



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

May 20, 1994

Docket Nos. 50-317  
and 50-318

Mr. Robert E. Denton  
Vice President - Nuclear Energy  
Baltimore Gas and Electric Company  
Calvert Cliffs Nuclear Power Plant  
1650 Calvert Cliffs Parkway  
Lusby, Maryland 20657-4702

Dear Mr. Denton:

SUBJECT: ISSUANCE OF AMENDMENTS FOR CALVERT CLIFFS NUCLEAR POWER PLANT,  
UNIT NO. 1 (TAC NO. M86140) AND UNIT NO. 2 (TAC NO. M86141)

The Commission has issued the enclosed Amendment No. 189 to Facility Operating License No. DPR-53 and Amendment No. 166 to Facility Operating License No. DPR-69 for the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Operating Licenses in response to your application transmitted by letter dated April 1, 1993.

The amendments make administrative, editorial, and format changes to the Operating Licenses. These changes include the deletion or incorporation, as appropriate, of all handwritten or "pasted-up" changes and the removal of all previous license conditions that have been completed to the satisfaction of the Commission. The changes and reformatting result in the Operating Licenses containing only those license conditions that are currently applicable.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Daniel G. McDonald, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

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Enclosures:

1. Amendment No. 189 to DPR-53
2. Amendment No. 166 to DPR-69
3. Safety Evaluation

cc w/enclosures:  
See next page

NRC FILE NUMBER COPY

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1/1

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Mr. Robert E. Denton  
Baltimore Gas & Electric Company

Calvert Cliffs Nuclear Power Plant  
Unit Nos. 1 and 2

cc:

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Regional Administrator, Region I  
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King of Prussia, Pennsylvania 19406

DATED: May 20, 1994

AMENDMENT NO. 189 TO FACILITY OPERATING LICENSE NO. DPR-53-CALVERT CLIFFS  
UNIT 1  
AMENDMENT NO. 166 TO FACILITY OPERATING LICENSE NO. DPR-69-CALVERT CLIFFS  
UNIT 2

Docket File

NRC & Local PDRs

PDI-1 Reading

S. Varga, 14/E/4

J. Calvo, 14/A/4

M. Boyle

C. Vogan

J. Menning

D. McDonald

OGC

D. Hagan, 3206 MNBB

C. Liang, 8/E/23

G. Hill (4), T5C10

C. Grimes, 11/F/23

ACRS (10)

OPA

OC/LFDCB

PD plant-specific file

C. Cowgill, Region I

cc: Plant Service list



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 189  
License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Baltimore Gas and Electric Company (the licensee) dated April 1, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, paragraph 2.D and the list of Incomplete Construction and Preoperational Test Items that are Required to be Completed of Facility Operating License No. DPR-53 are hereby deleted. In addition, paragraph 2.C.(3) is amended to read as follows:

- (3) The licensee is required to implement and maintain the administrative controls identified in Section 6 of the NRC's Fire Protection Safety Evaluation on the facility dated September 14, 1979.
3. The license is also amended by several editorial and format changes as follows:
- A. The handwritten references to previous License Amendments at paragraphs 2.A., 2.B.(3), 2.C.(3), 2.C.(4), 2.C.(5), and 2.E. are deleted.
  - B. Pages 3a and 3b are deleted and the remaining license conditions on these pages are transferred to page 4. A new page 5 is added to accommodate the additions to page 4.
  - C. The designation for paragraph 2.C.(4) on page 4 becomes (4).
  - D. The designation for paragraph 2.C.(5) on page 4 becomes (5).
  - E. The designation for paragraph 2.E. on page 5 becomes D.
4. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael L. Boyle, Acting Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Pages 1, 2, 3, 4, and 5  
of License No. DPR-53

Date of Issuance: May 20, 1994



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NO. 2

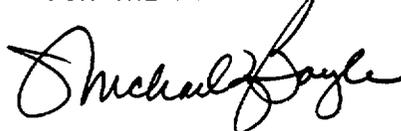
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 166  
License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Baltimore Gas and Electric Company (the licensee) dated April 1, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, paragraphs 2.C.2.a, 2.C.3, 2.C.4, 2.E., and 2.G. of Facility Operating License No. DPR-69 are hereby deleted. In addition, paragraph 2.C.6 is redesignated as paragraph 2.C.4 and amended to read as follows:

4. The licensee is required to implement and maintain the administrative controls identified in Section 6 of the NRC's Fire Protection Safety Evaluation on the facility dated September 14, 1979.
3. The license is also amended by several editorial and format changes as follows:
  - A. The handwritten references to previous License Amendments at paragraphs 2.A., 2.C.1., 2.C.5, 2.C.7, 2.D., 2.F. and 2.H. are deleted.
  - B. Previously deleted paragraphs 2.C.3 and 2.C.4 are removed.
  - C. Pages 6, 6a, 7, and 7a are deleted and the remaining license conditions on these pages are transferred to pages 4 and 5.
  - D. The designation for paragraph 2.C.5 on page 4 becomes 3.
  - E. The designation for paragraph 2.C.7 on page 4 becomes 5.
  - F. The designation for paragraph 2.D. on page 5 becomes D.
  - G. The designation for paragraph 2.F. on page 5 becomes E.
  - H. The designation of paragraph 2.H. on page 5 becomes F.
4. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael L. Boyle, Acting Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Pages 1, 2, 3, 4, and 5  
of License No. DPR-69

Date of Issuance: May 20, 1994

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 189 FACILITY OPERATING LICENSE NO. DPR-53

AMENDMENT NO. 166 FACILITY OPERATING LICENSE NO. DPR-69

DOCKET NOS. 50-317 AND 50-318

Revise License DPR-53 as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
1	1
2	2
3	3
3a	-
3b	-
4	4
List of Incomplete Construction and Preoperational Test Items that are Required to be Completed	-
5	5

Revise License DPR-69 as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
1	1
2	2
3	3
4	4
5	5
6	-
6a	-
7	-
7a	-

BALTIMORE GAS & ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT 1

FACILITY OPERATING LICENSE

License No. DPR-53

1. The Atomic Energy Commission (the Commission) having found that:
  - A. The application for license filed by Baltimore Gas & Electric Company (the licensee) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I and all required notifications to other agencies or bodies have been duly made;
  - B. Construction of the Calvert Cliffs Nuclear Power Plant, Unit 1, (facility) has been substantially completed in conformity with Construction Permit No. CPPR-63 and the application, as amended, the provisions of the Act and the rules and regulations of the Commission;
  - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - D. There is reasonable assurance: (i) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;
  - E. The licensee is technically and financially qualified to engage in the activities authorized by this operating license in accordance with the rules and regulations of the Commission;
  - F. The licensee has satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission's regulations;

- G. The issuance of this operating license will not be inimical to the common defense and security or to the health and safety of the public;
  - H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental costs and considering available alternatives, the issuance of Facility Operating License No. DPR-53 (subject to the conditions for protection of the environment set forth herein) is in accordance with 10 CFR Part 50, Appendix D, of the Commission's regulations and all applicable requirements of said Appendix D have been satisfied; and
  - I. The receipt, possession, and use of source, byproduct, and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40, and 70, including 10 CFR 30.33, 40.32, and 70.23, and 70.31.
2. Facility Operating License No. DPR-53 is hereby issued to the Baltimore Gas & Electric Company to read as follows:
- A. This license applies to the Calvert Cliffs Nuclear Plant Unit 1, a pressurized water reactor and associated equipment (the facility), owned by Baltimore Gas & Electric Company. The facility is located in Calvert County, Maryland and is described in the "Final Safety Analysis Report" as supplemented and amended (Amendments 11 through 47), and the Environmental Report as supplemented and amended (Amendment Nos. 1 and 2).
  - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses Baltimore Gas & Electric Company:
    - 1. Pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess, use, and operate the facility at the designated location in Calvert County, Maryland, in accordance with the procedures and limitations set forth in this license;
    - 2. Pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess, and use at any time any byproduct, source and special nuclear material as reactor fuel, sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required for reactor operation;

3. Pursuant to the Act and 10 CFR Part 30 to receive, possess, and use at any time 100 millicuries each of any byproduct material without restriction to chemical or physical form, for sample analysis or instrument calibration; 500 millicuries of byproduct material in the form of equipment that is radioactively contaminated by radioisotopes with atomic numbers within the range 3 to 83, for inspection and maintenance of the facility, as described in the licensee's letter-application for license amendment dated November 13, 1974, as amended by letter dated November 15, 1974, and Sodium-24, in liquid form, not to exceed 500 millicuries for tracer measurements for steam turbine acceptance testing.
4. Pursuant to the Act and 10 CFR Parts 40 and 70 to receive, possess, and use at any time 100 milligrams each of any source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration;
5. Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

1. Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 2700 megawatts (thermal).

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. \_\_\_\_\_, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The licensee is required to implement and maintain the administrative controls identified in Section 6 of the NRC's Fire Protection Safety Evaluation on the facility dated September 14, 1979.

4. Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Calvert Cliffs Nuclear Power Plant Physical Security Plan," with revisions submitted through February 17, 1988; "Calvert Cliffs Nuclear Power Plant Guard Training and Qualification Plan," with revisions submitted through November 1, 1985; and "Calvert Cliffs Nuclear Power Plant Safeguards Contingency Plan," with revisions submitted February 9, 1988. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

5. Secondary Water Chemistry Monitoring Program

The licensee shall implement a secondary water chemistry monitoring program to inhibit steam generator tube degradation. This program shall include:

- a. Identification of a sampling schedule for the critical parameters and control points for these parameters.
- b. Identification of the procedures used to quantify parameters that are critical to control points.
- c. Identification of process sampling points.
- d. Procedure for recording and management of data.
- e. Procedures defining corrective actions for off control point chemistry conditions; and
- f. A procedure identifying the authority responsible for the interpretation of the data and the sequence and timing of administrative events required to initiate corrective action.

D. This license is effective as of the date of issuance and shall expire at midnight July 31, 2014.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed by  
Roger S. Boyd

A. Giambusso, Deputy Director  
for Reactor Projects  
Directorate of Licensing

Attachment:

Appendices A & B -  
Technical Specifications

Date of Issuance: July 31, 1974

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT 2

FACILITY OPERATING LICENSE

License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) having found that:
  - A. The application for license and the application for license amendment, dated October 25, 1976, filed by Baltimore Gas and Electric Company (the licensee) comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I and all required notifications to other agencies or bodies have been duly made;
  - B. Construction of the Calvert Cliffs Nuclear Power Plant, Unit 2 (the facility) has been substantially completed in conformity with Provisional Construction Permit No. CPPR-64 and the application, as amended, the provisions of the Act and the rules and regulations of the Commission;
  - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - D. There is reasonable assurance: (i) that the activities authorized by this amended operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;
  - E. The licensee is technically and financially qualified to engage in the activities authorized by this amended operating license in accordance with the rules and regulations of the Commission;
  - F. The licensee has satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission's regulations;

- G. The issuance of this amended operating license will not be inimical to the common defense and security or to the health and safety of the public;
  - H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs and considering available alternatives, the issuance of Amendment No. 2 to Facility Operating License No. DPR-69 subject to the conditions for protection of the environment set forth herein is in accordance with 10 CFR Part 51 (and with former Appendix D to 10 CFR Part 50) of the Commission's regulations and all applicable requirements have been satisfied; and
  - I. The receipt, possession, and use of source, byproduct, and special nuclear material as authorized by this amended license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40, and 70, including 10 CFR Sections 30.33, 40.32, 70.23, and 70.31.
2. Facility Operating License No. DPR-69, issued to the Baltimore Gas and Electric Company, is hereby amended in its entirety to read as follows:
- A. This license applies to the Calvert Cliffs Nuclear Plant Unit 2, a pressurized water nuclear reactor and associated equipment (the facility), owned by the Baltimore Gas & Electric Company. The facility is located in Calvert County, Maryland, on the western shore of the Chesapeake Bay, about 10 and 1/2 miles southeast of Prince Frederick, Maryland, and is described in the "Final Safety Analysis Report" as supplemented and amended (Amendments 11 through 53), and the Environmental Report as supplemented and amended (Amendment Nos. 1 and 2).
  - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses Baltimore Gas and Electric Company:
    - 1. Pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess, use, and operate the facility at the designated location in Calvert County, Maryland in accordance with the procedures and limitations set forth in this amended license;
    - 2. Pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;

3. Pursuant to the Act and 10 CFR Parts 30, 40, and 70 to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
  4. Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
  5. Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

1. Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 2700 megawatts (thermal).

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. \_\_\_\_\_, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. Less Than Four Pump Operation

The licensee shall not operate the reactor at power levels in excess of five (5) percent of rated thermal power with less than four (4) reactor coolant pumps in operation. This condition shall remain in effect until the licensee has submitted safety analyses for less than four pump operation, and approval for such operation has been granted by the Commission by amendment of this license.

4. The licensee is required to implement and maintain the administrative controls identified in Section 6 of the NRC's Fire Protection Safety Evaluation on the facility dated September 14, 1979.

5. Secondary Water Chemistry Monitoring Program

The licensee shall implement a secondary water chemistry monitoring program to inhibit steam generator tube degradation. This program shall include:

- a. Identification of a sampling schedule for the critical parameters and control points for these parameters.
- b. Identification of the procedures used to quantify parameters that are critical to control points.
- c. Identification of process sampling points.
- d. Procedure for recording and management of data.
- e. Procedures defining corrective actions for off control point chemistry conditions; and
- f. A procedure identifying the authority responsible for the interpretation of the data and the sequence and timing of administrative events required to initiate corrective action.

D. Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Calvert Cliffs Nuclear Power Plant Physical Security Plan," with revisions submitted through February 17, 1988; "Calvert Cliffs Nuclear Power Plant Guard Training and Qualification Plan," with revisions submitted through November 1, 1985; and "Calvert Cliffs Nuclear Power Plant Safeguards Contingency Plan," with revisions submitted February 9, 1988. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

- E. If harmful effects or evidence of irreversible damage are detected by the biological monitoring program, hydrological monitoring program, and the radiological monitoring program specified in the Appendix B Technical Specifications, the licensee will provide to the staff a detailed analysis of the problem and a program of remedial action to be taken promptly to eliminate or significantly reduce the detrimental effects or damage.
- F. This license is effective as of the date of issuance and shall expire at midnight August 13, 2016.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by  
Roger S. Boyd, Director  
Division of Project Management  
Office of Nuclear Reactor Regulation

Attachment:

Changes to Technical Specifications,  
Appendix B

Date of Issuance: November 30, 1976



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 189 TO FACILITY OPERATING LICENSE NO. DPR-53  
AND AMENDMENT NO. 166 TO FACILITY OPERATING LICENSE NO. DPR-69  
BALTIMORE GAS AND ELECTRIC COMPANY  
CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2  
DOCKET NOS. 50-317 AND 50-318

1.0 INTRODUCTION

By letter dated April 1, 1993, the Baltimore Gas and Electric Company (BG&E or the licensee) submitted a request for changes to Facility Operating License No. DPR-53 and Facility Operating License No. DPR-69 for the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The requested changes would make administrative, editorial, and format changes to the Operating Licenses. These changes include the deletion or incorporation, as appropriate, of all handwritten or "pasted-up" changes and the removal of all previous license conditions that have been completed to the satisfaction of the Commission. The changes and reformatting would result in the Operating Licenses containing only those license conditions that are currently applicable.

2.0 BACKGROUND

Facility Operating License No. DPR-53 (Unit 1) and Facility Operating License No. DPR-69 (Unit 2) were issued to BG&E on July 31, 1974, and August 13, 1976, respectively. Subsequently, the Unit 1 Operating License has been substantively amended at least 14 times and the Unit 2 Operating License has been substantively amended at least 12 times. All changes made to the Unit 1 Operating License have been either handwritten or cut and pasted into the license. The Unit 2 Operating License was reissued in total by Amendment No. 2 and all ensuing changes to the Unit 2 Operating License have been handwritten or cut and pasted into the license. At present, both licenses contain faded text and handwritten changes. As a result, some portions of the licenses are almost illegible.

The licenses also contain outdated license conditions. The outdated conditions for the most part deal with incomplete construction items, preoperational tests, and startup tests. Other outdated conditions involve additional surveillances, the outcome of legal proceedings, the studies associated with the selection of an optimum cooling tower system, and a one-time exemption from compliance with the charcoal testing Technical Specifications (TSs).

### 3.0 EVALUATION

BG&E noted in its submittal that a substantial number of changes were being requested which include the deletion of 19 items from the Unit 1 license (DPR-53) and 6 items from the Unit 2 license (DPR-69). The licensee further requested that the licenses be retyped in their entirety. As previously noted, both licenses contain handwritten or cut and pasted changes and faded text. It was noted that the Unit 2 license was retyped when Amendment No. 2 was issued due to the substantial number of changes at that time.

#### 3.1 Changes to Facility Operating License No. DPR-53 (Unit 1)

The licensee has proposed that the following outdated license conditions be deleted from the license:

##### Paragraph 2.C.(3)

Paragraph 2.C.(3) requires the licensee to complete the modifications identified in Paragraph 3.1.1 through 3.1.21 of the NRC's Fire Protection Safety Evaluation Report (FPSER) for Calvert Cliffs, dated September 14, 1979, submit additional information identified in Table 3.2 of the FPSER, and implement the administrative controls identified in Section 6 of the FPSER. A NRC staff safety evaluation dated September 27, 1982, indicates that these license conditions have been completed. The licensee has, therefore, requested that all of paragraph 2.C.(3) be deleted with the exception of the first sentence of the third paragraph. This sentence requires BG&E to implement and maintain the administrative controls identified in Section 6 of the NRC's FPSER dated September 14, 1979.

##### Paragraph 2.D

Paragraph 2.D requires the licensee to select an optimum cooling tower system and complete the preliminary design and engineering work of the selected system prior to exceeding 60 percent capacity, but in no event later than 6 months after the date of commercial operation. This item has been completed and was documented in a letter from BG&E to the NRC dated January 30, 1975.

##### Item A.1.a in List of Incomplete Construction and Preoperational Test Items that are Required to be Completed (List of Items Requiring Completion)

Item A.1.a requires BG&E to complete reactor protective system preoperational testing prior to proceeding beyond fuel loading. This item has been completed. Satisfactory completion was documented in NRC Inspection Report Nos. 50-317/74-33 and 50-317/74-36.

##### Item A.1.b in List of Items Requiring Completion

Item A.1.b requires the licensee to complete control element drive mechanism cooling system preoperational testing prior to proceeding beyond fuel

loading. This item has been completed. Satisfactory completion of this testing was documented in NRC Inspection Report No. 50-317/74-38.

Item A.1.c in List of Items Requiring Completion

Item A.1.c requires the completion of reactor component handling preoperational testing prior to proceeding beyond fuel loading. This item has been completed. Satisfactory completion of this testing was documented in NRC Inspection Report Nos. 50-317/74-33 and 50-317/74-36.

Item A.1.d in List of Items Requiring Completion

Item A.1.d requires the completion of preoperational testing on the main steam isolation valves prior to proceeding beyond fuel loading. This item has been completed. Satisfactory completion of this testing was documented in NRC Inspection Report Nos. 50-317/74-33 and 50-317/74-36.

Item A.2.a in List of Items Requiring Completion

Item A.2.a is a construction license condition requiring the establishment of administrative controls to preclude inadvertent overpressurization of the reactor coolant system during solid system operation prior to proceeding beyond fuel loading. This item has been completed. Satisfactory completion of this license condition was documented in NRC Inspection Report No. 50-317/74-35.

Item A.2.b in List of Items Requiring Completion

Item A.2.b is a license condition requiring the addition of spacers and the replacement of expansion joints in the saltwater cooling system that were found to be elongated beyond their design limits prior to proceeding beyond fuel loading. This item has been completed. Satisfactory completion of this license condition was documented in NRC Inspection Report No. 50-317/74-35.

Item A.2.c in List of Items Requiring Completion

Item A.2.c is a license condition requiring the completion of the hydrogen supply line to the volume control tank prior to proceeding beyond fuel loading. This item has been completed. Satisfactory completion of this license condition was documented in NRC Inspection Report No. 50-317/74-35.

Item B.1.a in List of Items Requiring Completion

Item B.1.a is a license condition requiring completion of preoperational tests of the liquid waste system evaporators prior to proceeding beyond post-core loading hot functional testing. This item has been completed. Satisfactory completion of this license condition was documented in NRC Inspection Report No. 50-317/74-38.

Item B.1.b in List of Items Requiring Completion

Item B.1.b is a license condition requiring the completion of preoperational tests of the radiation monitoring and process radiation systems prior to proceeding beyond the post-core loading hot functional testing. This item has been completed. Satisfactory completion of this testing was documented in NRC Inspection Report Nos. 50-317/74-33 and 50-317/74-38.

Item B.1.c in List of Items Requiring Completion

Item B.1.c is a license condition requiring the completion of preoperational tests of the variable overpower trip system prior to proceeding beyond post-core loading hot functional testing. This item has been completed. Satisfactory completion of this testing was documented in NRC Inspection Report No. 50-317/74-36.

Item B.2.a in List of Items Requiring Completion

Item B.2.a is a license condition requiring the completion of installation and design documentation of safety-related pipe hangers, restraints, and supports prior to proceeding beyond post-core loading hot functional testing. This item has been completed. Satisfactory resolution of this license condition was documented in NRC Inspection Report Nos. 50-317/74-38 and 50-317/74-39.

Item C.1.a in List of Items Requiring Completion

Item C.1.a is a license condition requiring completion of solid waste processing system preoperational testing prior to proceeding beyond low power physics tests. This item has been completed. Satisfactory resolution of this license condition was documented in NRC Inspection Report Nos. 50-317/74-38, 50-317/74-39, and 50-317/75-09.

Item C.1.b in List of Items Requiring Completion

Item C.1.b is a license condition requiring completion of preoperational tests of the hydrogen purge system prior to proceeding beyond the low power physics tests. This item has been completed. Satisfactory completion of this testing was documented in NRC Inspection Report No. 50-317/74-38.

Item C.2.a in List of Items Requiring Completion

Item C.2.a is a license condition requiring the completion of an evaluation of the adequacy of the diesel generator air start system modifications prior to proceeding beyond the low power physics tests. This item has been completed. Satisfactory completion of this license condition was documented in NRC Inspection Report No. 50-317/74-39.

Item C.2.b in List of Items Requiring Completion

Item C.2.b is a license condition requiring the completion of incore instrumentation prior to proceeding beyond the low power physics tests. This item has been completed. Satisfactory completion of this license condition was documented in NRC Inspection Report No. 50-317/74-38.

Item C.2.c in List of Items Requiring Completion

Item C.2.c is a license condition requiring the completion of reactor internals vibration monitoring capability prior to proceeding beyond the low power physics tests. This item has been completed. Satisfactory completion of this license condition was documented in NRC Inspection Report No. 50-317/75-01.

Item C.2.d in List of Items Requiring Completion

Item C.2.d is a license condition requiring the completion of the modification of the primary control element assembly position indication system prior to proceeding beyond low power physics tests. This item has been completed. Satisfactory completion of this license condition was documented in NRC Inspection Report No. 50-317/74-38.

BG&E has also requested that Facility Operating License No. DPR-53 be retyped in its entirety and that certain editorial and format changes be made to the license as discussed below.

The licensee has proposed that the handwritten reference to Amendment No. 19 at paragraph 2.A. be deleted. The Amendment Number (19) would be included at the bottom of the page with a deletion mark through it indicating that a previous change had been made to this page. BG&E has proposed that handwritten references to previous License Amendments be similarly removed at paragraphs 2.B.(3), 2.C.(3), 2.C.(4), 2.C.(5), and 2.E.

With the previously discussed deletions of paragraphs, pages 3a and 3b of the license would be deleted and the remaining effective license conditions on these pages would be transferred to page 4. A new page 5 would be added to accommodate the additions to page 4.

The licensee has proposed that the designations for certain paragraphs be changed for consistency. Specifically, the designation for paragraph 2.C.(4) on page 4 would become (4). The designation for paragraph 2.C.(5) on page 4 would become (5). The designation for paragraph 2.E. on page 5 would become D.

The NRC staff has concluded that the proposed deletions from Facility Operating License No. DPR-53 described above are acceptable since they represent requirements that have been satisfactorily completed. The staff has also concluded that the retyping of the license and the proposed editorial and format changes are acceptable since they are administrative in nature.

### 3.2 Changes to Facility Operating License No. DPR-69 (Unit 2)

BG&E has proposed that the following outdated license conditions be deleted from the license:

#### Paragraph 2.C.2.a

Paragraph 2.C.2.a exempted the licensee from compliance with certain TSs relating to charcoal testing until: (1) the first regularly scheduled refueling outage, or (2) the then installed charcoal was replaced, whichever occurred first. Since the first regularly scheduled refueling outage began on September 15, 1978, this condition no longer applies.

#### Paragraph 2.C.3

Paragraph 2.C.3 contains license conditions which were to be completed to the satisfaction of the Commission prior to achieving certain specified operating conditions or proceeding with operations beyond certain specified dates. These conditions have been satisfied. Satisfactory completion of paragraphs 2.C.3.a.(1) and 2.C.3.a.(2) was documented in a letter from R. S. Boyd (NRC) to J. W. Gore (BG&E) dated December 3, 1976. Satisfactory completion of paragraph 2.C.3.b.(1) was documented in a letter from D. B. Vassallo (NRC) to J. W. Gore (BG&E) dated January 28, 1977.

#### Paragraph 2.C.4

Paragraph 2.C.4 contains a license condition requiring the performance of a surveillance to provide early detection of reactivity or power distribution anomalies prior to the first regularly scheduled refueling outage and during operation at or above 50 percent of rated thermal power. The results were to be included in the monthly operating reports (MORs). The required surveillance was performed and the results were reported in MORs. This license condition has, therefore, been satisfactorily completed.

#### Paragraph 2.C.6

Paragraph 2.C.6 requires BG&E to complete the modifications identified in Paragraphs 3.1.1 through 3.1.21 of the NRC's FPSE for Calvert Cliffs dated September 14, 1979, submit additional information identified in Table 3.2 of the FPSE, and implement the administrative controls identified in Section 6 of the FPSE. A NRC staff safety evaluation dated September 27, 1982, indicates that these license conditions have been completed. The licensee has, therefore, requested that all of paragraph 2.C.6 be deleted with the exception of the first sentence of the third paragraph. This sentence requires BG&E to implement and maintain the administrative controls identified in Section 6 of the FPSE dated September 14, 1979.

Paragraph 2.E.

Paragraph 2.E. currently states that information previously contained in that paragraph is now contained in paragraph 2.D.2. BG&E has proposed that paragraph 2.E. be deleted since this information is available in other docketed sources and is not a license condition.

Paragraph 2.G.

Paragraph 2.G. states that the license shall be subject to the outcome of the proceedings in the United States Court of Appeals for the District of Columbia Circuit involving Natural Resources Defense Council v. Nuclear Regulatory Commission. The United States Court of Appeals for the D.C. Circuit decision favoring the Natural Resources Defense Council was essentially overruled by the United States Supreme Court in 1978 (Vermont Yankee v. NRDC) and no conditions applicable to Calvert Cliffs were upheld. BG&E has requested that paragraph 2.G. be deleted on the basis that its provisions no longer apply.

BG&E has also requested that Facility Operating License No. DPR-69 be retyped in its entirety and that certain editorial and format changes be made to the license as discussed below.

The licensee has proposed that the handwritten reference to Amendment No. 5 at paragraph 2.A. be deleted. The Amendment Number (5) would be included at the bottom of the page with a deletion mark through it indicating that a previous change had been made to this page. BG&E has proposed that handwritten references to previous License Amendments be similarly removed at paragraphs 2.C.1, 2.C.5, 2.C.7, 2.D., 2.F., and 2.H.

BG&E has requested that paragraphs 2.C.3 and 2.C.4 be removed from the license since the provisions of these paragraphs were deleted by Amendment No. 25. With the previously discussed deletions of paragraphs, pages 6, 6a, 7, and 7a of the license would be deleted and the remaining effective license conditions on these pages would be transferred to pages 4 and 5.

The licensee has also proposed that the designations for certain paragraphs be changed for consistency. The designation for paragraph 2.C.5 on page 4 would become 3. The designation for paragraph 2.C.7 on page 4 would become 5. The designation for paragraph 2.D. on page 5 would become D. The designation for paragraph 2.F. on page 5 would become E. The designation for paragraph 2.H. on page 5 would become F.

The NRC staff has concluded that the proposed deletions from Facility Operating License No. DPR-69 described above are acceptable since they represent requirements that have been satisfactorily completed or are no longer applicable. The staff has also concluded that the retyping of the license and the proposed editorial and format changes are acceptable since they are administrative in nature.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Maryland State official was notified of the proposed issuance of the amendments. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20, and changes recordkeeping, reporting, or administrative procedures or requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (58 FR 28052). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:  
J. E. Menning

Date: May 20, 1994

May 20, 1994

Docket Nos. 50-317  
and 50-318

Mr. Robert E. Denton  
Vice President - Nuclear Energy  
Baltimore Gas and Electric Company  
Calvert Cliffs Nuclear Power Plant  
1650 Calvert Cliffs Parkway  
Lusby, Maryland 20657-4702

Dear Mr. Denton:

SUBJECT: ISSUANCE OF AMENDMENTS FOR CALVERT CLIFFS NUCLEAR POWER PLANT,  
UNIT NO. 1 (TAC NO. M86140) AND UNIT NO. 2 (TAC NO. M86141)

The Commission has issued the enclosed Amendment No. 189 to Facility Operating License No. DPR-53 and Amendment No. 166 to Facility Operating License No. DPR-69 for the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Operating Licenses in response to your application transmitted by letter dated April 1, 1993.

The amendments make administrative, editorial, and format changes to the Operating Licenses. These changes include the deletion or incorporation, as appropriate, of all handwritten or "pasted-up" changes and the removal of all previous license conditions that have been completed to the satisfaction of the Commission. The changes and reformatting result in the Operating Licenses containing only those license conditions that are currently applicable.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:

Daniel G. McDonald, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 189 to DPR-53
2. Amendment No. 166 to DPR-69
3. Safety Evaluation

cc w/enclosures:

See next page

\*See previous concurrence

Distribution:

See attached sheet

LA:PDI-1 <i>[Signature]</i>	PM:PDI-1	PM:PDI-1 <i>[Signature]</i>	*OGC	D:PDI-1	
CVogan	JMenning <i>[Signature]</i>	DMcDonald		MBoyle <i>[Signature]</i>	
5/20/94	5/20/94	05/20/94	05/12/94	5/20/94	1/1

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