

June 23, 1998

Mr. Charles H. Cruse  
Vice President - Nuclear Energy  
Baltimore Gas and Electric Company  
Calvert Cliffs Nuclear Power Company  
1650 Calvert Cliffs Parkway  
Lusby, MD 20657-4702

SUBJECT: MAY 4, 1998, ISSUANCE OF IMPROVED TECHNICAL SPECIFICATIONS FOR  
CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2  
(TAC NOS. M97363 AND M97364)

Dear Mr. Cruse:

On May 4, 1998, the Commission issued Amendment No. 227 to Facility Operating License No. DPR-53 and Amendment No. 201 to Facility Operating License No. DPR-69 for the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments consisted of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated December 4, 1996, as supplemented by letters dated March 27, June 9, June 18, July 21, August 14, August 19, September 10, October 6, October 20, October 23, November 5, 1997, and January 12, January 28, and March 16, 1998.

The mail out of these amendments, inadvertently failed to include the following pages: Page 10 through Page 13, Section 3.4, "Reactor Coolant System," or the Safety Evaluation (SE), Table L, Matrix of Less Restrictive Changes, and in the Bases, page B 3.7.6-5. These pages are enclosed. If you have any questions on the SE or tables, please contact me on (301) 415-3473 or Mary Lynn Reardon (301) 415-1177. Thank you for your cooperation in these efforts.

Sincerely,

Original Signed by:  
Alexander W. Dromerick, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-317  
and 50-318

Enclosures: As stated

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in cursive script, reading "Alexander W. Dromerick".

Alexander W. Dromerick, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

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Mr. Charles H. Cruse  
Baltimore Gas & Electric Company

Calvert Cliffs Nuclear Power Plant

cc:

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King of Prussia, PA 19406

ENCLOSURE

**TABLE L - MATRIX OF LESS RESTRICTIVE CHANGES  
SECTION 3.4, "REACTOR COOLANT SYSTEM"**

Discussion of Change	Description	ITS Section	CTS Section	Category
3.4.14 L2	CTS 3.4.6.1 Action b requires the inoperable RCS leakage detection instrument to be restored to Operable status within 7 days when only one RCS leakage detection instrument remains Operable. ITS 3.4.14 increases the allowed outage time from 7 days to 30 days.	3.4.14	3.4.6.1	III
3.4.14 L3	In the event only one of the required RCS leakage detection systems is operable, Action b of the CTS 3.4.6.1 permits the plant to continue to operate provided grab samples of the containment atmosphere are obtained and analyzed periodically, and an RCS water inventory balance is performed in accordance with CTS SR 4.4.6.2.c at an increased frequency. In the event the Containment Sump level Alarm System is inoperable (i.e., only the Containment Atmosphere Radioactivity Monitoring System is operable), Action A of ITS 3.4.14 permits continued operation of the plant provided an RCS water inventory balance is performed in accordance with ITS SR 3.4.13.1 at an increased frequency.	3.4.14	3.4.6.1	IV

**Categories**

- I. Relaxation of Applicability
- II. Relaxation of Surveillance Frequency
- III. Relaxation of Allowed Outage Time
- IV. Relaxation of Required Actions
- V. Relaxation of Surveillance Requirement Acceptance Criteria
- VI. Deletion of Requirement for 30-day Special Report to NRC
- VII. Relaxation of LCO
- VIII. Deletion of SR

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**TABLE L - MATRIX OF LESS RESTRICTIVE CHANGES  
SECTION 3.4, "REACTOR COOLANT SYSTEM"**

Discussion of Change	Description	ITS Section	CTS Section	Category
3.4.14 L4	CTS 3.4.6.1 Action b requires grab samples of the containment atmosphere to be analyzed once per 12 hours when used as contingency actions when only one RCS leakage detection system remains Operable. ITS 3.4.14 decreases the Frequency for the performance of this action to once every 24 hours.	3.4.14	3.4.6.1	III
3.4.14 L5	ITS 3.4.14 adds an LCO 3.0.4 exemption to CTS 3.4.6.1 which allows the changing of Modes within the Modes of Applicability while RCS leakage detection instrumentation is inoperable.	3.4.14	3.4.6.1	VII
<b>3.4.15, RCS SPECIFIC ACTIVITY</b>				
3.4.15 L1	CTS 3.4.8 (RCS Specific Activity) Modes of Applicability are Modes 1, 2, 3, 4, and 5. ITS 3.4.15 Modes of Applicability are Modes 1 and 2 and Mode 3 with $T_{avg} \geq 500$ F.	3.4.15	3.4.8	I
3.4.15 L2	CTS 3.4.8 SR Table 4.4-4 Item 1 requires a gross activity determination at least once per 72 hours. ITS 3.4.15 SR 3.4.15.1 will decrease the Frequency from 72 hours to 7 days.	3.4.15	3.4.8	II

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- VII. Relaxation of LCO
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**TABLE L - MATRIX OF LESS RESTRICTIVE CHANGES  
SECTION 3.4, "REACTOR COOLANT SYSTEM"**

Discussion of Change	Description	ITS Section	CTS Section	Category
3.4.15 L3	CTS 3.4.8 SR Table 4.4-4 Item 4.b requires the Dose Equivalent I-131 specific activity be verified in Modes 1, 2, and 3. ITS 3.4.15 SR 3.4.15.2 adds a Note which only requires the SR to be performed in Mode 1.	3.4.15	3.4.8	I
3.4.15 L4	CTS 3.4.8 SR Table 4.4-4 Item 3 requires E-Bar to be determined from a sample taken in Mode 1 after a minimum of 2 Effective Fuel Power Days and 20 days of Mode 1 operation have elapsed since the reactor was last subcritical for $\geq 48$ hours. ITS 3.4.15 SR 3.4.15.3 allows the Surveillance to be delayed for 31 days after a minimum of 2 Effective Full Power Days and 20 days of Mode 1 operation have elapsed since the reactor was last subcritical for $\geq 48$ hours.	3.4.15	3.4.8	II

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- I. Relaxation of Applicability
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- V. Relaxation of Surveillance Requirement Acceptance Criteria
- VI. Deletion of Requirement for 30-day Special Report to NRC
- VII. Relaxation of LCO
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**TABLE L - MATRIX OF LESS RESTRICTIVE CHANGES  
SECTION 3.4, "REACTOR COOLANT SYSTEM"**

Discussion of Change	Description	ITS Section	CTS Section	Category
3.4.15 L5	CTS 3.4.8 Action a contains a clause stating that plant operation may continue for up to 100 hours when specific activity of the primary coolant is > 1.0 uCi/gram DOSE EQUIVALENT I-131 but within the allowable limit shown on Figure 3.4.8-1, provided that operation under these circumstances shall not exceed 10 percent of the units total yearly operating time. ITS 3.4.15 will not contain this clause regarding 10 percent of the units total yearly operating time.	3.4.15	3.4.8	VII

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- I. Relaxation of Applicability
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under the conditions that apply during a unit outage and the potential for an unplanned transient if the Surveillance were performed with the reactor at power. Operating experience has shown that these components usually pass the Surveillance when performed at the 24 month Frequency. Therefore, the Frequency is acceptable from a reliability standpoint.

SR 3.7.6.3

The SR verifies proper automatic operation of the SRW system pumps on an actual or simulated actuation signal (SIAS or CSAS). The SRW system is a normally operating system that cannot be fully actuated as part of the normal testing during normal operation. Operating experience has shown that these components usually pass the Surveillance when performed at the 24 month Frequency. Therefore, the Frequency is acceptable from a reliability standpoint.

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REFERENCES

1. UFSAR, Section 9.5.2.2
  2. UFSAR, Section 14.20
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