

Docket File



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555
July 1, 1991

Docket Nos. 50-317
and 50-318

Mr. G. C. Creel
Vice President - Nuclear Energy
Baltimore Gas and Electric Company
Calvert Cliffs Nuclear Power Plant
MD Rts. 2 & 4
P. O. Box 1535
Lusby, Maryland 20657

Dear Mr. Creel:

SUBJECT: ISSUANCE OF AMENDMENTS FOR CALVERT CLIFFS NUCLEAR POWER PLANT,
UNIT NO. 1 (TAC NO. 80112) AND UNIT NO. 2 (TAC NO. 80113)

The Commission has issued the enclosed Amendment No. 156 to Facility Operating License No. DPR-53 and Amendment No. 136 to Facility Operating License No. DPR-69 for the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated April 2, 1991.

The amendments modify the Technical Specifications (TS) Definitions, Section 1.8, Item 1.8.3, and the TS Surveillance Requirements, Section 4.6.1.1.b. Section 1.8 defines when containment integrity exists, Item 1.8.3 defines the conditions the containment air locks must meet to assure containment integrity, and Section 4.6.1.1.b specifies the required surveillances of the air locks to assure containment integrity. The changes remove the existing requirement that the air locks be operable, in both the TS definitions and surveillance sections, and replace it with the requirement that the air locks be in compliance with the requirements of TS Section 3.6.1.3. Section 3.6.1.3 provides the limiting conditions of operation (LCO), applicable modes, and actions to be taken (including the allowed out-of service times for repairs) when the air locks are inoperable prior to requiring a unit to shutdown.

FOR THE COMMISSION

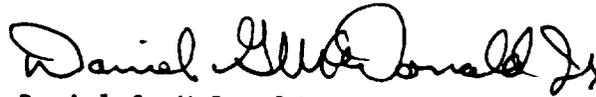
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July 1, 1991

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,



Daniel G. McDonald, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 156 to DPR-53
2. Amendment No. 136 to DPR-69
3. Safety Evaluation

cc w/enclosures:

See next page

Mr. G. C. Creel
Baltimore Gas & Electric Company

Calvert Cliffs Nuclear Power Plant
Unit Nos. 1 and 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 156
License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Baltimore Gas and Electric Company (the licensee) dated April 2, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-53 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 156, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 1, 1991



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 136
License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Baltimore Gas and Electric Company (the licensee) dated April 2, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-69 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.136 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 1, 1991

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 156 FACILITY OPERATING LICENSE NO. DPR-53

AMENDMENT NO. 136 FACILITY OPERATING LICENSE NO. DPR-69

DOCKET NOS. 50-317 AND 50-318

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
1-1* (DRP-53 only)	1-1* (DRP-53 only)
1-2	1-2
3/4 6-1	3/4 6-1
3/4 6-2*	3/4 6-2*

*Pages that did not change, but are overlief

1.0 DEFINITIONS

DEFINED TERMS

1.1 The **DEFINED TERMS** of this section appear in capitalized type and are applicable throughout these Technical Specifications.

THERMAL POWER

1.2 **THERMAL POWER** shall be the total reactor core heat transfer rate to the reactor coolant.

RATED THERMAL POWER

1.3 **RATED THERMAL POWER** shall be a total reactor core heat transfer rate to the reactor coolant of 2700 MWt.

OPERATIONAL MODE

1.4 An **OPERATIONAL MODE** shall correspond to any one inclusive combination of core reactivity condition, power level and average reactor coolant temperature specified in Table 1.1.

ACTION

1.5 **ACTION** shall be that part of a Specification which prescribes remedial measures required under designated conditions.

OPERABLE - OPERABILITY

1.6 A system, subsystem, train, component or device shall be **OPERABLE** or have **OPERABILITY** when it is capable of performing its specified function(s). Implicit in this definition shall be the assumption that all necessary attendant instrumentation, controls, normal and emergency electrical power sources, cooling or seal water, lubrication or other required auxiliary equipment that are required for the system, subsystem, train, component or device to perform its function(s) are also capable of performing their related support function(s).

DEFINITIONS

REPORTABLE EVENT

1.7 A **REPORTABLE EVENT** shall be any of those conditions specified in Section 50.73 to 10 CFR Part 50.

CONTAINMENT INTEGRITY

1.8 **CONTAINMENT INTEGRITY** shall exist when:

- 1.8.1 All penetrations required to be closed during accident conditions are either:
 - a. Capable of being closed by an **OPERABLE** containment automatic isolation valve system, or
 - b. Closed by manual valves, blind flanges, or deactivated automatic valves secured in their closed positions, except as provided in Table 3.6-1 of Specification 3.6.4.1.
- 1.8.2 All equipment hatches are closed and sealed,
- 1.8.3 Each airlock is in compliance with the requirements of Specification 3.6.1.3,
- 1.8.4 The containment leakage rates are within the limits of Specification 3.6.1.2, and
- 1.8.5 The sealing mechanism associated with each penetration (e.g., welds, bellows or O-rings) is **OPERABLE**.

CHANNEL CALIBRATION

1.9 A **CHANNEL CALIBRATION** shall be the adjustment, as necessary, of the channel output such that it responds with the necessary range and accuracy to known values of the parameter which the channel monitors. The **CHANNEL CALIBRATION** shall encompass the entire channel including the sensor and alarm and/or trip functions, and shall include the **CHANNEL FUNCTIONAL TEST**. The **CHANNEL CALIBRATION** may be performed by any series of sequential, overlapping or total channel steps such that the entire channel is calibrated.

DEFINITIONS

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 - b. Closed by manual valves, blind flanges, or deactivated automatic valves secured in their closed positions, except as provided in Table 3.6-1 of Specification 3.6.4.1.
- 1.8.2 All equipment hatches are closed and sealed,
- 1.8.3 Each airlock is in compliance with the requirements of Specification 3.6.1.3,
- 1.8.4 The containment leakage rates are within the limits of Specification 3.6.1.2, and
- 1.8.5 The sealing mechanism associated with each penetration (e.g., welds, bellows, or O-rings) is **OPERABLE**.

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1.9 A **CHANNEL CALIBRATION** shall be the adjustment, as necessary, of the channel output such that it responds with the necessary range and accuracy to known values of the parameter which the channel monitors. The **CHANNEL CALIBRATION** shall encompass the entire channel including the sensor and alarm and/or trip functions, and shall include the **CHANNEL FUNCTIONAL TEST**. The **CHANNEL CALIBRATION** may be performed by any series of sequential, overlapping or total channel steps such that the entire channel is calibrated.

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 PRIMARY CONTAINMENT

CONTAINMENT INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.1 Primary CONTAINMENT INTEGRITY shall be maintained.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

Without primary CONTAINMENT INTEGRITY, restore CONTAINMENT INTEGRITY within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.1 Primary CONTAINMENT INTEGRITY shall be demonstrated:

- a. At least once per 31 days by verifying that all penetrations* not capable of being closed by OPERABLE containment automatic isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in their positions, except as provided in Table 3.6-1 of Specification 3.6.4.1.
- b. By verifying that each containment air lock is in compliance with the requirements of Specification 3.6.1.3.
- c. By verifying that the equipment hatch is closed and sealed, prior to entering MODE 4 following a shutdown where the equipment hatch was opened, by conducting a Type B test per Appendix J to 10 CFR Part 50.

* Except valves, blind flanges, and deactivated automatic valves which are located inside the containment and are locked, sealed, or otherwise secured in the closed position. These penetrations shall be verified closed during each COLD SHUTDOWN except that such verification need not be performed more often than once per 92 days.

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 PRIMARY CONTAINMENT

CONTAINMENT INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.1 Primary **CONTAINMENT INTEGRITY** shall be maintained.

APPLICABILITY: **MODES** 1, 2, 3, and 4.

ACTION:

Without primary **CONTAINMENT INTEGRITY**, restore **CONTAINMENT INTEGRITY** within one hour or be in at least **HOT STANDBY** within the next 6 hours and in **COLD SHUTDOWN** within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.1 Primary **CONTAINMENT INTEGRITY** shall be demonstrated:

- a. At least once per 31 days by verifying that all penetrations* not capable of being closed by **OPERABLE** containment automatic isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in their positions, except as provided in Table 3.6-1 of Specification 3.6.4.1.
- b. By verifying that each containment air lock is in compliance with the requirements of Specification 3.6.1.3.
- c. By verifying that the equipment hatch is closed and sealed, prior to entering **MODE** 4 following a shutdown where the equipment hatch was opened, by conducting a Type B test per Appendix J to 10 CFR Part 50.

* Except valves, blind flanges, and deactivated automatic valves which are located inside the containment and are locked, sealed, or otherwise secured in the closed position. These penetrations shall be verified closed during each **COLD SHUTDOWN** except that such verification need not be performed more often than once per 92 days.

CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGES

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

- a. An overall integrated leakage rate of:
 1. $\leq L_a$ (346,000 SCCM), 0.20 percent by weight of the containment air per 24 hours at P_a , 50 psig, or
 2. $\leq L_t$ (61,600 SCCM), 0.058 percent by weight of the containment air per 24 hours at a reduced pressure of P_t , 25 psig.
- b. A combined leakage rate of $\leq 0.60 L_a$ (207,600 SCCM), for all penetrations and valves subject to Type B and C tests when pressurized to P_a .

APPLICABILITY: **MODES 1, 2, 3, and 4.**

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75 L_a$ (259,500 SCCM) or $0.75 L_t$ (46,200 SCCM), as applicable, or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.60 L_a$, restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR Part 50 using the methods and provisions of ANSI N45.4 - 1972:

- a. Three Type A tests (overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals during shutdown at either P_a (50 psig) or at P_t (25 psig) during each 10-year service period.

CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGES

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

- a. An overall integrated leakage rate of:
 1. $\leq L_a$ (346,000 SCCM), 0.20 percent by weight of the containment air per 24 hours at P_a , 50 psig, or
 2. $\leq L_t$ (44,600 SCCM), 0.042 percent by weight of the containment air per 24 hours at a reduced pressure of P_t , 25 psig.
- b. A combined leakage rate of $\leq 0.60 L_a$ (207,600 SCCM) for all penetrations and valves subject to Type B and C tests when pressurized to P_a .

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75 L_a$ (259,500 SCCM), or $0.75 L_t$ (33,400 SCCM), as applicable, or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.60 L_a$, restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4 - 1972:

- a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals during shutdown at either P_a (50 psig) or at P_t (25 psig) during each 10-year service period.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 156 TO FACILITY OPERATING LICENSE NO. DPR-53
AND AMENDMENT NO. 136 TO FACILITY OPERATING LICENSE NO. DPR-69
BALTIMORE GAS AND ELECTRIC COMPANY
CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-317 AND 50-318

1.0 INTRODUCTION

By letter dated April 2, 1991, the Baltimore Gas and Electric Company (the licensee) submitted a request for changes to the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2, Technical Specifications (TS). The requested changes would modify the Definitions, Section 1.8, and the TS Surveillance Requirements, Section 4.6.1.1.b. Section 1.8 defines when containment integrity exists, Item 1.8.3 defines the conditions the containment air locks must meet to assure containment integrity, and Section 4.6.1.1.b specifies the required surveillances of the air locks to assure containment integrity. The proposed changes will remove the existing requirement that the air locks be operable, in both the TS definitions and surveillance sections, and replace it with the requirement that the air locks be in compliance with the requirements of TS Section 3.6.1.3. Section 3.6.1.3 provides the limiting conditions of operation (LCO), applicable modes, and actions to be taken (including the allowed out-of service times for repairs) when the airlocks are inoperable prior to requiring a unit to shutdown.

2.0 EVALUATION

As noted in the licensee's submittal, the Calvert Cliffs Unit 2 Technical Specifications, as originally issued, and the subsequent conversion of Unit 1 to the Standardized Technical Specification format, were written such that the definition and the surveillance sections both require operable containment air locks to show containment integrity. However, TS Section 3.6.1.3 provides the LCO, applicable modes, and the actions to be taken when the air locks are inoperable. The design of the air locks and the impact of their being inoperable in relation to overall primary containment integrity was taken into consideration in establishing the LCOs and the action statements of TS Section 3.6.1.3. The proposed change will allow the use of the current TS specified out-of-service times to repair air lock problems prior to requiring a unit to shutdown. The current TS requires shutdown within 36 hours with no provision to allow for the intended out-of-service times specified. This resulted in unintended and unnecessary restrictions.

Overall containment integrity is required to limit offsite doses from postulated Designs Basis Accidents (DBA) described in the Calvert Cliffs Nuclear Power Plant, Units 1 and 2, Updated Final Safety Analysis Report (UFSAR). The DBAs identified in Chapter 14 of the UFSAR which could occur at power and result in a release of radioactive material within the affected unit include a loss of coolant accident, main steam line break, and a control element assembly ejection accident. The containment design, safety function, and the related TS provide reasonable assurance that the release of fission products to the environment is limited to a small fraction of the total containment volume.

The licensee's safety evaluation indicated that the containment design provides for an allowable leakage rate (L_a) of 0.20% of the containment air weight per day. This leakage rate was used in the offsite radiological dose calculations for Calvert Cliffs Nuclear Power Plant, Units 1 and 2, and is the basis for TS 3.6.1.2.a. An additional restriction imposed by 10 CFR Part 50, Appendix J, and TS 3.6.1.2.b, is that no more than 60% of this L_a be through Type, B and C penetrations. The air locks are Type B penetrations. Each containment air lock is further limited, by TS 3.6.1.3.b, to an overall leakage rate of 5% L_a .

Both of the containment air locks are designed with a door on each end. Both doors are required to be closed during operation unless the air lock is being used for transit entry or exit through the containment. Then, at least one door is required to be closed. Each door contains a double seal. The area between the double seal on a door is maintained pressurized to provide the leakage barrier. In accordance with TS Surveillance Requirement 4.6.1.3.a, the seal leakage of each containment air lock is routinely verified following its use to be no greater than 0.02% of the L_a .

The operable seal of either door is sufficient to maintain the required containment barrier since each is designed to withstand the peak containment pressure calculated to occur following a DBA; however, both doors are kept closed when not in use to provide an additional barrier. TS 3.6.1.3 was written to recognize this capability of a single door to serve as a leakage barrier and provided an allowed out-of-service time for one door in order to attempt repairs prior to requiring a shutdown. However, due to the existing TS wording by which the containment integrity requirements reference the air lock doors (stating that the doors must be operable), the intent to govern airlock doors with the action statements of TS 3.6.1.3 is not allowed.

The staff has reviewed and agrees with the licensee's findings as detailed above and therefore; find the proposed changes acceptable. The staff's finding was based on: 1) the design of the air lock doors 2) the surveillance requirements of TS 4.6.1.3 imposed on the air locks, 3) the LCOs and actions required by TS 3.6.1.3 when an air lock is determined to be inoperable and 4) the restrictions imposed by TS 3.6.1.2a and 3.6.1.2b relating to the overall allowed containment leakage, including any air lock door leakage.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Maryland State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (56 FR 24205). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:
D. McDonald

Date: July 1, 1991

July 1, 1991

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Daniel G. McDonald, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No.156 to DPR-53
- 2. Amendment No.136 to DPR-69
- 3. Safety Evaluation

cc w/enclosures:
See next page

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amendment NOT issued until 30 days after publication of initial safety notice in Federal Register