January 8, 1986

Docket Nos. 50-317
and 50-318

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OELD	LJHarmon
ACRS-10	Calvert Cliffs Gray

Mr. A. E. Lundvall, Jr.
Vice President - Supply
Baltimore Gas & Electric Company
P. O. Box 1475
Baltimore, Maryland 21203

Dear Mr. Lundvall:

The Commission has issued the enclosed Amendment Nos. 112 and 95 to Facility Operating License Nos. DPR-53 and DPR-69 for Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated April 26, 1985.

The amendments revise the Technical Specifications to allow completion of the third containment Integrated Leak Rate Test prior to the 10-year Inservice Inspection outage.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

The Commission has also issued the enclosed Exemption to 10 CFR Part 50, Appendix J, which was necessary in that the schedular requirements of TS 4.6.1.2a are also included in Section III.D.1(a) of Appendix J. This Exemption is in response to your application dated August 15, 1985. A copy of the Exemption is being filed with the Office of the Federal Register for publication.

Sincerely,

/S/

David H. Jaffe, Project Manager
PWR Project Directorate #8
Division of PWR Licensing-B

Enclosures:
1. Amendment No. 112 to DPR-53
2. Amendment No. 95 to DPR-69
3. Safety Evaluation
4. Exemption

cc w/enclosures:
See next page

PBB#8 *	PBB#8 *	PBB#8 *	OELD
PMKreutzer	DJaffe	AThadani	J. GRAY
/ /85	/ /85	/ /85	12/31/85

* see previous concurrence

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David H. Jaffe, Project Manager
PWR Project Directorate #8
Division of PWR Licensing-B

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See next page

PBB#8	PBB#8	PBB#8	OELD
PMKreutzer	DJaffe	AThadani	
1/6/86	12/20/85	12/20/85	12/20/85

cc:
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 112
License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated April 26, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-53 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 112 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Ashok C. Thadani, Director
PWR Project Directorate #8
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 8, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 112

FACILTIY OPERATING LICENSE NO. DPR-53

DOCKET NO. 50-317

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the areas of change. The corresponding overleaf page is provided to maintain document completeness.

Remove Page	Insert Page
3/4 6-2	3/4 6-2

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 PRIMARY CONTAINMENT

CONTAINMENT INTEGRITY

LIMITING CONDITION FOR OEPRATION

3.6.1.1 Primary CONTAINMENT INTEGRITY shall be maintained.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

Without primary CONTAINMENT INTEGRITY, restore CONTAINMENT INTEGRITY within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.1 Primary CONTAINMENT INTEGRITY shall be demonstrated:

a. At least once per 31 days by verifying that all penetrations* not capable of being closed by OPERABLE containment automatic isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in their positions, except as provided in Table 3.6-1 of Specification 3.6.4.1.

b. By verifying that each containment air lock is OPERABLE per Specification 3.6.1.3.

c. By verifying that the equipment hatch is closed and sealed, prior to entering Mode 4 following a shutdown where the equipment hatch was opened, by conducting a Type B test per Appendix J to 10 CFR Part 50.

*Except valves, blind flanges, and deactivated automatic valves which are located inside the containment and are locked, sealed, or otherwise secured in the closed position. These penetrations shall be verified closed during each COLD SHUTDOWN except that such verification need not be performed more often than once per 92 days.

CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

a. An overall integrated leakage rate of:

1. $\leq L_a$ (346,000 SCCM), 0.20 percent by weight of the containment air per 24 hours at P_a, 50 psig, or

2. $\leq L_t$ (61,600 SCCM), 0.058 percent by weight of the containment air per 24 hours at a reduced pressure of P_t, 25 psig.

b. A combined leakage rate of $\leq 0.60\ L_a$ (207,600 SCCM), for all penetrations and valves subject to Type B and C tests when pressurized to P_a.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75\ L_a$ (259,500 SCCM) or $0.75\ L_t$ (46,200 SCCM), as applicable, or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.60\ L_a$, restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR Part 50 using the methods and provisions of ANSI N45.4 - 1972:

a. Three Type A tests (overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals during shutdown at either P_a (50 psig) or at P_t (25 psig) during each 10-year service period.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 95
License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated April 26, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-69 is hereby amended to read as follows:

   2. Technical Specifications

      The Technical Specifications contained in Appendix A, as revised through Amendment No. 95 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Ashok C. Thadani, Director
PWR Project Directorate #8
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 8, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 95

FACILTIY OPERATING LICENSE NO. DPR-69

DOCKET NO. 50-318

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the areas of change. The corresponding overleaf page is provided to maintain document completeness.

Remove Page	Insert Page
3/4 6-2	3/4 6-2

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 PRIMARY CONTAINMENT

CONTAINMENT INTEGRITY

LIMITING CONDITION FOR OEPRATION

3.6.1.1 Primary CONTAINMENT INTEGRITY shall be maintained.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

Without primary CONTAINMENT INTEGRITY, restore CONTAINMENT INTEGRITY within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.1 Primary CONTAINMENT INTEGRITY shall be demonstrated:

a. At least once per 31 days by verifying that all penetrations* not capable of being closed by OPERABLE containment automatic isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in their positions, except as provided in Table 3.6-1 of Specification 3.6.4.1.

b. By verifying that each containment air lock is OPERABLE per Specification 3.6.1.3.

c. By verifying that the equipment hatch is closed and sealed, prior to entering Mode 4 following a shutdown where the equipment hatch was opened, by conducting a Type B test per Appendix J to 10 CFR Part 50.

---

*Except valves, blind flanges, and deactivated automatic valves which are located inside the containment and are locked, sealed, or otherwise secured in the closed position. These penetrations shall be verified closed during each COLD SHUTDOWN except that such verification need not be performed more often than once per 92 days.

CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

a. An overall integrated leakage rate of:

1. $\leq L_a$ (346,000 SCCM), 0.20 percent by weight of the containment air per 24 hours at P_a, 50 psig, or

2. $\leq L_t$ (44,600 SCCM), 0.042 percent by weight of the containment air per 24 hours at a reduced pressure of P_t, 25 psig.

b. A combined leakage rate of $\leq 0.60\ L_a$ (207,600 SCCM) for all penetrations and valves subject to Type B and C tests when pressurized to P_a.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75\ L_a$ (259,500 SCCM), or $0.75\ L_t$ (33,400 SCCM), as applicable, or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.60\ L_a$, restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4 - 1972:

a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 $\pm$ 10 month intervals during shutdown at either P_a (50 psig) or at P_t (25 psig) during each 10-year service period.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 112 AND 95

TO FACILITY OPERATING LICENSE NOS. DPR-53 AND DPR-69

BALTIMORE GAS AND ELECTRIC COMPANY

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-317 AND 50-318

Introduction

By application for license amendment dated April 26, 1985, Baltimore Gas and Electric Company (BG&E) requested changes to the Technical Specifications (TS) for Calvert Cliffs Units 1 and 2. The proposed amendments would change the Unit 1 and Unit 2 TS 4.6.1.2a to allow completion of the third containment Integrated Leak Rate Test (ILRT) prior to the 10-year Inservice Inspection (ISI) outage.

Discussion and Evaluation

The existing Calvert Cliffs TS 4.6.1.2a references Appendix J of 10 CFR Part 50 and requires that: "Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals during shutdown at either P_a (50 psig) or at P_t (25 psig) during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection." Performance of the third ILRT during the 10-year ISI outage which is required by Appendix J would result in the violation of the more frequent 40 ± 10 month interval required by the TS. Accordingly, the licensee has requested a change to the ILRT schedule, as reflected in TS 4.6.1.2a, to allow the third ILRT to be conducted earlier than required by Appendix J and to eliminate the reference to the Appendix J 10-year inspection. The proposed schedule would satisfy the 40 ± 10 month inspection interval requirement of TS 4.6.1.2a which is of primary importance because of the more frequent maintenance of containment integrity through the ILRT program. The coincidence of the third ILRT with the 10-year ISI outage is of clearly secondary importance for this schedule change. An exemption from Appendix J to allow earlier containment testing is being considered in a separate but parallel action.

The proposed change to TS 4.6.1.2a would only affect the scheduling of one of three Type A tests during each 10-year service period; the scheduling of the two remaining Type A tests would not be affected. The maximum impact of scheduling on the third Type A test would be minor and amount to the duration of one refueling cycle. Moreover, the impact would not be cumulative since each 10 year service period would start a new Type A test schedule. Accordingly, the staff finds the proposed change to TS 4.6.1.2a to be acceptable.

Finally, the licensee has proposed deletion of the footnote associated with TS 4.6.1.2a. This footnote provides specific dates for completion of the third ILRT for Calvert Cliffs. This footnote would no longer be necessary since any ILRT test would be conducted on a 40 ± 10 month interval, regardless of the ISI schedule. Accordingly, deletion of the footnote associated with TS 4.6.1.2a affects neither ISI nor ILRT schedules, is administrative in nature, and is acceptable.

Environmental Consideration

These amendments involve a change in surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Sec. 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: January 8, 1985

Principal Contributor:
D. Jaffe

UNITED STATES OF AMERICA

NUCLEAR REGULATORY COMMISSION

In the Matter of	)	
	)	
BALTIMORE GAS AND ELECTRIC COMPANY	)	Docket Nos. 50-317 and 50-318
	)	
(Calvert Cliffs Nuclear Power Plant Unit Nos. 1 and 2)	)	

EXEMPTION

I.

The Baltimore Gas and Electric Company (the licensee) is the holder of Facility Operating License Nos. DPR-53 and DPR-69 which authorize operation of the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2. These licenses provide, among other things, that they are subject to all rules, regulations and Orders of the Commission now or hereafter in effect.

The facility comprises two pressurized water reactors at the licensee's site located in Calvert County, Maryland.

II.

Appendix J to 10 CFR Part 50, "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors," includes schedular and technical requirements for the conduct of "Type A" reactor containment Integrated Leak Rate Tests (ILRTs). With regard to schedule for ILRTs, Appendix J, Section II.D.1.(a) requires that ". . . three Type A tests shall be performed, at approximately equal intervals during each 10-year service period. The third test of each set shall be conducted when the plant is shutdown for the 10-year plant inservice inspections."

By application dated August 15, 1985 the licensee requested a change to the ILRT schedule, as specified in Appendix J, Section III.D.1.(a) to allow the third ILRT to be conducted prior to the 10-year ISI outage. The proposed exemption to Appendix J, Section III.D.1.(a) would only affect the scheduling of one of three Type A tests during each 10-year service period; the scheduling of the two remaining Type A tests would not be affected. The maximum impact of scheduling on the third Type A test would be minor and amount to the duration of one refueling cycle. Moreover, the impact would not be cumulative since each 10 year service period would start a new Type A test schedule. The remaining schedular requirements of Appendix J Section III.D.1.(a), ". . .three Type A tests shall be performed, at approximately equal intervals during each 10-year service period," which is of principal importance, would remain in effect.

III.

Accordingly, the Commission has determined that, pursuant to 10 CFR 50.12, an exemption is authorized by law and will not endanger life or property or the common defense and security and is otherwise in the public interest and hereby grants the following exemptions with respect to the requirements of Appendix J to 10 CFR Part 50:

With regard to the third ILRT of a 10-year service period, this ILRT need not be conducted when the plant is shut down for the 10-year plant inservice inspection.

Pursuant to 10 CFR 51.32 the Commission has determined that the issuance of the exemption will have no significant impact on the environment (50 FR 40632).

FOR THE NUCLEAR REGULATORY COMMISSION

Frank J. Miraglia, Director
Division of PWR Licenisng-B

Dated at Bethesda, Maryland this 8th day of January, 1986.