



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

September 11, 1991

Docket Nos. 50-317  
and 50-318

Mr. G. C. Creel  
Vice President - Nuclear Energy  
Baltimore Gas and Electric Company  
Calvert Cliffs Nuclear Power Plant  
MD Rts. 2 & 4  
P. O. Box 1535  
Lusby, Maryland 20657

Dear Mr. Creel:

SUBJECT: ISSUANCE OF AMENDMENTS FOR CALVERT CLIFFS NUCLEAR POWER PLANT,  
UNIT NO. 1 (TAC NO. 77616) AND UNIT NO. 2 (TAC NO. 77617)

The Commission has issued the enclosed Amendment No. 161 to Facility Operating License No. DPR-53 and Amendment No. 141 to Facility Operating License No. DPR-69 for the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application transmitted by letter dated August 23, 1990, as supplemented on September 28, 1990, and June 28, 1991.

The amendments change specific titles of managers throughout the TS, Section 6.0, Administrative Controls, to be consistent with the organization of Baltimore Gas and Electric Company at the Calvert Cliffs' facilities as of October 1, 1990. TS 6.5.1.2 deletes specific titles for members of the Plant Operations and Safety Review Committee (POSRC), denotes the areas of expertise for the POSRC members, and adds a requirement for the Plant General Manager to appoint the POSRC members. A new TS, 6.5.1.3, specifies that the POSRC Chairman and Alternate Chairman are appointed by the Plant General Manager. Several TS in Section 6.5 are renumbered, some removed, and the POSRC functions are changed. A new TS, 6.5.1.8d, is added requiring POSRC to evaluate root causes and recommended corrective actions.

TS Sections, 6.5.2 and 6.5.3, were added to describe and provide the requirements for the use of committees or individuals to perform selected reviews in lieu of reviews by the POSRC. Several TS in Section 6.8, Procedures, were reorganized, renumbered, and new conditions relating to the Procedure Review Committee and Qualified Reviewer Program were added. A requirement to maintain records of the Procedure Review Committee was added to TS Section 6.10.

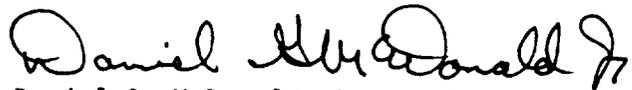
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September 11, 1991

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,



Daniel G. McDonald, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 161to DPR-53
2. Amendment No. 141to DPR-69
3. Safety Evaluation

cc w/enclosures:  
See next page

Mr. G. C. Creel  
Baltimore Gas & Electric Company

Calvert Cliffs Nuclear Power Plant  
Unit Nos. 1 and 2

cc:

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Resident Inspector  
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Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
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DATED: September 11, 1991

AMENDMENT NO. 161 TO FACILITY OPERATING LICENSE NO. DPR-53-CALVERT CLIFFS UNIT 1  
AMENDMENT NO. 141 TO FACILITY OPERATING LICENSE NO. DPR-69-CALVERT CLIFFS UNIT 2

Docket File

NRC & Local PDRs

PDI-1 Reading

S. Varga, 14/E/4

J. Calvo, 14/A/4

R. Capra

C. Vogan

D. McDonald

C. Cowgill

OGC-WF

D. Hagan, 3302 MNBB

G. Hill (8), P-137

Wanda Jones, P-130A

C. Grimes, 11/F/23

ACRS (10)

GPA/PA

OC/LFMB

Plant File

FAllenspach

cc: Plant Service list

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DF01  
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September 11, 1991

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:

Daniel G. McDonald, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 161 to DPR-53
- 2. Amendment No. 141 to DPR-69
- 3. Safety Evaluation

cc w/enclosures:  
See next page

OFC	:PDI-1:LA	:PDI-1:PM	:OGC	:PDI-1:D
NAME	:CVogan	:McDonald:avl	:RACapra	
DATE	:8/29/91	:8/29/91	:9/13/91	:9/11/91

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 161  
License No. DPR-53

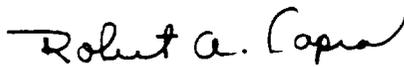
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Baltimore Gas and Electric Company (the licensee) dated August 23, 1990, as supplemented on September 28, 1990, and June 28, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-53 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 161, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 11, 1991



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 141  
License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Baltimore Gas and Electric Company (the licensee) dated August 23, 1990, as supplemented on September 28, 1990, and June 28, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-69 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 141, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

*Robert A. Capra*

Robert A. Capra, Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 11, 1991

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 161 - FACILITY OPERATING LICENSE NO. DPR-53

AMENDMENT NO. 141 - FACILITY OPERATING LICENSE NO. DPR-69

DOCKET NOS. 50-317 AND 50-318

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
6-1	6-1
6-1a	6-1a
6-6	6-6
6-7	6-7
6-7a	6-7a
6-8	6-8
-	6-8a
-	6-8b
-	6-8c
6-9	6-9
6-10	6-10
6-11	6-11
6-12	6-12
6-13	6-13
6-13a	6-13a
6-14	6-14
6-15*	6-15*
6-19*	6-19*
6-20	6-20
6-21*	6-21*
6-22	6-22

\*Pages that did not change, but are overleaf.

## ADMINISTRATIVE CONTROLS

### 6.1 RESPONSIBILITY

6.1.1 The Plant General Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

### 6.2 ORGANIZATION

#### 6.2.1 ONSITE & OFFSITE ORGANIZATIONS

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in FSAR Chapter 12, and updated in accordance with 10 CFR 50.71(e).
- b. The Plant General Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Vice President-Nuclear Energy shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

#### 6.2.2 UNIT STAFF

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.

Effective at 12:01 a.m. on November 30, 1988. Replaces page 6-1(t).

## ADMINISTRATIVE CONTROLS

- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown, and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All **CORE ALTERATIONS** after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of both units (4 members) or any personnel required for other essential functions during a fire emergency.
- g. The Superintendent-Nuclear Operations shall hold or have held a senior reactor operator license at Calvert Cliffs. The General Supervisor-Nuclear Plant Operations, Shift Supervisor and Control Room Supervisor shall hold a senior reactor operator license. The Control Room Operator shall hold a reactor operator license.

## ADMINISTRATIVE CONTROLS

### 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the Radiation Safety Engineer who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and (2) the Shift Technical Advisor who shall have a Bachelor's Degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

### 6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the General Supervisor-Nuclear Training and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55, as applicable.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the General Supervisor-Planning and Support and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975.

### 6.5 REVIEW AND AUDIT

#### 6.5.1 PLANT OPERATIONS AND SAFETY REVIEW COMMITTEE (POSRC)

##### FUNCTION

6.5.1.1 The POSRC shall function to advise the Plant General Manager on all matters related to nuclear safety.

##### COMPOSITION

6.5.1.2 The POSRC shall be composed of at least seven, but no more than ten, members, including the Chairman. Members shall collectively have experience in the following areas:

- Nuclear Operations
- Electrical and Controls Maintenance
- Chemistry
- Mechanical Maintenance
- Nuclear Engineering
- Radiation Safety
- Plant Engineering
- Design Engineering

Members shall be appointed in writing by the Plant General Manager. Members shall have a minimum of eight years power plant experience of which a minimum of three years shall be nuclear power experience. At least one member shall have an SRO license on Calvert Cliffs Units 1 and 2

## ADMINISTRATIVE CONTROLS

### CHAIRMAN

6.5.1.3 The Chairman and alternate Chairmen of the POSRC shall be appointed in writing by the Plant General Manager. Chairmen shall have a minimum of 10 years power plant experience of which a minimum of three years shall be nuclear power experience.

### ALTERNATES

6.5.1.4 All alternate members shall be appointed in writing by the Plant General Manager. Alternate members shall have a minimum of eight years power plant experience of which a minimum of three years shall be nuclear power experience.

### MEETING FREQUENCY

6.5.1.5 The POSRC shall meet at least once per calendar month and as convened by the POSRC Chairman or one of the designated alternates.

### QUORUM

6.5.1.6 A quorum of the POSRC shall include the Chairman or one of the designated alternate chairmen and shall consist of a majority of the members, including alternates. No more than half of the quorum shall be alternates, including an alternate chairman.

### RESPONSIBILITIES

6.5.1.7 The POSRC shall be responsible for the following except for those items designated for review by the Procedure Review Committee or Qualified Reviewer in accordance with Specification 6.5.2 and 6.5.3, respectively:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, and 2) any other proposed procedures or changes thereto as determined by the Plant General Manager to affect nuclear safety.

Cross-disciplinary reviews of these procedures are conducted in accordance with administrative procedures in addition to the reviews conducted by POSRC, the Procedure Review Committee, or Qualified Reviewer.

- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.

ADMINISTRATIVE CONTROLS

- e. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Off-Site Safety Review Committee.
- f. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Off-Site Safety Review Committee.
- g. Review of changes to the **PROCESS CONTROL PROGRAM** and the **OFFSITE DOSE CALCULATION MANUAL**.
- h. Review of all 10 CFR 50.59 Safety Evaluations that support procedures in 6.5.1.7.a and changes or modifications in 6.5.1.7.d.
- i. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Plant General Manager, the Vice President Nuclear Energy and to the Chairman of the Off-Site Safety Review Committee.
- j. Review of all **REPORTABLE EVENTS**.
- k. Review of facility operations to detect potential safety hazards.
- l. Review of any accidental, unplanned or uncontrolled radioactive release that exceeds 25% of the limits of Specification 3.11.1.2, 3.11.2.2 or 3.11.2.3, including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and for forwarding of these reports to the Plant General Manager and the Off-Site Safety Review Committee.
- m. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Off-Site Safety Review Committee.

## ADMINISTRATIVE CONTROLS

### AUTHORITY

- 6.5.1.8 The Plant Operations and Safety Review Committee shall:
- a. Recommend to the approval authority approval or disapproval of procedures considered under 6.5.1.7.a.
  - b. Recommend to the Plant General Manager approval or disapproval of items considered under 6.5.1.7(b) through (h) above.
  - c. Render determinations in writing with regard to whether or not each item considered under 6.5.1.7(a) through (h) above constitutes an unreviewed safety question.
  - d. Evaluate root causes and recommended actions to prevent recurrence for items considered under 6.5.1.7(i) through (l).
  - e. Provide written notification within 24 hours to the Vice President-Nuclear Energy and the Chairman of the Off-Site Safety Review Committee of disagreement between the POSRC and the responsible approval authority in the case of item 6.5.1.7.a or between the POSRC and the Plant General Manager; however, the Plant General Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

### RECORDS

6.5.1.9 The POSRC shall maintain written minutes of each meeting and copies shall be provided to the Vice President-Nuclear Energy, Chairman of the Off-Site Safety Review Committee, and the Plant General Manager.

## 6.5.2 PROCEDURE REVIEW COMMITTEE

### FUNCTION

6.5.2.1 The Procedure Review Committee may function to review items listed in Specification 6.5.1.7(a) in lieu of review by POSRC or Qualified Reviewer as directed by the Plant General Manager.

### COMPOSITION

6.5.2.2 The Procedure Review Committee shall be composed of a Chairman and eight individuals who shall collectively have expertise in the areas contained in Technical Specification 6.5.1.2.

## ADMINISTRATIVE CONTROLS

Members shall be appointed in writing by the Plant General Manager. Members shall have a minimum of eight years power plant experience of which a minimum of three years shall be nuclear power experience. At least one member shall be a POSRC member or alternate. The charter for the Procedure Review Committee shall include a description of membership, qualifications, functions, and reports and shall be described in plant administrative procedures. The Procedure Review Committee may be dissolved at the discretion of the Plant General Manager.

### CHAIRMAN

6.5.2.3 The Chairman and alternate Chairmen of the Procedure Review Committee shall be appointed in writing by the Plant General Manager. Chairmen shall have a minimum of eight years power plant experience of which a minimum of three years shall be nuclear power experience.

### ALTERNATES

6.5.2.4 All alternate members shall be appointed in writing by the Plant General Manager. Alternate members shall have a minimum of eight years power plant experience of which a minimum of three years shall be nuclear power experience.

### MEETING FREQUENCY

6.5.2.5 The Procedure Review Committee shall meet at least once per calendar month and as convened by the Chairman or his designated alternates.

### QUORUM

6.5.2.6 A quorum for the Procedure Review Committee shall consist of the Chairman or one of the designated alternate Chairmen and three primary or alternate members provided at least four disciplines are represented.

### AUTHORITY

6.5.2.7 The Procedure Review Committee shall:

- a. Recommend to the Approval Authority approval or disapproval of procedures considered under 6.5.1.7(a).
- b. Render determinations in writing with regard to whether or not each procedure under 6.5.1.7(a) constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President-Nuclear Energy and the Chairman of the Off-Site Safety Review Committee of disagreements between the Procedure Review Committee and the responsible approval authority. The Plant General Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

## ADMINISTRATIVE CONTROLS

### RECORDS

6.5.2.8 The Procedure Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Plant General Manager.

### 6.5.3 QUALIFIED REVIEWERS

#### FUNCTION

6.5.3.1 The Plant General Manager may designate specific procedures or classes of procedures described in Specification 6.5.1.7.a to be reviewed by Qualified Reviewers in lieu of review by POSRC or the Procedure Review Committee.

#### AUTHORITY

6.5.3.2 Qualified Reviewers shall:

- a. Recommend to the approval authority approval or disapproval of designated procedures and changes considered under 6.5.1.7.a, and
- b. Render determination in writing with regard to whether or not each procedure under 6.5.1.7.a constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President-Nuclear Energy and the Chairman of the Off-Site Safety Review Committee of disagreements between the Qualified Reviewer and the approval authority. The Plant General Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1. above.

#### CERTIFICATION

6.5.3.3 Qualified Reviewers shall be nominated, trained, and certified in accordance with administrative procedures. Certification shall be by a department manager.

6.5.3.4 Certification requirements of personnel designated as Qualified Reviewers shall be in accordance with administrative procedures.

## ADMINISTRATIVE CONTROLS

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Qualified Reviewers shall have:

- a. A Bachelors degree in engineering, related science, or technical discipline, and two years of nuclear power plant experience;

OR

- b. Six years of nuclear power plant experience

OR

- c. Equivalent combination of education and experience as approved by a Department Manager.

## RECORDS

6.5.3.5 Review of procedures by Qualified Reviewers shall be documented in accordance with administrative procedures.

## 6.5.4 OFF-SITE SAFETY REVIEW COMMITTEE (OSSRC)

### FUNCTION

6.5.4.1 The Off-Site Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy and non-destructive examination
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

## ADMINISTRATIVE CONTROLS

### COMPOSITION

6.5.4.2 The OSSRC shall be composed of at least seven members, including the Chairman. Members of the OSSRC may be from the Nuclear Energy Division or other BG&E organization or from organizations external to BG&E and shall collectively have expertise in all of the areas of 6.5.4.1.

### QUALIFICATIONS

6.5.4.3 The Chairman, members and alternate members of the OSSRC shall be appointed in writing by the Vice President-Nuclear Energy and shall have an academic degree in engineering or a physical science, or the equivalent, and in addition shall have a minimum of five years technical experience in one or more areas given in 6.5.4.1. No more than two alternates shall participate as voting members in OSSRC activities at any one time.

### CONSULTANTS

6.5.4.4 Consultants shall be utilized as determined by the OSSRC Chairman to provide expert advice to the OSSRC.

### MEETING FREQUENCY

6.5.4.5 The OSSRC shall meet at least once per six months.

### QUORUM

6.5.4.6 The quorum of the OSSRC necessary for the performance of the OSSRC review and audit functions of these Technical Specifications shall consist of more than half the OSSRC membership or at least four members, whichever is greater. This quorum shall include the Chairman or his appointed alternate and the OSSRC members, including appointed alternates, meeting the requirements of Specification 6.5.4.3. No more than a minority of the quorum shall have line responsibility for operation of the plant.

## ADMINISTRATIVE CONTROLS

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### REVIEW

6.5.4.7 The OSSRC shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provisions of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes in Technical Specifications or this Operating License.
- e. Violation of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. All REPORTABLE EVENTS.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meetings minutes of the POSRC.

## ADMINISTRATIVE CONTROLS

### AUDITS

6.5.4.8.1 Audits of facility activities shall be performed under the cognizance of the OSSRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualification of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR Part 50, at least once per 24 months.
- e. Deleted.
- f. The Safeguards Contingency Plan and implementing procedures at least once per 12 months in accordance with 10 CFR 73.40(d).
- g. Any other area of facility operation considered appropriate by the OSSRC or the Vice President-Nuclear Energy.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.
- k. The radiological environmental monitoring program and the results thereof at least once per 12 months.
- l. The **OFFSITE DOSE CALCULATION MANUAL** and implementing procedures at least once per 24 months.
- m. The **PROCESS CONTROL PROGRAM** and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months.

## ADMINISTRATIVE CONTROLS

- n. The performance of activities required by The Quality Assurance Program for effluent and environmental monitoring at least once per 12 months.

6.5.4.8.2 Review of facility activities shall be performed under the cognizance of the OSSRC. These reviews shall encompass:

- a. The Facility Emergency Plan and implementing procedures at least once per 12 months in accordance with 10 CFR Part 50.54(t).

### AUTHORITY

6.5.4.9 The OSSRC shall report to and advise the Vice President-Nuclear Energy on those areas of responsibility specified in Section 6.5.4.7 and 6.5.4.8.

### RECORDS

6.5.4.10 Records of OSSRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each OSSRC meeting shall be prepared, approved and forwarded to the Vice President-Nuclear Energy within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.4.7 above, shall be prepared, approved and forwarded to the Vice President-Nuclear Energy within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.4.8 above, shall be forwarded to the Vice President-Nuclear Energy and to the management positions responsible for the areas audited within 30 days after completion of the audit.

## 6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the POSRC and the results of this review shall be submitted to the OSSRC and the Vice President-Nuclear Energy.

## ADMINISTRATIVE CONTROLS

### 6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least **HOT STANDBY** within one hour.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Vice President-Nuclear Energy and the OSSRC shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the POSRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the OSSRC and the Vice President-Nuclear Energy within 14 days of the violation.

### 6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.\*
- g. The amount of overtime worked by plant staff members performing safety related functions must be limited in accordance with the NRC Policy Statement on working Hours (Generic Letter No. 82-12).

\* POSRC is only required to review Fire Protection procedures and changes thereto which affect nuclear safety.

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- h. PROCESS CONTROL PROGRAM implementation.
- i. OFFSITE DOSE CALCULATION MANUAL implementation.

6.8.2 The Plant General Manager may designate specific procedures or classes of procedures in writing to be reviewed by the Procedure Review Committee or by Qualified Reviewers in lieu of review by the POSRC. Review by the Procedure Review Committee shall be in accordance with paragraph 6.5.2. Review by Qualified Reviewers shall be in accordance with paragraph 6.5.3.

6.8.3 Procedures listed in 6.8.1 shall be approved by the Plant General Manager or by cognizant Managers, Superintendents, or General Supervisors (or Directors) that report directly to a Manager prior to implementation as specified by administrative requirements. The approval authority for specific procedures or classes of procedures shall be designated in writing by the Plant General Manager and shall be a different individual from the Qualified Reviewer.

6.8.4 Each procedure of 6.8.1 above and changes thereto shall be reviewed periodically as set forth in administrative procedures.

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6.8.5 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the POSRC, the Procedure Review Committee, or by a Qualified Reviewer and approved by the designated approval authority within 14 days of implementation.

## 6.9 REPORTING REQUIREMENTS

### ROUTINE REPORTS

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

### STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

6.9.1.2 The startup report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

## ADMINISTRATIVE CONTROLS

6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

### ANNUAL REPORTS<sup>1/</sup>

6.9.1.4 Annual reports covering the activities of the unit as described below for the previous calendar year shall be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

6.9.1.5 Reports required on an annual basis shall include:

- a. A tabulation on an annual basis of the number of station, utility, and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions, <sup>2/</sup> e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
- b. The complete results of steam generator tube inservice inspections performed during the report period (reference Specification 4.4.5.5.b).
- c. Documentation of all failures and challenges to the pressurizer PORVs or safety valves.

<sup>1/</sup> A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

<sup>2/</sup> This tabulation supplements the requirements of 10 CFR Part 20.407.

## ADMINISTRATIVE CONTROLS

### 6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. All REPORTABLE EVENTS.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to Operating Procedures.
- g. Records of radioactive shipments.
- h. Records of sealed source and fission detector leak tests and results.
- i. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.

## ADMINISTRATIVE CONTROLS

- d. Records of radiation exposure for all individuals entering radiation control areas.
- e. Records of gaseous and liquid radioactive material released to the environs.
- f. Records of transient or operational cycles for those facility components identified in Table 5.7-1.
- g. Records of training and qualification for current members of the plant staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities identified in the NRC approved QA Manual as lifetime records.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the POSRC, the Procedure Review Committee, and the OSSRC.
- l. Records of Environmental Qualification which are covered under the provisions of paragraph 6.13.
- m. Records of the service lives of all safety related snubbers including the date at which the service life commences and associated installation and maintenance records.

### 6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

### 6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20:

- a. A high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Special or Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.

## ADMINISTRATIVE CONTROLS

- b. A high radiation area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.12.1.a, above, and in addition locked barricades shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained by the Supervisor-Radiation Control Operations and the Operations Shift Supervisor on duty under their separate administrative control

### 6.13 SYSTEM INTEGRITY

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and
2. Leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

### 6.14 IODINE MONITORING

The licensee shall implement a program\* which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

1. Training of personnel,
2. Procedures for monitoring, and
3. Provisions for maintenance of sampling and analysis equipment.

### 6.15 POST ACCIDENT SAMPLING

The licensee shall establish, implement and maintain a program\* which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

1. Training of personnel,
2. Procedures for sampling and analysis, and
3. Provisions for maintenance of sampling and analysis equipment.

\* It is acceptable if the licensee maintains details of the program in plant operation manuals (e.g., chemistry procedures, training instructions, maintenance procedures, ERIPs).

## ADMINISTRATIVE CONTROLS

### 6.16 PROCESS CONTROL PROGRAM (PCP)

6.16.1 The PCP shall be approved by the Commission prior to implementation.

6.16.2 Licensee initiated changes to the PCP:

1. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made. This submittal shall contain:
  - a. An evaluation supporting the premise that the change did not reduce the overall conformance of the solidified waste product to existing criteria for solid wastes; and
  - b. A reference to the date and the POSRC meeting number in which the change(s) were reviewed and found acceptable to the POSRC.
2. Shall become effective upon review by the POSRC and approval of the Plant General Manager.

### 6.17 OFFSITE DOSE CALCULATION MANUAL (ODCM)

6.17.1 The ODCM shall be approved by the Commission prior to implementation.

6.17.2 Licensee initiated changes to the ODCM:

1. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made effective. This submittal shall contain:
  - a. Sufficient information to support the rationale for the change. Information submitted should consist of a package of those pages of the ODCM to be changed with each page numbered and provided with a change number and/or change date together with appropriate analyses or evaluations justifying the change(s);
  - b. A determination that the change will not reduce the accuracy or reliability of dose calculations or setpoint determinations; and
  - c. Documentation of the fact that the change has been reviewed and found acceptable by the POSRC.
2. Shall become effective upon review by the POSRC and approval of the Plant General Manager.

## ADMINISTRATIVE CONTROLS

### 6.1 RESPONSIBILITY

6.1.1 The Plant General Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

### 6.2 ORGANIZATION

#### 6.2.1 ONSITE & OFFSITE ORGANIZATIONS

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in FSAR Chapter 12, and updated in accordance with 10 CFR 50.71(e).
- b. The Plant General Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Vice President-Nuclear Energy shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

#### 6.2.2 UNIT STAFF

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.

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## ADMINISTRATIVE CONTROLS

- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown, and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All **CORE ALTERATIONS** after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of both units (4 members) or any personnel required for other essential functions during a fire emergency.
- g. The Superintendent-Nuclear Operations shall hold or have held a senior reactor operator license at Calvert Cliffs. The General Supervisor-Nuclear Plant Operations, Shift Supervisor and Control Room Supervisor shall hold a senior reactor operator license. The Control Room Operator shall hold a reactor operator license.

## ADMINISTRATIVE CONTROLS

### 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the Radiation Safety Engineer who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and (2) the Shift Technical Advisor who shall have a Bachelor's Degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

### 6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the General Supervisor-Nuclear Training and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55, as applicable.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the General Supervisor-Planning and Support and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975.

### 6.5 REVIEW AND AUDIT

#### 6.5.1 PLANT OPERATIONS AND SAFETY REVIEW COMMITTEE (POSRC)

##### FUNCTION

6.5.1.1 The POSRC shall function to advise the Plant General Manager on all matters related to nuclear safety.

##### COMPOSITION

6.5.1.2 The POSRC shall be composed of at least seven, but no more than ten, members, including the Chairman. Members shall collectively have experience in the following areas:

- Nuclear Operations
- Electrical and Controls Maintenance
- Chemistry
- Mechanical Maintenance
- Nuclear Engineering
- Radiation Safety
- Plant Engineering
- Design Engineering

Members shall be appointed in writing by the Plant General Manager. Members shall have a minimum of eight years power plant experience of which a minimum of three years shall be nuclear power experience. At least one member shall have an SRO license on Calvert Cliffs Units 1 and 2

## ADMINISTRATIVE CONTROLS

### CHAIRMAN

6.5.1.3 The Chairman and alternate Chairmen of the POSRC shall be appointed in writing by the Plant General Manager. Chairmen shall have a minimum of 10 years power plant experience of which a minimum of three years shall be nuclear power experience.

### ALTERNATES

6.5.1.4 All alternate members shall be appointed in writing by the Plant General Manager. Alternate members shall have a minimum of eight years power plant experience of which a minimum of three years shall be nuclear power experience.

### MEETING FREQUENCY

6.5.1.5 The POSRC shall meet at least once per calendar month and as convened by the POSRC Chairman or one of the designated alternates.

### QUORUM

6.5.1.6 A quorum of the POSRC shall include the Chairman or one of the designated alternate chairmen and shall consist of a majority of the members, including alternates. No more than half of the quorum shall be alternates, including an alternate chairman.

### RESPONSIBILITIES

6.5.1.7 The POSRC shall be responsible for the following except for those items designated for review by the Procedure Review Committee or Qualified Reviewer in accordance with Specification 6.5.2 and 6.5.3, respectively:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, and 2) any other proposed procedures or changes thereto as determined by the Plant General Manager to affect nuclear safety.

Cross-disciplinary reviews of these procedures are conducted in accordance with administrative procedures in addition to the reviews conducted by POSRC, the Procedure Review Committee, or Qualified Reviewer.

- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.

## ADMINISTRATIVE CONTROLS

- e. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Off-Site Safety Review Committee.
- f. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Off-Site Safety Review Committee.
- g. Review of changes to the **PROCESS CONTROL PROGRAM** and the **OFFSITE DOSE CALCULATION MANUAL**.
- h. Review of all 10 CFR 50.59 Safety Evaluations that support procedures in 6.5.1.7.a and changes or modifications in 6.5.1.7.d.
- i. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Plant General Manager, the Vice President Nuclear Energy and to the Chairman of the Off-Site Safety Review Committee.
- j. Review of all **REPORTABLE EVENTS**.
- k. Review of facility operations to detect potential safety hazards.
- l. Review of any accidental, unplanned or uncontrolled radioactive release that exceeds 25% of the limits of Specification 3.11.1.2, 3.11.2.2 or 3.11.2.3, including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and for forwarding of these reports to the Plant General Manager and the Off-Site Safety Review Committee.
- m. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Off-Site Safety Review Committee.

## ADMINISTRATIVE CONTROLS

### AUTHORITY

- 6.5.1.8 The Plant Operations and Safety Review Committee shall:
- a. Recommend to the approval authority approval or disapproval of procedures considered under 6.5.1.7.a.
  - b. Recommend to the Plant General Manager approval or disapproval of items considered under 6.5.1.7(b) through (h) above.
  - c. Render determinations in writing with regard to whether or not each item considered under 6.5.1.7(a) through (h) above constitutes an unreviewed safety question.
  - d. Evaluate root causes and recommended actions to prevent recurrence for items considered under 6.5.1.7(i) through (l).
  - e. Provide written notification within 24 hours to the Vice President-Nuclear Energy and the Chairman of the Off-Site Safety Review Committee of disagreement between the POSRC and the responsible approval authority in the case of item 6.5.1.7.a or between the POSRC and the Plant General Manager; however, the Plant General Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

### RECORDS

6.5.1.9 The POSRC shall maintain written minutes of each meeting and copies shall be provided to the Vice President-Nuclear Energy, Chairman of the Off-Site Safety Review Committee, and the Plant General Manager.

## 6.5.2 PROCEDURE REVIEW COMMITTEE

### FUNCTION

6.5.2.1 The Procedure Review Committee may function to review items listed in Specification 6.5.1.7(a) in lieu of review by POSRC or Qualified Reviewer as directed by the Plant General Manager.

### COMPOSITION

6.5.2.2 The Procedure Review Committee shall be composed of a Chairman and eight individuals who shall collectively have expertise in the areas contained in Technical Specification 6.5.1.2.

## ADMINISTRATIVE CONTROLS

Members shall be appointed in writing by the Plant General Manager. Members shall have a minimum of eight years power plant experience of which a minimum of three years shall be nuclear power experience. At least one member shall be a POSRC member or alternate. The charter for the Procedure Review Committee shall include a description of membership, qualifications, functions, and reports and shall be described in plant administrative procedures. The Procedure Review Committee may be dissolved at the discretion of the Plant General Manager.

### CHAIRMAN

6.5.2.3 The Chairman and alternate Chairmen of the Procedure Review Committee shall be appointed in writing by the Plant General Manager. Chairmen shall have a minimum of eight years power plant experience of which a minimum of three years shall be nuclear power experience.

### ALTERNATES

6.5.2.4 All alternate members shall be appointed in writing by the Plant General Manager. Alternate members shall have a minimum of eight years power plant experience of which a minimum of three years shall be nuclear power experience.

### MEETING FREQUENCY

6.5.2.5 The Procedure Review Committee shall meet at least once per calendar month and as convened by the Chairman or his designated alternates.

### QUORUM

6.5.2.6 A quorum for the Procedure Review Committee shall consist of the Chairman or one of the designated alternate Chairmen and three primary or alternate members provided at least four disciplines are represented.

### AUTHORITY

6.5.2.7 The Procedure Review Committee shall:

- a. Recommend to the Approval Authority approval or disapproval of procedures considered under 6.5.1.7(a).
- b. Render determinations in writing with regard to whether or not each procedure under 6.5.1.7(a) constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President-Nuclear Energy and the Chairman of the Off-Site Safety Review Committee of disagreements between the Procedure Review Committee and the responsible approval authority. The Plant General Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

## ADMINISTRATIVE CONTROLS

### RECORDS

6.5.2.8 The Procedure Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Plant General Manager.

### 6.5.3 QUALIFIED REVIEWERS

#### FUNCTION

6.5.3.1 The Plant General Manager may designate specific procedures or classes of procedures described in Specification 6.5.1.7.a to be reviewed by Qualified Reviewers in lieu of review by POSRC or the Procedure Review Committee.

#### AUTHORITY

6.5.3.2 Qualified Reviewers shall:

- a. Recommend to the approval authority approval or disapproval of designated procedures and changes considered under 6.5.1.7.a, and
- b. Render determination in writing with regard to whether or not each procedure under 6.5.1.7.a constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President-Nuclear Energy and the Chairman of the Off-Site Safety Review Committee of disagreements between the Qualified Reviewer and the approval authority. The Plant General Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1. above.

#### CERTIFICATION

6.5.3.3 Qualified Reviewers shall be nominated, trained, and certified in accordance with administrative procedures. Certification shall be by a department manager.

6.5.3.4 Certification requirements of personnel designated as Qualified Reviewers shall be in accordance with administrative procedures.

## ADMINISTRATIVE CONTROLS

Qualified Reviewers shall have:

- a. A Bachelors degree in engineering, related science, or technical discipline, and two years of nuclear power plant experience;

OR

- b. Six years of nuclear power plant experience

OR

- c. Equivalent combination of education and experience as approved by a Department Manager.

## RECORDS

6.5.3.5 Review of procedures by Qualified Reviewers shall be documented in accordance with administrative procedures.

## 6.5.4 OFF-SITE SAFETY REVIEW COMMITTEE (OSSRC)

### FUNCTION

6.5.4.1 The Off-Site Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy and non-destructive examination
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

## ADMINISTRATIVE CONTROLS

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### COMPOSITION

6.5.4.2 The OSSRC shall be composed of at least seven members, including the Chairman. Members of the OSSRC may be from the Nuclear Energy Division or other BG&E organization or from organizations external to BG&E.

### QUALIFICATIONS

6.5.4.3 The Chairman, members and alternate members of the OSSRC shall be appointed in writing by the Vice President-Nuclear Energy and shall have an academic degree in engineering or a physical science, or the equivalent, and in addition shall have a minimum of five years technical experience in one or more areas given in 6.5.4.1. No more than two alternates shall participate as voting members in OSSRC activities at any one time.

### CONSULTANTS

6.5.4.4 Consultants shall be utilized as determined by the OSSRC Chairman to provide expert advice to the OSSRC.

### MEETING FREQUENCY

6.5.4.5 The OSSRC shall meet at least once per six months.

### QUORUM

6.5.4.6 The quorum of the OSSRC necessary for the performance of the OSSRC review and audit functions of these Technical Specifications shall consist of more than half the OSSRC membership or at least four members, whichever is greater. This quorum shall include the Chairman or his appointed alternate and the OSSRC members, including appointed alternates, meeting the requirements of Specification 6.5.4.3. No more than a minority of the quorum shall have line responsibility for operation of the plant.

## ADMINISTRATIVE CONTROLS

### REVIEW

#### 6.5.4.7 The OSSRC shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provisions of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes in Technical Specifications or this Operating License.
- e. Violation of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. **All REPORTABLE EVENTS.**
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meetings minutes of the POSRC.

## ADMINISTRATIVE CONTROLS

### AUDITS

6.5.4.8.1 Audits of facility activities shall be performed under the cognizance of the OSSRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualification of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR Part 50, at least once per 24 months.
- e. Deleted.
- f. The Safeguards Contingency Plan and implementing procedures at least once per 12 months in accordance with 10 CFR 73.40(d).
- g. Any other area of facility operation considered appropriate by the OSSRC or the Vice President-Nuclear Energy.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.
- k. The radiological environmental monitoring program and the results thereof at least once per 12 months.
- l. The **OFFSITE DOSE CALCULATION MANUAL** and implementing procedures at least once per 24 months.
- m. The **PROCESS CONTROL PROGRAM** and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months.

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- n. The performance of activities required by The Quality Assurance Program for effluent and environmental monitoring at least once per 12 months.

6.5.4.8.2 Review of facility activities shall be performed under the cognizance of the OSSRC. These reviews shall encompass:

- a. The Facility Emergency Plan and implementing procedures at least once per 12 months in accordance with 10 CFR Part 50.54(t).

### AUTHORITY

6.5.4.9 The OSSRC shall report to and advise the Vice President-Nuclear Energy on those areas of responsibility specified in Section 6.5.4.7 and 6.5.4.8.

### RECORDS

6.5.4.10 Records of OSSRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each OSSRC meeting shall be prepared, approved and forwarded to the Vice President-Nuclear Energy within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.4.7 above, shall be prepared, approved and forwarded to the Vice President-Nuclear Energy within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.4.8 above, shall be forwarded to the Vice President-Nuclear Energy and to the management positions responsible for the areas audited within 30 days after completion of the audit.

## 6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the POSRC and the results of this review shall be submitted to the OSSRC and the Vice President-Nuclear Energy.

## ADMINISTRATIVE CONTROLS

### 6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least **HOT STANDBY** within one hour.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Vice President-Nuclear Energy and the OSSRC shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the POSRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the OSSRC and the Vice President-Nuclear Energy within 14 days of the violation.

### 6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.\*
- g. The amount of overtime worked by plant staff members performing safety related functions must be limited in accordance with the NRC Policy Statement on working Hours (Generic Letter No. 82-12).

\* POSRC is only required to review Fire Protection procedures and changes thereto which affect nuclear safety.

## ADMINISTRATIVE CONTROLS

- h. PROCESS CONTROL PROGRAM implementation.
- i. OFFSITE DOSE CALCULATION MANUAL implementation.

6.8.2 The Plant General Manager may designate specific procedures or classes of procedures in writing to be reviewed by the Procedure Review Committee or by Qualified Reviewers in lieu of review by the POSRC. Review by the Procedure Review Committee shall be in accordance with paragraph 6.5.2. Review by Qualified Reviewers shall be in accordance with paragraph 6.5.3.

6.8.3 Procedures listed in 6.8.1 shall be approved by the Plant General Manager or by cognizant Managers, Superintendents, or General Supervisors (or Directors) that report directly to a Manager prior to implementation as specified by administrative requirements. The approval authority for specific procedures or classes of procedures shall be designated in writing by the Plant General Manager and shall be a different individual from the Qualified Reviewer.

6.8.4 Each procedure of 6.8.1 above and changes thereto shall be reviewed periodically as set forth in administrative procedures.

## ADMINISTRATIVE CONTROLS

6.8.5 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the POSRC, the Procedure Review Committee, or by a Qualified Reviewer and approved by the designated approval authority within 14 days of implementation.

## 6.9 REPORTING REQUIREMENTS

### ROUTINE REPORTS

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

### STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

6.9.1.2 The startup report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

## ADMINISTRATIVE CONTROLS

6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

### ANNUAL REPORTS<sup>1/</sup>

6.9.1.4 Annual reports covering the activities of the unit as described below for the previous calendar year shall be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

6.9.1.5 Reports required on an annual basis shall include:

- a. A tabulation on an annual basis of the number of station, utility, and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions, <sup>2/</sup> e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
- b. The complete results of steam generator tube inservice inspections performed during the report period (reference Specification 4.4.5.5.b).
- c. Documentation of all failures and challenges to the pressurizer PORVs or safety valves.

<sup>1/</sup> A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

<sup>2/</sup> This tabulation supplements the requirements of 10 CFR Part 20.407.

## ADMINISTRATIVE CONTROLS

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### 6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. All REPORTABLE EVENTS.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to Operating Procedures.
- g. Records of radioactive shipments.
- h. Records of sealed source and fission detector leak tests and results.
- i. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.

## ADMINISTRATIVE CONTROLS

- d. Records of radiation exposure for all individuals entering radiation control areas.
- e. Records of gaseous and liquid radioactive material released to the environs.
- f. Records of transient or operational cycles for those facility components identified in Table 5.7.1.
- g. Records of training and qualification for current members of the plant staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities identified in the NRC approved QA Manual as lifetime records.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the POSRC, the Procedure Review Committee, and the OSSRC.
- l. Records of Environmental Qualification which are covered under the provisions of paragraph 6.13.
- m. Records of the service lives of all safety related snubbers including the date at which the service life commences and associated installation and maintenance records.

### 6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

### 6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20:

- a. A high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Special or Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.

## ADMINISTRATIVE CONTROLS

- b. A high radiation area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.12.1.a, above, and in addition locked barricades shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained by the Supervisor-Radiation Control Operations and the Operations Shift Supervisor on duty under their separate administrative control

### 6.13 SYSTEM INTEGRITY

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and
2. Leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

### 6.14 IODINE MONITORING

The licensee shall implement a program\* which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

1. Training of personnel,
2. Procedures for monitoring, and
3. Provisions for maintenance of sampling and analysis equipment.

### 6.15 POST ACCIDENT SAMPLING

The licensee shall establish, implement and maintain a program\* which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

1. Training of personnel,
2. Procedures for sampling and analysis, and
3. Provisions for maintenance of sampling and analysis equipment.

\* It is acceptable if the licensee maintains details of the program in plant operation manuals (e.g., chemistry procedures, training instructions, maintenance procedures, ERPIPs).

## ADMINISTRATIVE CONTROLS

### 6.16 PROCESS CONTROL PROGRAM (PCP)

6.16.1 The PCP shall be approved by the Commission prior to implementation.

6.16.2 Licensee initiated changes to the PCP:

1. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made. This submittal shall contain:
  - a. An evaluation supporting the premise that the change did not reduce the overall conformance of the solidified waste product to existing criteria for solid wastes; and
  - b. A reference to the date and the POSRC meeting number in which the change(s) were reviewed and found acceptable to the POSRC.
2. Shall become effective upon review by the POSRC and approval of the Plant General Manager.

### 6.17 OFFSITE DOSE CALCULATION MANUAL (ODCM)

6.17.1 The ODCM shall be approved by the Commission prior to implementation.

6.17.2 Licensee initiated changes to the ODCM:

1. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made effective. This submittal shall contain:
  - a. Sufficient information to support the rationale for the change. Information submitted should consist of a package of those pages of the ODCM to be changed with each page numbered and provided with a change number and/or change date together with appropriate analyses or evaluations justifying the change(s);
  - b. A determination that the change will not reduce the accuracy or reliability of dose calculations or setpoint determinations; and
  - c. Documentation of the fact that the change has been reviewed and found acceptable by the POSRC.
2. Shall become effective upon review by the POSRC and approval of the Plant General Manager.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

SAFETY-EVALUATION-BY-THE-OFFICE-OF-NUCLEAR-REACTOR-REGULATION  
RELATED-TO-AMENDMENT-NO. 161-TO-FACILITY-OPERATING-LICENSE-NO. DPR-53  
AND-AMENDMENT-NO. 141-TO-FACILITY-OPERATING-LICENSE-NO. DPR-69  
BALTIMORE-GAS-AND-ELECTRIC-COMPANY  
CALVERT-CLIFFS-NUCLEAR-POWER-PLANT, UNIT NOS. 1-AND-2  
DOCKET NOS. 50-317-AND-50-318

1.0 INTRODUCTION

By letter dated August 23, 1990, as supplemented on September 28, 1990, and June 28, 1991, the Baltimore Gas and Electric Company (the licensee) submitted a request for changes to the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2, Technical Specifications (TS). The requested changes would revise and add sections to the Administrative Controls Section of the TS for both units. In addition, numerous editorial changes have been made in relation to titles, section numbers, and other formatting to be consistent with the proposed revisions and new requirements; such as, the use of committees or qualified individuals to perform selected reviews. The June 28, 1991, letter provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

The BG&E request can be divided into four areas which include: Title changes; revisions to the Plant Operation and Safety Review Committee (POSRC); the inclusion of a new Procedure Review Committee and the use of Qualified Reviewers; and editorial and administrative changes. The following provides a brief description of the request and our evaluation.

2.1 Title changes

The title of Manager-Calvert Cliffs Nuclear Power Plant has been replaced by the title of Plant General Manager throughout Section 6, and the title of General Supervisor-Quality Control and Support has been replaced by the title of General Supervisor-Planning and Support in Section 6.4.2.

These changes in title reflect the most recent organizational changes and the positions maintain a comparable level of responsibility to that of the previous title and are, therefore, acceptable.

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## 2.2 POSRC Revisions

Section 6.5.1, POSRC, has been extensively revised. These revisions include:

a. The composition has been revised to be at least seven, but no more than ten members. The titles of the members have been deleted and replaced by a statement as to the areas of expertise covered and a statement that each member, and alternates, shall have a minimum of eight years power plant experience of which a minimum of three years shall be nuclear power experience. In addition, at least one member shall have an Senior Reactor Operator (SRO) license for Calvert Cliffs Units 1 and 2. The Chairman shall have a minimum of ten years power plant experience of which a minimum of three years shall be nuclear power experience.

b. The quorum requirement has been changed from a fixed number to state that it shall consist of a majority of the members, including alternates, and that no more than half the quorum shall be alternates.

c. The responsibilities of the POSRC, Section 6.5.1.7, have been revised to allow for the review of procedures by a procedure review committee or by a qualified reviewer. Additionally, a new Subsection, 6.5.1.7h, has been added that requires the review of all 10 CFR 50.59 Safety Evaluations that support procedures in TS 6.5.1.7.a and changes or modifications in TS 6.5.1.7.d.

d. TS Section 6.5.1.8, formerly Section 6.5.1.7, has been revised to delete the requirement that the POSRC recommend approval of procedures to the Plant General Manager and replace it with a requirement that the POSRC recommend approval or disapproval to the approval authority; and to require that the POSRC evaluate root causes and recommended actions to prevent recurrence for items covered under Section 6.5.1.7i through l.

The revisions to the POSRC, as described above, meet the applicable acceptance criteria of NUREG 0800, "The Standard Review Plan," Section 13.4, "Operational Review," and are, therefore, acceptable.

## 2.3 Procedures Review Committee and Qualified Reviewers

Several portions of TS Sections 6.5 and 6.8 have been relabeled, revised, or new sections added to address the use of a procedure review committee or the use of qualified reviewers. These changes include:

a. TS Section 6.5.2 - Procedure Review Committee - A new committee has been established that may function to review procedures listed in TS 6.5.1.7a as directed by the Plant General Manager. The committee shall be composed of a Chairman and eight individuals who shall have a minimum of eight years power plant experience of which a minimum of three years shall be nuclear power plant experience. The Procedures Review Committee shall meet at least once per

calendar month and as convened by the Chairman or his designated alternates. A quorum shall consist of the Chairman or one of the designated alternate Chairman and three primary or alternate members provided at least four disciplines are represented.

b. TS Section 6.5.3 - Qualified Reviewers - A new provision has been established that may function to review specific procedures or classes of procedures of Specification 6.5.1.7a as designated by the Plant General Manager in lieu of review by the POSRC or the Procedure Review Committee. A qualified reviewer shall have a Bachelor's degree in engineering, or related science or technical discipline, and two years of nuclear power experience, or six years of nuclear power experience, or the equivalent combination of education and experience. Qualified reviewers shall be certified by a Department Manager. Cross-disciplinary reviews of these procedures are conducted in accordance with administrative procedures. The qualified reviewer makes a determination with regard to whether or not each procedure under TS 6.5.1.7a constitutes an unreviewed safety question.

c. TS Section 6.8.2 - This section has been revised to require the Plant General Manager to designate those specific procedures or classes of procedures to be reviewed by the Procedures Review Committee or by a qualified reviewer.

d. Section 6.8.3 - This section has been relabelled 6.8.4 and a new Section 6.8.3 established. The new TS, Section 6.8.3, states that procedures listed in TS Section 6.8.1, shall be approved by the Plant General Manager or by cognizant Managers, Superintendents, or General Supervisors (or Directors) that report directly to the Plant General Manager prior to implementation as specified by administrative procedures.

e. TS Section 6.8.5, formerly Section 6.8.4, has been revised to allow for the review of temporary changes to procedures of TS 6.8.1 by the Procedures Review Committee, or by a qualified reviewer, in addition to the POSRC.

The use of the Procedure Review Committee and qualified reviewers provides a means for efficient operation of the POSRC. These changes will allow for a more extensive and inter-disciplinary review of procedures. The changes, as detail above, are an acceptable means for procedure review and are, therefore, acceptable.

#### 2.4 Editorial and administrative changes

The numerous editorial and administrative changes are necessary to support the revisions and new provisions to the TS previously discussed. We have determined that they are appropriate and acceptable.

#### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Maryland State official was notified of the proposed issuance of the amendments. The State official had no comments.

#### 4.0 ENVIRONMENTAL CONSIDERATION

The amendments involve only changes in administrative procedure and requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:  
F. Allenspach

Date: September 11, 1991

September 11, 1991

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:

Daniel G. McDonald, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 161 to DPR-53
2. Amendment No. 141 to DPR-69
3. Safety Evaluation

cc w/enclosures:

See next page

OFC	:PDI-1:LA	:PDI-1:PM	:OGC	:PDI-1:D
NAME	:CVogan	:McDonald:avl	:RACapra	
DATE	:9/29/91	:9/29/91	:9/13/91	:9/11/91

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