

June 30, 1986

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Docket Nos. 50-317
and 50-318

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Mr. J. A. Tiernan
Vice President - Nuclear Energy
Baltimore Gas & Electric Company
P. O. Box 1475
Baltimore, Maryland 21203

Dear Mr. Tiernan:

The Commission has issued the enclosed Amendment Nos. 119 and 101 to Facility Operating License Nos. DPR-53 and DPR-69 for Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated December 22, 1983 and October 25, 1985 as supplemented by your letter dated April 21, 1986. Issuance of these amendments completes NRC action on these applications.

The amendments change the Unit 1 and Unit 2 Technical Specification (TS) as follows: (1) a provision is added to TS 3.6.5.1, "Combustible Gas Control-Hydrogen Analyzers," to allow the changing of operational Modes with inoperable hydrogen analyzers (3.0.4 not applicable), (2) the reporting requirements of TS 4.4.5.5, "Reports-Steam Generators" are clarified, and (3) a new TS 3/4.4.13, "Reactor Coolant System Vents" is incorporated in the TS. In addition to the above, we have deleted the TS Table of Contents entry for "Secondary Water Chemistry", since this TS had been deleted previously.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

/S/

David H. Jaffe, Project Manager
PWR Project Directorate #8
Division of PWR Licensing-B

Enclosures:

1. Amendment No. 119 to DPR-53
2. Amendment No. 101 to DPR-69
3. Safety Evaluation

cc w/enclosure:
See next page

*See previous white for concurrences

PBD#8*	PBD#8	PBD#8*	RSB	FOB*	OELD
PMKreutzer	DJaffe;eh	ATHadani	CThomas	WRegan	<i>Woodward</i>
6/9/86	6/9/86	6/12/86	6/10/86	6/13/86	<i>6/20/86</i>

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Docket Nos. 50-317
and 50-318

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Mr. J. A. Tiernan
Vice President - Nuclear Energy
Baltimore Gas & Electric Company
P. O. Box 1475
Baltimore, Maryland 21203

Dear Mr. Tiernan:

The Commission has issued the enclosed Amendment Nos. and to Facility Operating License Nos. DPR-53 and DPR-69 for Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications in partial response to your applications dated December 22, 1983 and October 25, 1985 as supplemented by your letter dated April 21, 1986. Issuance of these amendments completes NRC action on these applications.

The amendments change the Unit 1 and Unit 2 Technical Specifications (TS) as follows: (1) a provision is added to TS 3.6.5.1, "Combustible Gas Control-Hydrogen Analyzers," to allow the changing of operational Modes with inoperable hydrogen analyzers (3.0.4 not applicable), (2) the reporting requirements of TS 4.4.5.5, "Reports-Steam Generators" are clarified, and (3) a new TS 3/4.4.13, "Reactor Coolant System Vents" is incorporated in the TS.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next bi-monthly Federal Register notice.

Sincerely,

/s/

David H. Jaffe, Project Manager
PWR Project Directorate #8
Division of PWR Licensing-B

Enclosures:

1. Amendment No. to DPR-53
2. Amendment No. to DPR-69
3. Safety Evaluation

cc w/enclosure:
See next page

*W. Regan / FOB
6/13/86*

"High Point Vents" Only

PBD#8
PMKreutzer
6/2/86

[Signature]
PBD#8
DJaffe
6/9/86

AT
PBD#8
ATHadani
6/12/86

[Signature]
RSB
CThomas
6/10/86

OELD
6/ /86

Mr. J. A. Tiernan
Baltimore Gas & Electric Company

Calvert Cliffs Nuclear Power Plant

cc:

Mr. William T. Bowen, President
Calvert County Board of
Commissioners
Prince Frederick, Maryland 20768

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
Office of Executive Director
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Department of Natural Resources
Energy Administration, Power Plant
Siting Program
ATTN: Mr. T. Magette
Tawes State Office Building
Annapolis, Maryland 21204



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.119
License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendments by Baltimore Gas & Electric Company (the licensee) dated December 22, 1983 and October 25, 1985 as supplemented by letter dated April 21, 1986 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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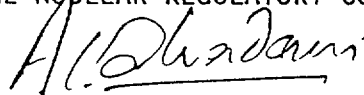
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-53 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 119, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Ashok C. Thadani, Director
PWR Project Directorate #8
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 30, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 119

FACILITY OPERATING LICENSE NO. DPR-53

DOCKET NO. 50-317

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are provided to maintain document completeness.

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3/4 6-26
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REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

5. Defect means an imperfection of such severity that it exceeds the plugging limit. A tube containing a defect is defective. Any tube which does not permit the passage of the eddy-current inspection probe shall be deemed a defective tube.
 6. Plugging Limit means the imperfection depth at or beyond which the tube shall be removed from service because it may become unserviceable prior to the next inspection and is equal to 40% of the nominal tube wall thickness.
 7. Unserviceable describes the condition of a tube if it leaks or contains a defect large enough to affect its structural integrity in the event of an Operating Basis Earthquake, a loss-of-coolant accident, or a steam line or feedwater line break as specified in 4.4.5.3.c, above.
 8. Tube Inspection means an inspection of the steam generator tube from the point of entry (hot leg side) completely around the U-bend to the top support of the cold leg.
- b. The steam generator shall be determined OPERABLE after completing the corresponding actions (plug all tubes exceeding the plugging limit and all tubes containing through-wall cracks) required by Table 4.4-2.

4.4.5.5 Reports

- a. Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the Commission within 15 days pursuant to Specification 6.9.2.
- b. The complete results of the steam generator tube inservice inspection shall be included in the Annual Operating Report for the period in which this inspection was completed (pursuant to Specification 6.9.1.5.b). This report shall include:
 1. Number and extent of tubes inspected.
 2. Location and percent of wall-thickness penetration for each indication of an imperfection.
 3. Identification of tubes plugged.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- c. Results of steam generator tube inspections which fall into Category C-3 require verbal notification of the NRC Regional Administrator by telephone within 24 hours prior to resumption of plant operation. The written followup of this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence and shall be submitted within the next 30 days pursuant to Specification 6.9.2.

REACTOR COOLANT SYSTEM

REACTOR COOLANT SYSTEM VENTS

LIMITING CONDITION FOR OPERATION

3.4.13 One reactor coolant system vent path consisting of two solenoid valves in series shall be OPERABLE and closed at each of the following locations:

- a. Reactor vessel head
- b. Pressurizer vapor space

APPLICABILITY: MODES 1 and 2

ACTION:

- a. With the reactor vessel head vent path inoperable, maintain the inoperable vent path closed with power removed from the actuator of the solenoid valves in the inoperable vent path, and:
 1. If the pressurizer vapor space vent path is also inoperable, restore both inoperable vent paths to OPERABLE status within 72 hours or be in at least HOT STANDBY within 6 hours, or
 2. If the pressurizer vapor space vent path is OPERABLE, restore the inoperable reactor vessel head vent path to OPERABLE status within 30 days or be in at least HOT STANDBY within 6 hours.
- b. With only the pressurizer vapor space vent path inoperable, maintain the inoperable vent path closed with power removed from the valve actuator of the solenoid valves in the inoperable vent path, and:
 1. Verify at least one PORV and its associated flow path is OPERABLE within 72 hours and restore the inoperable pressurizer vapor space vent path to OPERABLE status prior to entering MODE 2 following the next HOT SHUTDOWN of sufficient duration, or
 2. Restore the inoperable pressurizer vapor space vent path to OPERABLE status within 30 days, or be in at least HOT STANDBY within 6 hours.
- c. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.4.13.1 Each reactor coolant system vent path shall be demonstrated OPERABLE by testing each valve in the vent path per Specification 4.0.5.

REACTOR COOLANT SYSTEM

REACTOR COOLANT SYSTEM VENTS

SURVEILLANCE REQUIREMENTS (Continued)

4.4.13.2 Each reactor coolant system vent path shall be demonstrated OPERABLE at least once per 18 months by:

- a. Verifying all manual isolation valves in each vent path are locked in the open position.
- b. Verifying flow through the reactor coolant system vent paths with the vent valves open.

CONTAINMENT SYSTEMS

3/4.6.5 COMBUSTIBLE GAS CONTROL

HYDROGEN ANALYZERS

LIMITING CONDITION FOR OPERATION

3.6.5.1 Two independent containment hydrogen analyzers shall be OPERABLE.

APPLICABILITY: MODES 1 and 2.

ACTION:

- a. With one hydrogen analyzer inoperable, restore the inoperable analyzer to OPERABLE status within 30 days or:
 1. Verify containment atmosphere grab sampling capability and prepare and submit a special report to the Commission pursuant to Specification 6.9.2 within the following 30 days, outlining the ACTION taken, the cause for the inoperability, and the plans and schedule for restoring the system to OPERABLE status, or
 2. Be in at least HOT STANDBY within the next 6 hours.
- b. With both hydrogen analyzers inoperable, restore at least one inoperable analyzer to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours.
- c. Specification 3.0.4 is not applicable to this requirement.

SURVEILLANCE REQUIREMENTS

4.6.5.1 Each hydrogen analyzer shall be demonstrated OPERABLE at least bi-weekly on a STAGGERED TEST BASIS by drawing a sample from the waste gas system through the hydrogen analyzer.

4.6.5.2 Each hydrogen analyzer shall be demonstrated OPERABLE at least once per 92 days on a STAGGERED TEST BASIS by performing a CHANNEL CALIBRATION using sample gases in accordance with manufacturers' recommendations.

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>PENETRATION NO.</u>	<u>ISOLATION CHANNEL</u>	<u>ISOLATION VALVE IDENTIFICATION NO.</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SECONDS)</u>
61	NA	SFP-176	Refueling Pool Outlet	NA
	NA	SFP-174		NA
	NA	SFP-172		NA
	NA	SFP-189		NA
62	SIAS A	PH-6579-MOV	Containment Heating Outlet	≤ 13
64	NA	PH-376	Containment Heating Inlet	NA

(1) Manual or remote manual valve which is closed during plant operation.

(2) May be opened below 300°F to establish shutdown cooling flow.

(3) Containment purge valves will be shut in MODES 1, 2, 3, and 4 per TS 3/4 6.1.7.

* May be open on an intermittent basis under administrative control.

** Containment purge isolation valves isolation times will only apply in MODE 6 when the valves are required to be OPERABLE and they are open. Isolation time for containment purge isolation valves is NA for MODES 1, 2, 3 and 4 per TS 3/4 6.1.7, during which time these valves must remain closed.

(4) Containment vent isolation valves shall be opened for containment pressure control, airborne radioactivity control, and surveillance testing purposes only.

REACTOR COOLANT SYSTEM

BASES

3/4.4.13 REACTOR COOLANT SYSTEM VENTS

Reactor Coolant System Vents are provided to exhaust noncondensable gases and/or steam from the primary system that could inhibit natural circulation core cooling. The OPERABILITY of at least one reactor coolant system vent path from the reactor vessel head and the pressurizer vapor space ensures the capability exists to perform this function.

The valve redundancy of the reactor coolant system vent paths serves to minimize the probability of inadvertent or irreversible actuation while ensuring that a single failure of a vent valve, power supply or control system does not prevent isolation of the vent path.

The function, capabilities, and testing requirements of the reactor coolant system vent systems are consistent with the requirements of Item II.B.1 of NUREG-0737, "Clarification of TMI Action Plan Requirements," November 1980.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 101
License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendments by Baltimore Gas & Electric Company (the licensee) dated December 22, 1983 and October 1985 as supplemented by letter dated April 21, 1986 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

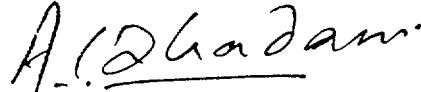
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-69 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 101, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Ashok C. Thadani, Director
PWR Project Directorate #8
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 30, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 101

FACILITY OPERATING LICENSE NO. DPR-69

DOCKET NO. 50-318

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are provided to maintain document completeness.

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REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

5. Defect means an imperfection of such severity that it exceeds the plugging limit. A tube containing a defect is defective. Any tube which does not permit the passage of the eddy-current inspection probe shall be deemed a defective tube.
6. Plugging Limit means the imperfection depth at or beyond which the tube shall be removed from service because it may become unserviceable prior to the next inspection and is equal to 40% of the nominal tube wall thickness.
7. Unserviceable describes the condition of a tube if it leaks or contains a defect large enough to affect its structural integrity in the event of an Operating Basis Earthquake, a loss-of-coolant accident, or a steam line or feedwater line break as specified in 4.4.5.3.c, above.
8. Tube Inspection means an inspection of the steam generator tube from the point of entry (hot leg side) completely around the U-bend to the top support of the cold leg.
 - b. The steam generator shall be determined OPERABLE after completing the corresponding actions (plug all tubes exceeding the plugging limit and all tubes containing through-wall cracks) required by Table 4.4-2.

4.4.5.5 Reports

- a. Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the Commission within 15 days pursuant to Specification 6.9.2.
- b. The complete results of the steam generator tube inservice inspection shall be included in the Annual Operating Report for the period in which this inspection was completed (pursuant to Specification 6.9.1.5.b). This report shall include:
 1. Number and extent of tubes inspected.
 2. Location and percent of wall-thickness penetration for each indication of an imperfection.
 3. Identification of tubes plugged.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- c. Results of steam generator tube inspections which fall into Category C-3 require verbal notification of the NRC Regional Administrator by telephone within 24 hours prior to resumption of plant operation. The written followup of this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence and shall be submitted within the next 30 days pursuant to Specification 6.9.2.

REACTOR COOLANT SYSTEM

REACTOR COOLANT SYSTEM VENTS

LIMITING CONDITION FOR OPERATION

3.4.13 One reactor coolant system vent path consisting of two solenoid valves in series shall be OPERABLE and closed at each of the following locations:

- a. Reactor vessel head
- b. Pressurizer vapor space

APPLICABILITY: MODES 1 and 2

ACTION:

- a. With the reactor vessel head vent path inoperable, maintain the inoperable vent path closed with power removed from the actuator of the solenoid valves in the inoperable vent path, and:
 1. If the pressurizer vapor space vent path is also inoperable, restore both inoperable vent paths to OPERABLE status within 72 hours or be in at least HOT STANDBY within 6 hours, or
 2. If the pressurizer vapor space vent path is OPERABLE, restore the inoperable reactor vessel head vent path to OPERABLE status within 30 days or be in at least HOT STANDBY within 6 hours.
- b. With only the pressurizer vapor space vent path inoperable, maintain the inoperable vent path closed with power removed from the valve actuator of the solenoid valves in the inoperable vent path, and:
 1. Verify at least one PORV and its associated flow path is OPERABLE within 72 hours and restore the inoperable pressurizer vapor space vent path to OPERABLE status prior to entering MODE 2 following the next HOT SHUTDOWN of sufficient duration, or
 2. Restore the inoperable pressurizer vapor space vent path to OPERABLE status within 30 days, or be in at least HOT STANDBY within 6 hours.
- c. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.4.13.1 Each reactor coolant system vent path shall be demonstrated OPERABLE by testing each valve in the vent path per Specification 4.0.5.

REACTOR COOLANT SYSTEM

REACTOR COOLANT SYSTEM VENTS

SURVEILLANCE REQUIREMENTS (Continued)

4.4.13.2 Each reactor coolant system vent path shall be demonstrated OPERABLE at least once per 18 months by:

- a. Verifying all manual isolation valves in each vent path are locked in the open position.
- b. Verifying flow through the reactor coolant system vent paths with the vent valves open.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

<u>PENETRATION NO.</u>	<u>ISOLATION CHANNEL</u>	<u>ISOLATION VALVE IDENTIFICATION NO.</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SECONDS)</u>
61	NA	SFP-184	Refueling Pool Outlet	NA
	NA	SFP-182		NA
	NA	SFP-180		NA
	NA	SFP-186		NA
62	SIAS A	PH-6579-MOV	Containment Heating Outlet	≤13
64	NA	PH-387	Containment Heating Inlet	NA

- (1) Manual or remote manual valve which is closed during plant operation.
- (2) May be opened below 300°F to establish shutdown cooling flow.
- (3) Containment purge valves will be shut in MODES 1, 2, 3 and 4 per TS 3/4 6.1.7.

* May be open on an intermittent basis under administrative control.

** Containment purge isolation valves isolation times will only apply in MODE 6 when the valves are required to be OPERABLE and they are open. Isolation time for containment purge isolation valves is NA for MODES 1, 2, 3 and 4 per TS 3/4 6.1.7, during which time these valves must remain closed.

- (4) Containment vent isolation valves shall be opened for containment pressure control, airborne radioactivity control, and surveillance testing purposes only.

CALVERT CLIFFS - UNIT 2

3/4 6-25

Amendment No. 471, 719, 789, 8198

CONTAINMENT SYSTEMS

3/4.6.5 COMBUSTIBLE GAS CONTROL

HYDROGEN ANALYZERS

LIMITING CONDITION FOR OPERATION

3.6.5.1 Two independent containment hydrogen analyzers shall be OPERABLE.

APPLICABILITY: MODES 1 and 2.

ACTION:

- a. With one hydrogen analyzer inoperable, restore the inoperable analyzer to OPERABLE status within 30 days or:
 1. Verify containment atmosphere grab sampling capability and prepare and submit a special report to the Commission pursuant to Specification 6.9.2 within the following 30 days, outlining the ACTION taken, the cause for the inoperability, and the plans and schedule for restoring the system to OPERABLE status, or
 2. Be in at least HOT STANDBY within the next 6 hours.
- b. With both hydrogen analyzers inoperable, restore at least one inoperable analyzer to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours.
- c. Specification 3.0.4 is not applicable to this requirement.

SURVEILLANCE REQUIREMENTS

4.6.5.1 Each hydrogen analyzer shall be demonstrated OPERABLE at least bi-weekly on a STAGGERED TEST BASIS by drawing a sample from the waste gas system through the hydrogen analyzer.

4.6.5.2 Each hydrogen analyzer shall be demonstrated OPERABLE at least once per 92 days on a STAGGERED TEST BASIS by performing a CHANNEL CALIBRATION using sample gases in accordance with manufacturers' recommendations.

REACTOR COOLANT SYSTEM

BASES

3/4.4.13 REACTOR COOLANT SYSTEM VENTS

Reactor Coolant System Vents are provided to exhaust noncondensable gases and/or steam from the primary system that could inhibit natural circulation core cooling. The OPERABILITY of at least one reactor coolant system vent path from the reactor vessel head and the pressurizer vapor space ensures the capability exists to perform this function.

The valve redundancy of the reactor coolant system vent paths serves to minimize the probability of inadvertent or irreversible actuation while ensuring that a single failure of a vent valve, power supply or control system does not prevent isolation of the vent path.

The function, capabilities, and testing requirements of the reactor coolant system vent systems are consistent with the requirements of Item II.B.1 of NUREG-0737, "Clarification of TMI Action Plan Requirements," November 1980.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 119 AND 101

TO FACILITY OPERATING LICENSE NOS. DPR-53 AND DPR-69

BALTIMORE GAS AND ELECTRIC COMPANY

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-317 AND 50-318

Introduction

By applications for license amendments dated December 22, 1983 and October 25, 1985, as supplemented by letter dated April 21, 1986, the Baltimore Gas and Electric Company (BG&E) requested changes to the Technical Specifications (TS) for Calvert Cliffs Units 1 and 2. The proposed amendments would change the Unit 1 and Unit 2 Technical Specifications (TS) as follows: (1) a provision would be added to TS 3.6.5.1, "Combustible Gas Control-Hydrogen Analyzers," to allow the changing of operational Modes with inoperable hydrogen analyzers (3.0.4 not applicable), (2) the reporting requirements of TS 4.4.5.5, "Reports-Steam Generators" would be clarified, and (3) a new TS 3/4.4.13, "Reactor Coolant System Vents" would be incorporated in the TS.

Discussion and Evaluation

BG&E has proposed a change to TS 3.6.5.1 to allow the changing of operational Modes with inoperable hydrogen analyzers. The purpose of the hydrogen analyzers is to sample the containment atmosphere, following a loss-of-coolant accident (LOCA), in order to determine the need for hydrogen reduction (via hydrogen recombiners or venting.) At the present time, TS 3.0.4 is applicable to TS 3.6.5.1 which means that the reactor cannot change operational Modes (for example, the reactor could not start up if already shutdown) should a hydrogen analyzer become inoperable. BG&E proposes that the statement "Specification 3.0.4 is not applicable to this requirement" be added to TS 3.6.5.1.

Under the requirements of TS 3.6.5.1, the reactor may operate at full power for up to 30 days with an inoperable hydrogen analyzer, or up to 72 hours with two inoperable hydrogen analyzers, prior to taking remedial action. The proposed change to the TS would not change these time limits or the requisite remedial actions. The proposed change to TS 3.6.5.1 would only allow the changing of operational modes (i.e., reactor startup) with one or two inoperable hydrogen analyzers. This action would not be more significant from a safety standpoint than full power operation over an equivalent period of time, which is permitted by TS 3.6.5.1.

Based upon the above, the proposed change to TS 3.6.5.1 is acceptable.

BG&E has proposed changes to TS 4.4.5.5 to clarify reporting requirements associated with steam generator tube inspections. Specifically, the following clarifications have been proposed: (1) the 15-day reporting requirement of TS 4.4.5.5a would be cross-referenced to the special reporting requirements of TS 6.9.2; (2) the annual operating report requirements of TS 4.4.5.5b would be cross-referenced to the annual operating report requirements of TS 6.9.1.5b; (3) the 30-day reporting requirement of TS 4.4.5.5c would be cross-referenced to the special reporting requirements of TS 6.9.2; and (4) the telephone reporting requirement of TS 4.4.5.5c would be corrected by removing the cross-reference to 6.9.2, since these are written reporting requirements. The proposed change to TS 4.4.5.5c would also identify the recipient of the telephone report as the Regional Administrator since the Regional Administrator is the recipient of other, similar, reports. The proposed changes to TS 4.4.5.5 would clarify the associated reporting requirements and reduce the likelihood of errors in the preparation and submittal of reports.

The proposed changes to TS 4.4.5.5 are administrative in nature, helps to assure consistency within the TS, and thus the proposed changes are acceptable.

BG&E has proposed adding a new TS for the reactor coolant system (RCS) high point vents to the existing TS. On November 1, 1983, the NRC issued Generic Letter No. 83-37 (GL 83-37) to all pressurized water reactor licensees. This letter contained guidance concerning TS which the NRC believed to be appropriate as addressed in NUREG-0737, "Clarification of TMI Action Plan Requirements". BG&E responded, in part, to GL 83-37 via their application for license amendment dated December 22, 1983 concerning the RCS High Point Vents. These vents, installed in the reactor vessel head and the pressurizer vapor space, would allow post-LOCA venting of non-condensable gases to enhance reactor core cooling via natural circulation. Following discussions with the NRC staff, BG&E supplemented their application by letter dated April 21, 1986.

BG&E's proposed TS for the RCS vents conform substantially to the model TS provided in GL 83-37 with regard to both Limiting Conditions for Operation and Surveillance Requirements. One area of difference is that BG&E's proposed TS allow use of the pressurizer power operated relief valves (PORVs) as a backup to the pressurizer high point vent. In the event that the pressurizer high point vent becomes inoperable, the proposed TS would allow an operable PORV to perform the high point vent function. The inoperable high point vent may remain inoperable until an outage of sufficient duration occurs to allow repair of the high point vent. The PORV represents a design feature that, by virtue of its location and reliability, provides a suitable back-up to the high point pressurizer vent.

Based upon the above, the proposed changes to the TS which provide new TS for the RCS high point vents are acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or a change in a surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). These amendments also involve changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, with respect to these items, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(10). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: June 30, 1986

Principal Contributor:

D. Jaffe