k_{eff} is less than 1.0 with no boron and less than or equal to 0.95 with credit for soluble boron.

U. S. Nuclear Regulatory Commission
February 15, 2001

ENCLOSURE 2

REPLACEMENT TS PAGES

**FOR ATTACHMENT 2 TO DECEMBER 28, 2000
LICENSE AMENDMENT APPLICATION**

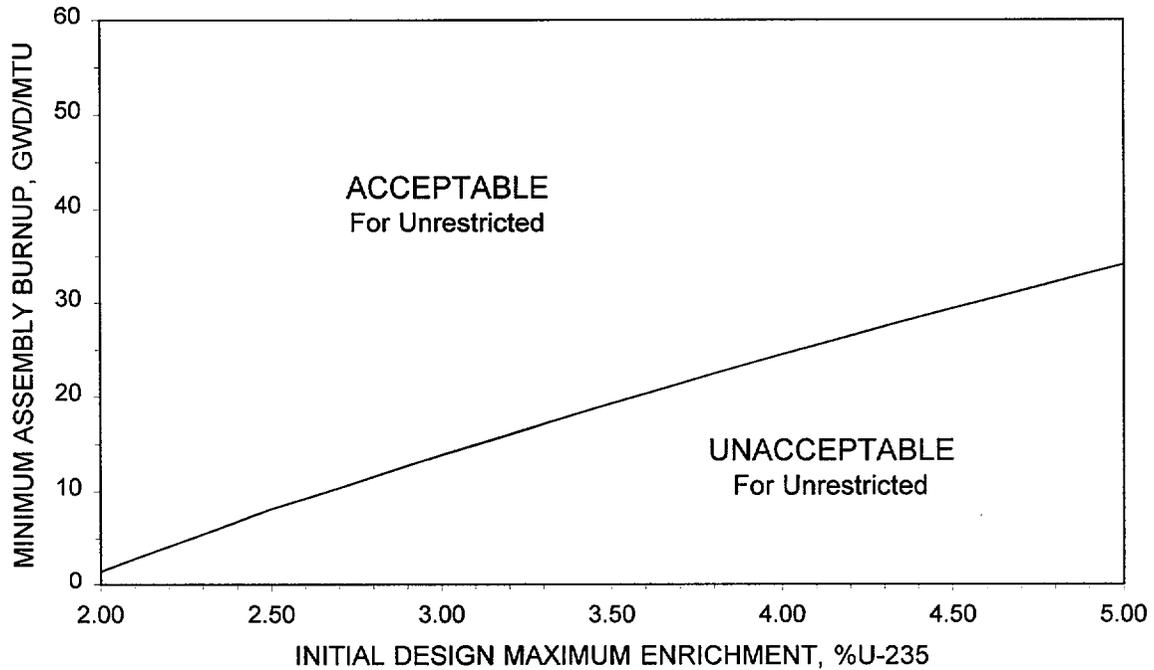
Replace the following listed Attachment 2 TS pages with the enclosed replacement TS pages numbered:

3.7.13-3

3.7.13-6

Table 3.7.13-1 (page 1 of 1)
Minimum Qualifying Burnup versus Design Maximum Enrichment
for Unrestricted Storage in the Units 1 and 2 Spent Fuel Pool

Initial Design Maximum Enrichment (Weight% U-235)	Minimum Assembly Burnup (GWD/MTU)
1.91 (or less)	0
2.00	1.43
2.50	8.08
3.00	13.85
3.50	19.30
4.00	24.47
4.50	29.35
5.00	34.07



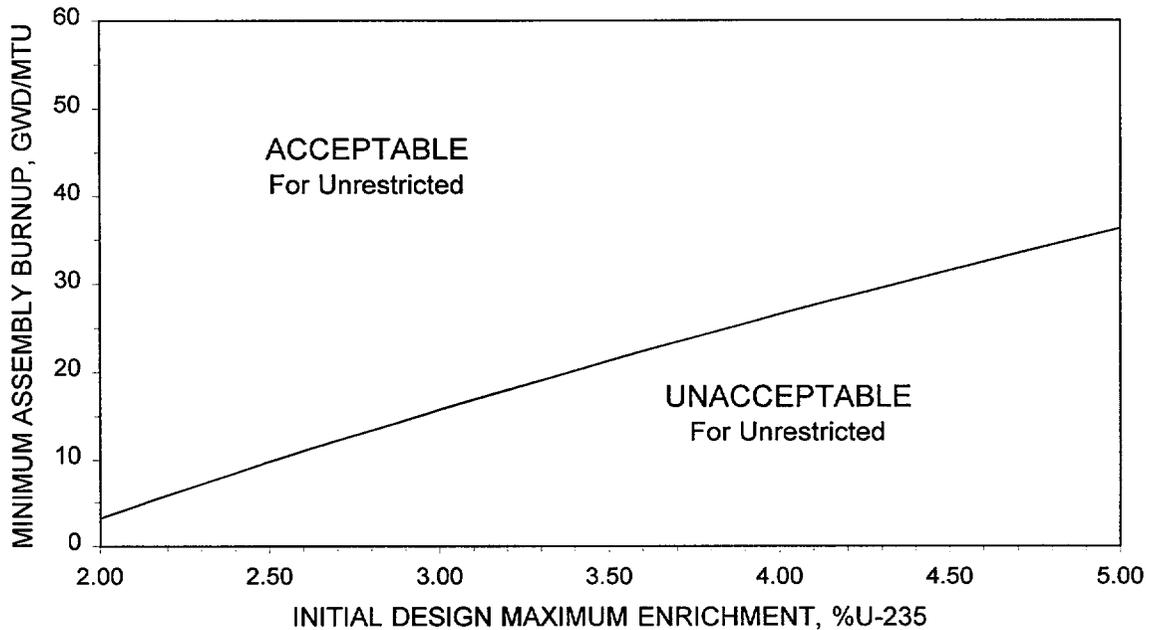
NOTES:

The Design Maximum enrichment indicated above is the nominal maximum enrichment of any fuel pin in the fuel assembly being considered. The as-built enrichment of a fuel assembly may exceed its specified Design Maximum by up to 0.05 wt % U-235 and still be stored in accordance with the above burnup limits for that Design Maximum enrichment.

Fuel which differs from those designs used to determine the requirements of Table 3.7.13-1 may be qualified for Unrestricted storage by means of an analysis using NRC approved methodology to assure that k_{eff} is less than 1.0 with no boron and less than or equal to 0.95 with credit for soluble boron.

Table 3.7.13-4 (page 1 of 1)
Minimum Qualifying Burnup versus Design Maximum Enrichment
for Unrestricted Storage in the Unit 3 Spent Fuel Pool

Initial Design Maximum Enrichment (Weight% U-235)	Minimum Assembly Burnup (GWD/MTU)
1.81 (or less)	0
2.00	3.16
2.50	9.79
3.00	15.72
3.50	21.30
4.00	26.54
4.50	31.50
5.00	36.30



NOTES:

The Design Maximum enrichment indicated above is the nominal maximum enrichment of any fuel pin in the fuel assembly being considered. The as-built enrichment of a fuel assembly may exceed its specified Design Maximum by up to 0.05 wt % U-235 and still be stored in accordance with the above burnup limits for that Design Maximum enrichment.

Fuel which differs from those designs used to determine the requirements of Table 3.7.13-4 may be qualified for Unrestricted storage by means of an analysis using NRC approved methodology to assure that k_{eff} is less than 1.0 with no boron and less than or equal to 0.95 with credit for soluble boron.

ENCLOSURE 3
REPLACEMENT PAGES
FOR ATTACHMENT 6 TO DECEMBER 28, 2000
LICENSE AMENDMENT APPLICATION

Replace the following listed Attachment 6 pages with the enclosed replacement pages:

Page No.

27
33
34

NOTE:

1. These replacement pages are dated February 15, 2001 to distinguish those of the original December 29, 2000 submittal.
2. Changes are identified on the replacement pages by a vertical line in the right margin.

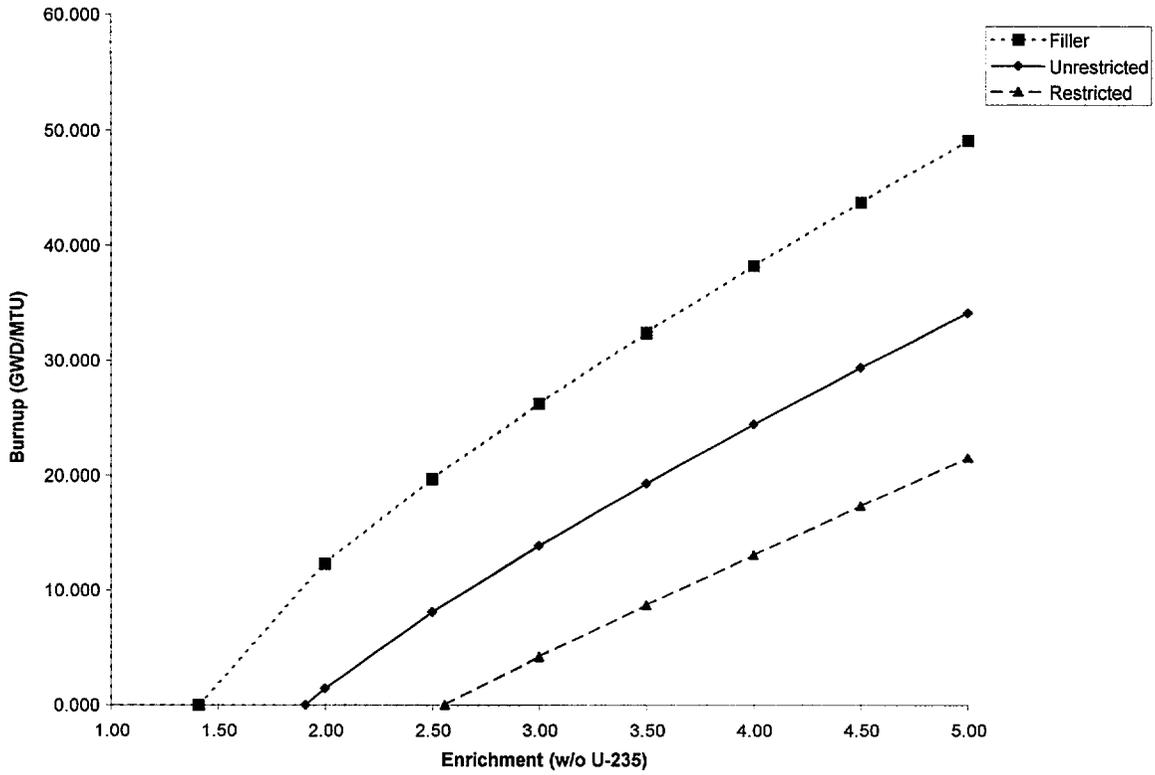
Table 7 - Minimum Qualifying Burnup versus Initial Design Maximum Enrichment For Unrestricted Storage

Oconee 1/2 Pool		Oconee 3 Pool	
Initial		Initial	
Design Maximum	Minimum	Design Maximum	Minimum
Enrichment	Burnup	Enrichment	Burnup
(wt % U-235)	(GWD/MTU)	(wt % U-235)	(GWD/MTU)
1.91	0.00	1.81	0.00
2.00	1.43	2.00	3.16
2.50	8.08	2.50	9.79
3.00	13.85	3.00	15.72
3.50	19.30	3.50	21.30
4.00	24.47	4.00	26.54
4.50	29.35	4.50	31.50
5.00	34.07	5.00	36.30

Table 8 - Minimum Qualifying Burnup versus Initial Design Maximum Enrichment For Restricted Storage

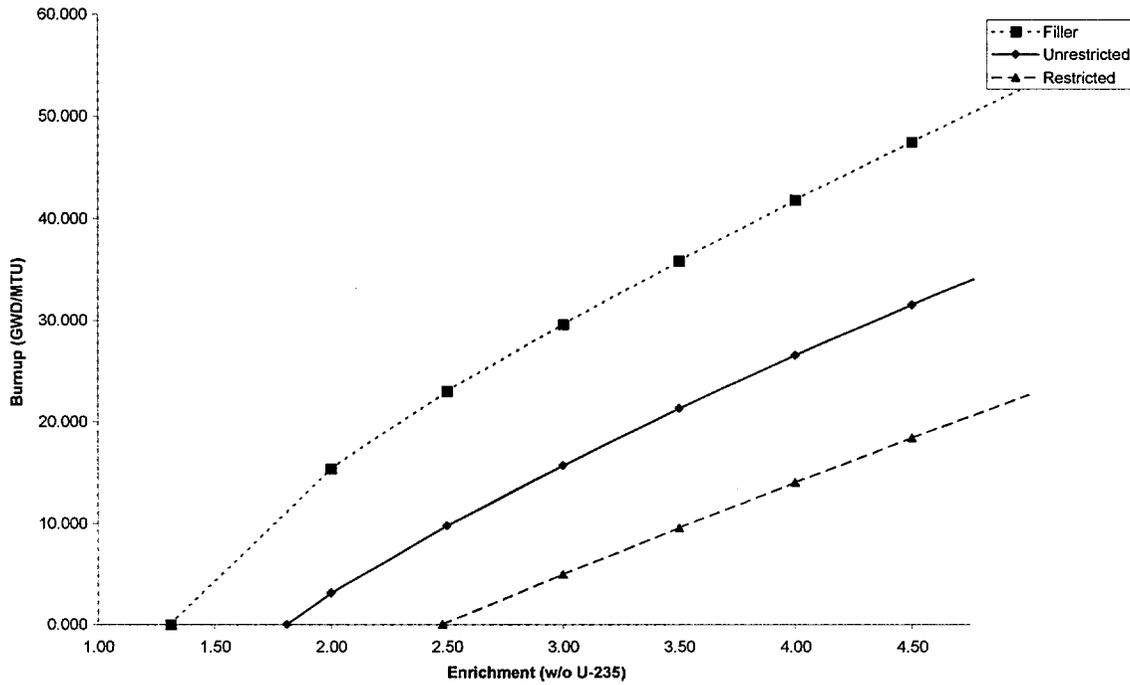
Oconee 1/2 Pool		Oconee 3 Pool	
Initial		Initial	
Design Maximum	Minimum	Design Maximum	Minimum
Enrichment	Burnup	Enrichment	Burnup
(wt % U-235)	(GWD/MTU)	(wt % U-235)	(GWD/MTU)
2.56	0.00	2.48	0.00
3.00	4.19	3.00	5.00
3.50	8.68	3.50	9.59
4.00	13.02	4.00	14.01
4.50	17.31	4.50	18.38
5.00	21.53	5.00	22.60

Figure 5 - Oconee Units 1 and 2 Pool, Burnup Versus Enrichment Limits



Note: Plotted from data provided on Tables 7, 8 and 9.

Figure 6 - Oconee Unit 3 Pool, Burnup versus Enrichment Limits



Note: Plotted from data provided on Tables 7, 8 and 9.