

February 19, 1991

Docket Nos. 50-317
and 50-318

Mr. G. C. Creel
Vice President - Nuclear Energy
Baltimore Gas and Electric Company
Calvert Cliffs Nuclear Power Plant
MD Rts 2 & 4
P. O. Box 1535
Lusby, Maryland 20657

Distribution	
Docket File	NRC & Local PDRs
PDI-1 Reading	SVarga
EGreenman	CVogan
DMcDonald	OGC
DHagan	GHill (8)
Wanda Jones	JCalvo
ACRS (10)	GPA/PA
OC/LFMB	Plant File
RACapra	JLinville

Dear Mr. Creel:

SUBJECT: CORRECTIONS TO AMENDMENTS NO. 149 AND NO. 130 FOR CALVERT CLIFFS
UNITS 1 AND 2, RESPECTIVELY (TAC NOS. 72075 AND 72076)

By letter dated April 16, 1990, the Commission issued Amendment No. 149 to Facility Operating License No. DPR-53 and Amendment No. 130 to Facility Operating License No. DPR-69 for the Calvert Cliffs Power Plant, Unit Nos. 1 and 2, respectively.

Your letter dated February 1, 1991, noted that two footnotes associated with Table 3.3-5 were inadvertently deleted from the second page when your staff prepared the final Technical Specification pages for the requested amendments identified above. Although these footnotes were not required for the Auxiliary Feedwater (AFW) Pumps, which were the subject of the amendment request, they are applicable to other components on the first (unchanged) page of the table.

We have verified that the correction is consistent with the amendments and their supporting safety evaluation. As was noted in your letter, this was clearly an administrative error. Enclosed are the corrected pages 3/4-21 for Facility Operating License Nos. DPR-53 and DPR-69. Please replace the existing pages with the corrected pages.

Sincerely,

Original Signed By:

Daniel G. McDonald, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

9102270340 910219
PDR ADDOCK 05000317
P PDR

Enclosure:
As stated

cc w/enclosure:
See next page

OFC	:PDI-1:LA	:PDI-1:APM	:PDI-1:D	:	:
NAME	:CVogan	:DMcDonald:cs	:RACapra	:	:
DATE	:2/19/91	:2/19/91	:2/19/91	:	:

OFFICIAL RECORD COPY

Document Name: CC 1/2 TAC NOS. 72075/72076

260010
JFOL
1/1

Mr. G. C. Creel
Baltimore Gas & Electric Company

Calvert Cliffs Nuclear Power Plant

cc:

Mr. William T. Bowen, President
Calvert County Board of
Commissioners
Prince Frederick, Maryland 20678

Mr. Joseph H. Walter
Engineering Division
Public Service Commission of Maryland
American Building
231 E. Baltimore Street
Baltimore, Maryland 21202-3486

D. A. Brune, Esq.
General Counsel
Baltimore Gas and Electric Company
P. O. Box 1475
Baltimore, Maryland 21203

Ms. Kirsten A. Burger, Esq.
Maryland People's Counsel
American Building, 9th Floor
231 E. Baltimore Street
Baltimore, Maryland 21202

Mr. Jay E. Silberg, Esq.
Shaw, Pittman, Potts and Trowbridge
2300 N Street, NW
Washington, DC 20037

Ms. Patricia Birnie
Co-Director
Maryland Safe Energy Coalition
P. O. Box 902
Columbia, Maryland 21044

Ms. G. L. Adams, Licensing
Calvert Cliffs Nuclear Power Plant
MD Rts 2 & 4,
P. O. Box 1535
Lusby, Maryland 20657

Resident Inspector
c/o U.S. Nuclear Regulatory Commission
P. O. Box 437
Lusby, Maryland 20657

Mr. Richard McLean
Administrator - Radioecology
Department of Natural Resources
580 Taylor Avenue
Tawes State Office Building
PPER B3
Annapolis, Maryland 21401

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

TABLE 3.3-5 (Continued)

ENGINEERED SAFETY FEATURES RESPONSE TIMES

<u>INITIATING SIGNAL AND FUNCTION</u>	<u>RESPONSE TIME IN SECONDS</u>
6. <u>Steam Generator Pressure-Low</u>	
a. Main Steam Isolation	≤ 6.9
b. Feedwater Isolation	≤ 80
7. <u>Refueling Water Tank-Low</u>	
a. Containment Sump Recirculation	≤ 80
8. <u>Reactor Trip</u>	
a. Feedwater Flow Reduction to 5%	≤ 20
9. <u>Loss of Power</u>	
a. 4.16 kv Emergency Bus Under-voltage (Loss of Voltage)	≤ 2.2***
b. 4.16 kv Emergency Bus Under-voltage (Degraded Voltage)	≤ 8.4***
10. <u>Steam Generator Level-Low</u>	
a. Steam Driven AFW Pump	≤ 180
b. Motor Driven AFW Pump	≤ 180
11. <u>Steam Generator ΔP-High</u>	
a. Auxiliary Feedwater Isolation	≤ 20.0

TABLE NOTATION

- * Diesel generator starting and sequence loading delays included.
- ** Diesel generator starting and sequence loading delays not included. Offsite power available.
- *** Response time measured from the incidence of the undervoltage condition to the diesel generator start signal.
- (1) Header fill time not included.

9102280067 910219
PDR ADCK 05000317
P PDR

TABLE 3.3-5 (Continued)

ENGINEERED SAFETY FEATURES RESPONSE TIMES

<u>INITIATING SIGNAL AND FUNCTION</u>	<u>RESPONSE TIME IN SECONDS</u>
6. <u>Steam Generator Pressure-Low</u>	
a. Main Steam Isolation	≤ 6.9
b. Feedwater Isolation	≤ 80
7. <u>Refueling Water Tank-Low</u>	
a. Containment Sump Recirculation	≤ 80
8. <u>Reactor Trip</u>	
a. Feedwater Flow Reduction to 5%	≤ 20
9. <u>Loss of Power</u>	
a. 4.16 kv Emergency Bus Under-voltage (Loss of Voltage)	$\leq 2.2^{***}$
b. 4.16 kv Emergency Bus Under-voltage (Degraded Voltage)	$\leq 8.4^{***}$
10. <u>Steam Generator Level - Low</u>	
a. Motor Driven AFW Pump	≤ 180
b. Steam Driven AFW Pump	≤ 180
11. <u>Steam Generator ΔP-High</u>	
a. Auxiliary Feedwater Isolation	≤ 20.0

TABLE NOTATION

- * Diesel generator starting and sequence loading delays included.
- ** Diesel generator starting and sequence loading delays not included. Offsite power available.
- *** Response time measured from the incidence of the undervoltage condition to the diesel generator start signal.