



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555

September 6, 1991

Docket Nos. 50-317
50-318

Mr. G. C. Creel
Vice President - Nuclear Energy
Baltimore Gas and Electric Company
Calvert Cliffs Nuclear Power Plant
MD Rts. 2 & 4
P. O. Box 1535
Lusby, Maryland 20657

Dear Mr. Creel:

SUBJECT: NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENTS FOR THE
CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT 1 (TAC NO. 81402)
AND UNIT 2 (TAC NO. 81403)

By letter dated September 5, 1991, you requested amendments to Facility Operating License Nos. DPR-53 and DPR-69 for the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The request is to modify the Technical Specification (TS) Limiting Condition for Operation (LCO) and action requirements to allow for the inspection and maintenance of the shared emergency diesel generator (EDG) No. 12 when one unit is operating and the other unit is shutdown. The TS Bases are also changed to support the proposed TS modifications.

Enclosed for your information is a copy of a Notice of Consideration of Issuance of Amendments to the Facility Operating Licenses and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing. The Notice has been forwarded to the Office of Federal Register for publication.

Sincerely,

Brian C. McCabe for

Daniel G. McDonald, Jr., Senior Project
Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
Notice

cc w/enclosure:
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Mr. G. C. Creel
Baltimore Gas & Electric Company

Calvert Cliffs Nuclear Power Plant
Unit Nos. 1 and 2

cc:

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UNITED STATES NUCLEAR REGULATORY COMMISSIONBALTIMORE GAS AND ELECTRIC COMPANYDOCKET NOS. 50-317 AND 50-318NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO
FACILITY OPERATING LICENSE, PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION DETERMINATION, AND OPPORTUNITY FOR HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License Nos. DPR-53 and DPR-69 issued to Baltimore Gas and Electric Company (the licensee) for operation of the Calvert Cliffs Nuclear Power Plant, Units 1 and 2 located in Calvert County, Maryland.

The proposed amendment would allow the removal of the dedicated Class 1E shared emergency power source from a shutdown unit for seven days. The current Technical Specifications (TS) are structured so that the shared No. 12 Emergency Diesel Generator (EDG) cannot be removed from service for more than 72 hours without both Calvert Cliffs units being shutdown. The licensee is required to remove No. 12 EDG from service on October 18, 1991, to perform the inspection and maintenance activities required by TS 4.8.1.1.2.d.1. The licensee's current maintenance policy recommends that 7 days be allotted to conduct the inspection and maintenance activities in a quality manner. To facilitate this and certain other required maintenance and surveillance testing, the licensee will shut down one unit before the current surveillance interval for the No. 12 EDG expires. The operating unit requires two Class 1E emergency power sources to be operable. The operable dedicated diesel from the

shutdown unit will be aligned to the operating unit during the time that the No. 12 EDG is out of service. Compensatory measures will be taken for the shutdown unit to provide assurance that A.C. electrical power will be available to the equipment necessary to maintain the shutdown unit in a safe condition during this scheduled maintenance interval.

Specifically, the proposed changes will modify TS Sections 3.8.1.2, "ELECTRICAL POWER SYSTEMS - SHUTDOWN", and 3.8.2.2, "A.C. DISTRIBUTION - SHUTDOWN", for both units. The TS change provides for a special test exception from the present requirement for an operable EDG on the shutdown unit, and will allow the dedicated EDG from the shutdown unit to be aligned to the operating unit. Compensatory measures which will be taken for the shutdown unit included in the proposed TS changes are:

Either two 500 kV offsite power circuits or one 500 kV offsite power circuit and the 69 kV Southern Maryland Electric Cooperative (SMECO) offsite power circuit shall be verified available and the availability confirmed once per shift;

Core alterations, positive reactivity changes, movement of irradiated fuel and movement of heavy loads over irradiated fuel will be suspended, and containment penetration closure will be established; and

A temporary diesel generator capable of carrying the shutdown unit's A.C. electrical loads will be verified available.

If these conditions are not met, then 4 hours are allotted to restore availability of the temporary diesel generator and the off-site power sources. If they are not made available within the next 4 hours, then an operable EDG must be aligned to the shutdown unit within 4 hours. This will place the operating unit in TS ACTION statement 3.8.1.1.b (lack of two EDGs) which allows 72 hours to restore two operable EDGs or be in hot standby in 6 hours and in cold shutdown in 30 hours. The TS Bases are also modified to support the above proposed changes.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

- (1) Would not involve a significant increase in the probability or consequences of an accident previously evaluated.

This change was evaluated to determine its impact on the probability or consequences of a loss of offsite power event, a boron dilution event, and a fuel handling incident. The boron dilution event and the fuel handling incident are the only two accidents that are explicitly analyzed in the Updated Final Safety Analysis Report (UFSAR) for a shutdown unit.

There is a very low probability of occurrence of a loss of offsite power during the seven day period that No. 12 Emergency Diesel Generator (EDG) would be out of service for inspection and maintenance. This configuration will only be required until two additional Class 1E EDGs (one for each unit) will be installed (about February 1995) as part of BG&E's implementation of the Station Blackout Rule (10 CFR 50.63). These new EDGs would provide sufficient flexibility for scheduling and performing maintenance such that this relief from technical specification requirements will no longer be needed.

To ensure a low probability of a loss of offsite power, BG&E has reviewed potential precursors such as weather events and onsite work activities. The Calvert Cliffs offsite power supply is designed to be diverse and redundant, and is therefore inherently capable of

withstanding severe weather events. In addition, BG&E's Emergency Response Plan already requires that certain actions be taken, up to and including shutdown of both units, on the approach of a severe storm.

As regards work-related events, the probability of a loss of offsite power is maintained low by prohibiting planned maintenance on the two 500 kV transmission lines and in the switchyard. Availability of the independent offsite power source, the 69 kV Southern Maryland Electric Cooperative (SMECO) feeder, will be verified once per shift. This requirement to maintain two available qualified offsite power sources compares favorably with the requirements of the current technical specifications to maintain only one offsite source to the shutdown unit.

A temporary diesel generator will also be installed to provide a backup onsite source of power capable of supporting necessary safety-related loads.

Finally, potential accident precursors such as core alterations, positive reactivity insertions, movement of irradiated fuel, and movement of heavy loads over irradiated fuel will be prohibited while No. 12 EDG is out of service. Therefore, the probability of a boron dilution event or fuel handling incident is decreased during the operations allowed by this change.

The requirement to maintain containment penetration closure while No. 12 EDG is out of service ensures that the consequences of an accident would not be significantly increased.

Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

- (2) Would not create the possibility of a new or different type of accident from any accident previously evaluated.

A temporary diesel generator is being installed onto the 4 kV bus of a shutdown (Mode 5 or 6) unit while the dedicated EDG for this unit is transferred for up to seven days to the operating unit. This configuration allows the performance of inspection and maintenance required by Technical Specification 4.8.1.1.2.d.1 for No. 12 EDG. This change has been evaluated and it has been determined that this installation does not impair any existing safety-related equipment needed to maintain the unit in a safe shutdown condition. Differences in the operation of the temporary diesel generator and the permanent EDGs include manual starting of the temporary diesel generator and manual loading of the 4 kV bus. These operations are not significantly different from typical operator activities.

Therefore, the proposed changes would not create the possibility of a new or different type of accident from any accident previously evaluated.

- (3) Would not involve a significant reduction in a margin of safety.

The operability of the minimum specified A.C. and D.C. electrical power sources and associated distribution systems during Modes 5 and 6 ensure that; 1) the unit can be maintained in Mode 5 or 6 for extended time periods and, 2) sufficient instrumentation and control capability is available for monitoring and maintaining the unit status. The proposed change does not affect the D.C. power sources or the A.C. and D.C. distribution systems. It affects only the A.C. power sources in that we are removing the emergency A.C. power source from the shutdown unit for seven days. This change will have no impact on the offsite power sources. Compensatory measures will be taken for the loss of the emergency power source. They are:

- o requiring that two offsite power sources are available,
- o core alterations, positive reactivity changes, movement of irradiated fuel, and movement of heavy loads over irradiated fuel will be suspended, and
- o containment penetration closure will be established.

These compensatory measures reduce the potential for a loss of offsite power, a fuel handling accident, and a boron dilution event during the seven days that the emergency power source is not available to the shutdown unit. Additionally, we are providing a temporary diesel generator capable of supplying the loads necessary to maintain the unit in a safe condition. These measures, along with the infrequent need to enter this condition, ensure that the margin of safety is not significantly reduced.

Therefore, the proposed change would not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within thirty (30) days after the date of publication of

this notice will be considered in making any final determination. The Commission will not normally make a final determination unless it receives a request for a hearing.

Written comments may be submitted by mail to the Regulatory Publications Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and should cite the publication date and page number of this FEDERAL REGISTER notice. Written comments may also be delivered to Room P-223, Phillips Building, 7920 Norfolk Avenue, Bethesda, Maryland, from 7:30 a.m. to 4:15 p.m. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555. The filing of requests for hearing and petitions for leave to intervene is discussed below.

By October 15, 1991 , the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555 and at the local public document room located at the Calvert County Library, Fourth Street, P.O. Box 405, Prince Frederick, Maryland. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic

Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted.

In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the FEDERAL REGISTER a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555, by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 325-6000 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number 3737 and the following message addressed to Robert A. Capra: petitioner's name and telephone number, date petition was mailed, plant name, and publication

date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to Jay E. Silbert, Esquire, Shaw, Pittman, Potts and Trowbridge, 2300 N Street, NW., Washington, DC 20037, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated September 5, 1991, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555 and at the local public document room located at the Calvert County Public Library, Fourth Street, P.O. Box 405, Prince Frederick, Maryland 20678.

Dated at Rockville, Maryland, this 6th day of September 1991.

FOR THE NUCLEAR REGULATORY COMMISSION



Brian C. McCabe, Acting Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

September 6, 1991

Docket Nos. 50-317
50-318

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Mr. G. C. Creel
Vice President - Nuclear Energy
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MD Rts. 2 & 4
P. O. Box 1535
Lusby, Maryland 20657

Dear Mr. Creel:

SUBJECT: NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENTS FOR THE
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AND UNIT 2 (TAC NO. 81403)

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Sincerely,

Brian C. McCabe for

Daniel G. McDonald, Jr., Senior Project
Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
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