August 26, 1985

DCS 01

Docket Nos. 50-317
and 50-318

Mr. A. E. Lundvall, Jr.
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Dear Mr. Lundvall:

The Commission has issued the enclosed Amendment Nos. 107 and 88 to Facility Operating License Nos. DPR-53 and DPR-69 for Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications completing our action in response to your application dated January 31, 1985.

The amendments change the Unit 1 and Unit 2 Technical Specifications (TS) to reflect clarification and increased flexibility for determination of reactor coolant system leakage as specified in TS 3/4.4.6.1, "Leakage Detection Systems" and TS 3/4.4.6.2, "Reactor Coolant System Leakage."

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

/S/

David H. Jaffe, Project Manager
Operating Reactors Branch #3
Division of Licensing

Enclosures:
1. Amendment No. 107 to DPR-53
2. Amendment No. 88 to DPR-69
3. Safety Evaluation

cc w/enclosure:
See next page

D. Crutchfield
ADSA
8/26/85

ORB#3:DL	ORB#3:DL	ORB#3:DL	OELD	AD:OR:DL
PMKreutzer	DJaffe	EButcher		GCLainas
8/20/85	7/12/85	7/19/85	7/22/85	8/26/85

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 107
License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated January 31, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-53 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 107, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Edward J. Butcher, Acting Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 26, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 107

FACILTIY OPERATING LICENSE NO. DPR-53

DOCKET NO. 50-317

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are provided to maintain document completeness.

Remove Pages	Insert Pages
3/4 4-13	3/4 4-13
3/4 4-14	3/4 4-14
3/4 4-15	3/4 4-15

REACTOR COOLANT SYSTEM

3/4.4.6 REACTOR COOLANT SYSTEM LEAKAGE

LEAKAGE DETECTION SYSTEMS

LIMITING CONDITION FOR OPERATION

3.4.6.1 The following Reactor Coolant System leakage detection systems shall be OPERABLE:

a. A containment atmosphere particulate radioactivity monitoring system,

b. The containment sump level alarm system, and

c. A containment atmosphere gaseous radioactivity monitoring system.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

a. With only two of the above required leakage detection systems OPERABLE, operation may continue for up to 30 days provided grab samples of the containment atmosphere are obtained and analyzed at least once per 24 hours when either the required gaseous or particulate radioactivity monitoring system is inoperable; otherwise be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

b. With only one of the above required leakage detection systems OPERABLE, operation may continue for up to 7 days provided that:

1. Grab samples of the containment atmosphere are obtained and analyzed at least once per 12 hours, and
2. The Reactor Coolant System water inventory balance of Surveillance Requirement 4.4.6.2.c is performed at least once per 24 hours.

Otherwise be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.6.1 The leakage detection systems shall be demonstrated OPERABLE by:

a. Containment atmosphere gaseous and particulate monitoring systems-performance of CHANNEL CHECK, CHANNEL CALIBRATION and CHANNEL FUNCTIONAL TEST at the frequencies specified in Table 4.3-3, and

b. Containment sump level alarm system-performance of CHANNEL CALIBRATION at least once per 18 months.

REACTOR COOLANT SYSTEM

REACTOR COOLANT SYSTEM LEAKAGE

LIMITING CONDITION FOR OPERATION

3.4.6.2 Reactor Coolant System leakage shall be limited to:

a. No PRESSURE BOUNDARY LEAKAGE,

b. 1 GPM UNIDENTIFIED LEAKAGE,

c. 1 GPM total primary-to-secondary leakage through steam generators, and

d. 10 GPM IDENTIFIED LEAKAGE from the Reactor Coolant System.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

b. With any Reactor Coolant System leakage greater than any one of the above limits, excluding PRESSURE BOUNDARY LEAKAGE, reduce the leakage rate to within limits within 4 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.6.2 Reactor Coolant System leakages shall be demonstrated to be within each of the above limits by:

a. Either:

1. Monitoring the containment atmosphere particulate or gaseous radioactivity at least once per 12 hours, or

2. With the gaseous and particulate monitors inoperable, conducting the containment atmosphere grab sample analysis in accordance with the ACTION requirements of T.S. 3.4.6.1.

b. Monitoring the containment sump discharge frequency at least once per 12 hours, when the containment sump level alarm system is OPERABLE,

SURVEILLANCE REQUIREMENTS (Continued)

c. Performance of a Reactor Coolant System water inventory balance at least once per 72 hours during steady state operation and at least once per 24 hours when required by ACTION 3.4.6.1.b, except when operating in the shutdown cooling mode, and

d. Monitoring the reactor vessel head closure seal leakage detection system at least once per 24 hours.

REACTOR COOLANT SYSTEM

CHEMISTRY

LIMITING CONDITION FOR OPERATION

3.4.7 The Reactor Coolant System chemistry shall be maintained within the limits specified in Table 3.4-1.

APPLICABILITY: At all times.

ACTION:

MODES 1, 2, 3 and 4

a. With any one or more chemistry parameter in excess of its Steady State Limit but within its Transient Limit, restore the parameter to within its Steady State Limit within 24 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

b. With any one or more chemistry parameter in excess of its Transient Limit, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

MODES 5 and 6

With the concentration of either chloride or fluoride in the Reactor Coolant System in excess of its Steady State Limit for more than 24 hours or in excess of its Transient Limit, reduce the pressurizer pressure to $\leq$ 500 psia, if applicable, and perform an engineering evaluation to determine the effects of the out-of-limit condition on the structural integrity of the Reactor Coolant System; determine that the Reactor Coolant System remains acceptable for continued operation prior to increasing the pressurizer pressure above 500 psia or prior to proceeding to MODE 4.

SURVEILLANCE REQUIREMENTS

4.4.7 The Reactor Coolant System chemistry shall be determined to be within the limits by analysis of those parameters at the frequencies specified in Table 4.4-3.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 88
License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated January 31, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-69 is hereby amended to read as follows:

   2. Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 88 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Edward J. Butcher, Acting Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 26, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 88

FACILTIY OPERATING LICENSE NO. DPR-69

DOCKET NO. 50-318

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are provided to maintain document completeness.

Remove Pages	Insert Pages
3/4 4-13	3/4 4-13
3/4 4-14	3/4 4-14
3/4 4-15	3/4 4-15

REACTOR COOLANT SYSTEM

3/4.4.6 REACTOR COOLANT SYSTEM LEAKAGE

LEAKAGE DETECTION SYSTEMS

LIMITING CONDITION FOR OPERATION

3.4.6.1 The following Reactor Coolant System leakage detection systems shall be OPERABLE:

a. A containment atmosphere particulate radioactivity monitoring system,

b. The containment sump level alarm system, and

c. A containment atmosphere gaseous radioactivity monitoring system.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

a. With only two of the above required leakage detection systems OPERABLE, operation may continue for up to 30 days provided grab samples of the containment atmosphere are obtained and analyzed at least once per 24 hours when either the required gaseous or particulate radioactivity monitoring system is inoperable; otherwise be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

b. With only one of the above required leakage detection systems OPERABLE, operation may continue for up to 7 days provided that:

1. Grab samples of the containment atmosphere are obtained and analyzed at least once per 12 hours, and

2. The Reactor Coolant System water inventory balance of Surveillance Requirement 4.4.5.2.c is performed at least once per 24 hours.

Otherwise be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.6.1 The leakage detection systems shall be demonstrated OPERABLE by:

a. Containment atmosphere gaseous and particulate monitoring systems-performance of CHANNEL CHECK, CHANNEL CALIBRATION and CHANNEL FUNCTIONAL TEST at the frequencies specified in Table 4.3-3, and

b. Containment sump level alarm system-performance of CHANNEL CALIBRATION at least once per 18 months.

REACTOR COOLANT SYSTEM

REACTOR COOLANT SYSTEM LEAKAGE

LIMITING CONDITION FOR OPERATION

3.4.6.2 Reactor Coolant System leakage shall be limited to:

a. No PRESSURE BOUNDARY LEAKAGE,

b. 1 GPM UNIDENTIFIED LEAKAGE,

c. 1 GPM total primary-to-secondary leakage through steam generators, and

d. 10 GPM IDENTIFIED LEAKAGE from the Reactor Coolant System.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

b. With any Reactor Coolant System leakage greater than any one of the above limits, excluding PRESSURE BOUNDARY LEAKAGE, reduce the leakage rate to within limits within 4 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.6.2 Reactor Coolant System leakages shall be demonstrated to be within each of the above limits by:

a. Either:

1. Monitoring the containment atmosphere particulate or gaseous radioactivity at least once per 12 hours, or

2. With the gaseous and particulate monitors inoperable, conducting the containment atmosphere grab sample analysis in accordance with the ACTION requirements of T.S. 3.4.6.1.

b. Monitoring the containment sump discharge frequency at least once per 12 hours, when the containment sump level alarm system is OPERABLE,

SURVEILLANCE REQUIREMENTS (Continued)

c. Performance of a Reactor Coolant System water inventory balance at least once per 72 hours during steady state operation and at least once per 24 hours when required by ACTION 3.4.6.1.b, except when operating in the shutdown cooling mode, and

d. Monitoring the reactor vessel head closure seal leakage detection system at least once per 24 hours.

REACTOR COOLANT SYSTEM

CHEMISTRY

LIMITING CONDITION FOR OPERATION

3.4.7 The Reactor Coolant System chemistry shall be maintained within the limits specified in Table 3.4-1.

APPLICABILITY: At all times.

ACTION:

MODES 1, 2, 3 and 4

a. With any one or more chemistry parameter in excess of its Steady State Limit but within its Transient Limit, restore the parameter to within its Steady State Limit within 24 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

b. With any one or more chemistry parameter in excess of its Transient Limit, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

MODES 5 and 6

With the concentration of either chloride or fluoride in the Reactor Coolant System in excess of its Steady State Limit for more than 24 hours or in excess of its Transient Limit, reduce the pressurizer pressure to $\leq$ 500 psia, if applicable, and perform an engineering evaluation to determine the effects of the out-of-limit condition on the structural integrity of the Reactor Coolant System; determine that the Reactor Coolant System remains acceptable for continued operation prior to increasing the pressurizer pressure above 500 psia or prior to proceeding to MODE 4.

SURVEILLANCE REQUIREMENTS

4.4.7 The Reactor Coolant System chemistry shall be determined to be within the limits by analysis of those parameters at the frequencies specified in Table 4.4-3.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 107 AND 88

TO FACILITY OPERATING LICENSE NOS. DPR-53 AND DPR-69

BALTIMORE GAS AND ELECTRIC COMPANY

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-317 AND 50-318

Introduction

By application for license amendment dated January 31, 1985, Baltimore Gas and Electric Company (BG&E) requested changes to the Technical Specifications (TS) for Calvert Cliffs Units 1 and 2 to reflect clarification and increased flexibility for determination of reactor coolant system leakage as specified in TS 3/4.4.6.1, "Leakage Detection Systems" and TS 3/4.4.6.2, "Reactor Coolant System Leakage."

Discussion and Evaluation

Technical Specification 3/4.4.6.1 provides operability and surveillance requirements for three instrumentation systems which will detect reactor coolant system leakage. Two of the instrumentation systems specified in 3.4.6.1, the containment particulate radioactivity monitoring (PRM) system and the containment atmospheric gaseous radioactivity monitoring (GRM) system, are not completely independent in that they both share the same sample pump. The containment sump level alarm system, the third leakage monitoring system addressed in TS 3/4.4.6.1, is independent of the PRM and GRM systems.

In the past, TS 3.4.6.1 has caused difficulty with regard to interpretation. This difficulty is caused by the "action" statement which is applicable when one leak detection system is inoperable but which also requires remedial action to be taken when two systems (the GRM and/or the PRM) are inoperable. The licensee has proposed a change to TS 3.4.6.1 to correct this ambiguity. The proposed change would make the existing action statement the first of a two-part requirement. The first part would be applicable, unambiguously, when only one leakage detection system is inoperable by changing the phrase "...when the required gaseous and/or particulate radioactivity monitoring system is inoperable..." to "...when either the required gaseous or particulate radioactivity monitoring system is inoperable...." The new second part of the action statement would be unambiguously applicable when two leakage detection systems are inoperable. This action statement requires that the existing remedial action be taken, obtaining and analyzing containment atmospheric grab samples at least once per 24 hours, and also that a reactor coolant system water inventory balance be undertaken every 24

hours. The reactor coolant system water inventory balance is an approved leak detection methodology that is currently required to be performed every 72 hours by TS 4.4.6.2c.

The operation of leak detection capabilities included in the proposed TS is as follows:

° Sump level alarm system - Unit 1 has a 49 gallon sump while Unit 2 has a 44 gallon sump. When the sump is full, a control room alarm is sounded. The control room operator logs the time of the alarm and drains the sump. When the sump is empty, the alarm clears (and is reset). The time between sump level alarms would indicate the leak rate.

° PRM and GRM - These radiation monitors provide indication of leakage but, due to the uncertainty associated with leak location and type and coolant activity, it is not practical to provide a correlation between containment radiation level and leakage rate. The PRM and GRM are provided with alarm setpoints set at 1.5 multiplied by the background radiation level. Plant procedures require the control room operator to perform a reactor coolant system water inventory balance if a PRM or GRM alarm is sounded. The GRM and PRM share a common sample pump. Should this pump fail, an installed spare pump can be promptly made operable. In the event that the installed spare pump is also inoperable, an on-site spare pump can be installed within 7 days.

° Reactor coolant system water inventory balance - This method involves a calculation of reactor coolant system volume based upon the level in various tanks and other volumes. This method is accurate enough to detect a leak of 0.1 gpm, takes between 2 and 6 hours to perform, and would provide adequate quantification of leakage detected by the GRM or PRM.

° Grab sample - A sample of the containment atmosphere can be obtained. This sample, analyzed by portable instruments, provides an indication of activity that is at least equivalent to the GRM or PRM.

Based upon our evaluation, we conclude that adequate means of reactor coolant system leak detection will be required to be operable as is presently the case in TS 3.4.6.1. Moreover, numerous other systems and means for leak detection are described and discussed in Section 4.3 of the Updated FSAR and are used by reactor operators for leak detection purposes. They include: Containment Pressure and Temperature Indication, Pressurizer Pressure and Level Indication and Alarm, Containment Area Radiation Monitor Indication and Alarm, Containment Humidity Indicators, Reactor Coolant Drain Tank Level Indication, and Reactor Coolant Make-up Water Flow Integrators. The proposed TS does not increase the probability of accidents or reduce any safety margins since no changes in design or operating modes of the leak detection systems are involved. Consequently, the proposed changes to TS 3.4.6.1 are acceptable.

Finally, the licensee has requested changes to TS 4.4.6.2 as follows:

- Surveillance Requirement (SR) 4.4.6.2a which requires monitoring the containment atmosphere once every 12 hours with the PRM would be modified to also permit monitoring with the GRM or grab samples.

- SR 4.4.6.2b which requires monitoring containment sump discharge every 12 hours using the containment sump level alarm system would be modified to suspend this requirement when the containment sump level alarm system is inoperable.

- SR 4.4.6.2c which requires a reactor coolant system water inventory balance at least once per 72 hours would be modified to require this balance every 24 hours when used as an alternate leak detection method per TS 3.4.6.1.

The proposed changes to TS 4.4.6.2 are required to achieve consistency with existing and proposed requirements of TS 3.4.6.1 and are, therefore, acceptable.

Environmental Consideration

These amendments involve a change in the installation or use and surveillance of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 26, 1985

Principal Contributor:
D. Jaffe