

February 19, 1991

Docket Nos. 50-317
and 50-318

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Mr. G. C. Creel
Vice President - Nuclear Energy
Baltimore Gas and Electric Company
Calvert Cliffs Nuclear Power Plant
MD Rts 2 & 4
P. O. Box 1535
Lusby, Maryland 20657

Dear Mr. Creel:

SUBJECT: CORRECTIONS TO AMENDMENTS NO. 149 AND NO. 130 FOR CALVERT CLIFFS
UNITS 1 AND 2, RESPECTIVELY (TAC NOS. 72075 AND 72076)

By letter dated April 16, 1990, the Commission issued Amendment No. 149 to Facility Operating License No. DPR-53 and Amendment No. 130 to Facility Operating License No. DPR-69 for the Calvert Cliffs Power Plant, Unit Nos. 1 and 2, respectively.

Your letter dated February 1, 1991, noted that two footnotes associated with Table 3.3-5 were inadvertently deleted from the second page when your staff prepared the final Technical Specification pages for the requested amendments identified above. Although these footnotes were not required for the Auxiliary Feedwater (AFW) Pumps, which were the subject of the amendment request, they are applicable to other components on the first (unchanged) page of the table.

We have verified that the correction is consistent with the amendments and their supporting safety evaluation. As was noted in your letter, this was clearly an administrative error. Enclosed are the corrected pages 3/4-21 for Facility Operating License Nos. DPR-53 and DPR-69. Please replace the existing pages with the corrected pages.

Sincerely,

Original Signed By:

Daniel G. McDonald, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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Enclosure:
As stated

cc w/enclosure:
See next page

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NAME	:CVogan <i>w</i>	:DMcDonald:cs	:RACapra <i>rac</i>	:	:
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Mr. G. C. Creel
Baltimore Gas & Electric Company

Calvert Cliffs Nuclear Power Plant

cc:

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TABLE 3.3-5 (Continued)

ENGINEERED SAFETY FEATURES RESPONSE TIMES

<u>INITIATING SIGNAL AND FUNCTION</u>	<u>RESPONSE TIME IN SECONDS</u>
6. <u>Steam Generator Pressure-Low</u>	
a. Main Steam Isolation	≤ 6.9
b. Feedwater Isolation	≤ 80
7. <u>Refueling Water Tank-Low</u>	
a. Containment Sump Recirculation	≤ 80
8. <u>Reactor Trip</u>	
a. Feedwater Flow Reduction to 5%	≤ 20
9. <u>Loss of Power</u>	
a. 4.16 kv Emergency Bus Under-voltage (Loss of Voltage)	≤ 2.2***
b. 4.16 kv Emergency Bus Under-voltage (Degraded Voltage)	≤ 8.4***
10. <u>Steam Generator Level-Low</u>	
a. Steam Driven AFW Pump	≤ 180
b. Motor Driven AFW Pump	≤ 180
11. <u>Steam Generator ΔP-High</u>	
a. Auxiliary Feedwater Isolation	≤ 20.0

TABLE NOTATION

* Diesel generator starting and sequence loading delays included.

** Diesel generator starting and sequence loading delays not included. Offsite power available.

*** Response time measured from the incidence of the undervoltage condition to the diesel generator start signal.

(1) Header fill time not included.

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TABLE 3.3-5 (Continued)

ENGINEERED SAFETY FEATURES RESPONSE TIMES

<u>INITIATING SIGNAL AND FUNCTION</u>	<u>RESPONSE TIME IN SECONDS</u>
6. <u>Steam Generator Pressure-Low</u>	
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8. <u>Reactor Trip</u>	
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9. <u>Loss of Power</u>	
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a. Motor Driven AFW Pump	≤ 180
b. Steam Driven AFW Pump	≤ 180
11. <u>Steam Generator ΔP-High</u>	
a. Auxiliary Feedwater Isolation	≤ 20.0

TABLE NOTATION

* Diesel generator starting and sequence loading delays included.

** Diesel generator starting and sequence loading delays not included. Offsite power available.

*** Response time measured from the incidence of the undervoltage condition to the diesel generator start signal.