

SEP 22 1983

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Docket File
NRC PDR
Local PDR
ORB#3 Rdg
PMKreutzer
DHJaffe
OELD
LJHarmon
EJordan
JTaylor
TBarnhart (8)
WJones
DBrinkman
ACRS (10)
OPA, C. Miles
~~R. Ferguson~~
RDiggs

Docket Nos. 50-317
and 50-318

Mr. A. E. Lundvall, Jr.
Vice President - Supply
Baltimore Gas & Electric Company
P. O. Box 1475
Baltimore, Maryland 21203

Dear Mr. Lundvall:

The Commission has issued the enclosed Amendment Nos. 87 and 69 to Facility Operating License Nos. DPR-53 and DPR-69 for Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated May 27, 1983.

These amendments to the Technical Specifications (TS) change TS 3.6.4.1, "Containment Isolation Valves" to reflect a modification to value 1(2)-SV-6529, changing it from an automatic isolation valve with a required response time (closure) of less than or equal to 7 seconds to a locked closed isolation valve with no required response time.

A copy of the Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular monthly Federal Register Notice.

Sincerely,

Original signed by:

David H. Jaffe, Project Manager
Operating Reactors Branch #3
Division of Licensing

Enclosures:

1. Amendment No. 87 to DPR-53
2. Amendment No. 69 to DPR-69
3. Safety Evaluation

cc: See next page

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OFFICE	ORB#3:DL	ORB#3:DL	ORB#3:DL	AD:OR:DL	OELD
SURNAME	PMKreutzer	DJaffe:cd	JRM:1er	GClainas	<i>[Signature]</i>
DATE	9/1/83	9/9/83	9/1/83	9/1/83	9/14/83

Baltimore Gas and Electric Company

cc:

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Plant Superintendent
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Lusby, MD 20657

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Calvert Cliffs Project Engineer
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Combustion Engineering, Inc.
Attn: Mr. R. R. Mills, Manager
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Baltimore Gas & Electric Company
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Operations Quality Assurance
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Ms. Mary Harrison, President
Calvert County Board of County Commissioners
Prince Frederick, MD 20768

U. S. Environmental Protection Agency
Region III Office
Attn: Regional Radiation Representative
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Sixth and Walnut Streets
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Mr. Ralph E. Architzel
Resident Reactor Inspector
NRC Inspection and Enforcement
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Administrator, Power Plant Siting Program
Energy and Coastal Zone Administration
Department of Natural Resources
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Annapolis, MD 21204

Regional Administrator
Nuclear Regulatory Commission, Region I
Office of Executive Director for Operations
631 Park Avenue
King of Prussia, Pennsylvania 19406



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 87
License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated May 27, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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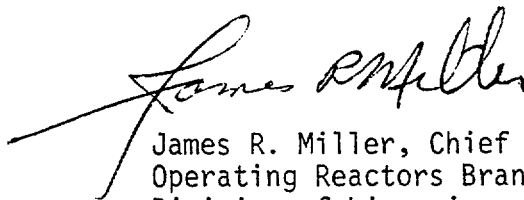
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-53 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 87, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: September 22, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 87

FACILITY OPERATING LICENSE NO. DPR-53

DOCKET NO. 50-317

Replace the following page of the Appendix A Technical Specifications with the enclosed page as indicated. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Page

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TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>PENETRATION NO.</u>	<u>ISOLATION CHANNELS</u>	<u>ISOLATION VALVE IDENTIFICATION NO.</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SECONDS)</u>
7A	NA NA	Blind Flange 19-1	ILRT	NA NA
7B	NA NA	Blind Flange 19-1	ILRT	NA NA
8	SIAS A SIAS B	MOV-5462 MOV-5463	Containment Normal Sump	≤ 13 ≤ 13
9	NA NA	238M3-1 238M3-2	Containment Spray	NA NA
10	NA NA	238M3-1 238M3-2	Containment Spray	NA NA
13	SIAS A, CRS A SIAS B, CRS B	CV-1410 (3) CV-1411 (3)	Purge Air Inlet	$\leq 7^{**}$ $\leq 7^{**}$

CALVERT CLIFFS - UNIT 1

3/4 6-19

Amendment No. 4A, 87

TABLE 3.6-1

CONTAINMENT ISOLATION VALVES

<u>PENETRATION NO.</u>	<u>ISOLATION CHANNEL</u>	<u>ISOLATION VALVE IDENTIFICATION NO.</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SECONDS)</u>
1A	SIAS A	CV-5465	R.C. and Pressurizer Sampling	<7
	SIAS A	CV-5466		<7
	SIAS A	CV-5467		<7
	SIAS B	CV-5464		<7
1B	SIAS A	CV-2180	Containment Vent Header to Waste Gas	<7
	SIAS B	CV-2181		<7
1C	SIAS A	CV-506	RCP Seals Controlled Bleedoff	<7
	SIAS B	CV-505		<7
1D	NA	SV-6529*	Post Accident Sampling Liquid Return to RC Drain Tank	NA
2A	SIAS A	CV-515	Letdown Line	<13
	SIAS B	CV-516		<13
	NA	7M3-1		NA
	NA	7M3-1		NA
2B	NA	CV-517	Charging Line	NA
	NA	CV-518		NA
	NA	CV-519		NA
	NA	SP-210M3-2		NA
	NA	210M3-2		NA

CALVERT CLIFFS - UNIT 1

3/4 6-19

Amendment No. 4/H, 87

TABLE 3.6-1

CONTAINMENT ISOLATION VALVES

<u>PENETRATION NO.</u>	<u>ISOLATION CHANNEL</u>	<u>ISOLATION VALVE IDENTIFICATION NO.</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SECONDS)</u>
1A	SIAS A	CV-5465	R.C. and Pressurizer Sampling	<7
	SIAS A	CV-5466		<7
	SIAS A	CV-5467		<7
	SIAS B	CV-5464		<7
1B	SIAS A	CV-2180	Containment Vent Header to Waste Gas	<7
	SIAS B	CV-2181		<7
1C	SIAS A	CV-506	RCP Seals Controlled Bleedoff	<7
	SIAS B	CV-505		<7
1D	NA	SV-6529*	Post Accident Sampling Liquid Return to RC Drain Tank	NA
2A	SIAS A	CV-515	Letdown Line	<13
	SIAS B	CV-516		<13
	NA	7M3-1		NA
	NA	7M3-1		NA
2B	NA	CV-517	Charging Line	NA
	NA	CV-518		NA
	NA	CV-519		NA
	NA	SP-210M3-2		NA
	NA	210M3-2		NA



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 69
License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated May 27, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

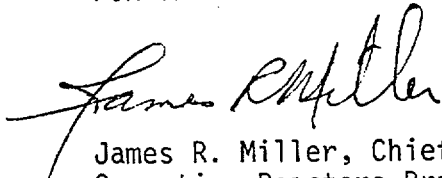
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-69 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 69, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: September 22, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 69

FACILITY OPERATING LICENSE NO. DPR-69

DOCKET NO. 50-318

Replace the following page of the Appendix A Technical Specifications with the enclosed page as indicated. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Page

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TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>PENETRATION NO.</u>	<u>ISOLATION CHANNELS</u>	<u>ISOLATION VALVE IDENTIFICATION NO.</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SECONDS)</u>
7A	NA NA	Blind Flange 19-1	ILRT	NA NA
7B	NA NA	Blind Flange 19-1	ILRT	NA NA
8	SIAS A SIAS B	MOV-5462 MOV-5463	Containment Normal Sump	<13 <13
9	NA NA	238M3-1 238M3-2	Containment Spray	NA NA
10	NA NA	238M3-1 238M3-2	Containment Spray	NA NA
13	SIAS A, CRS A SIAS B, CRS B	CV-1410 (3) CV-1411 (3)	Purge Air Inlet	<7** <7**

TABLE 3.6-1
CONTAINMENT ISOLATION VALVES

<u>PENETRATION NO.</u>	<u>ISOLATION CHANNEL</u>	<u>ISOLATION VALVE IDENTIFICATION NO.</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SECONDS)</u>
1A	SIAS A	CV-5465	R.C. and Pressurizer Sampling	<u><7</u>
	SIAS A	CV-5466		
	SIAS A	CV-5467		
	SIAS B	CV-5464		
1B	SIAS A	CV-2180	Containment Vent Header to Waste Gas	<u><7</u>
	SIAS B	CV-2181		
1C	SIAS A	CV-506	RCP Seals Controlled Bleedoff	<u><7</u>
	SIAS B	CV-505		
1D	NA	SV-6529*	Post Accident Sampling Liquid Return to RC Drain Tank	NA
2A	SIAS A	CV-515	Letdown Line	<u><13</u>
	SIAS B	CV-516		
	NA	7M3-1		
	NA	7M3-1		
2B	NA	CV-517	Charging Line	NA
	NA	CV-518		
	NA	CV-519		
	NA	SP-210M3-2		
	NA	210M3-2		



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 87 AND 69 TO

FACILITY OPERATING LICENSES NOS. DPR-53 AND DPR-69

BALTIMORE GAS AND ELECTRIC COMPANY

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NOS. 1 & 2

DOCKET NOS. 50-317 AND 50-318

Introduction

By application for license amendment dated May 27, 1983, Baltimore Gas and Electric Company (BG&E) requested changes to the Technical Specifications (TS) for Calvert Cliffs Units 1 and 2. The proposed change to TS 3.6.4.1, "Containment Isolation Valves" would reflect a modification to valve 1(2)-SV-6529 changing it from an automatic isolation valve with a required response time (closure) of less than or equal to 7 seconds, to a locked closed isolation valve with no required response time.

Discussion

Modifications associated with TMI Action Item II.B.3.2, "Post Accident Sampling" required BG&E to reassign the reactor coolant drain tank (RCDT) oxygen sample line to provide a path to return sampled liquids to the RCDT. In performing the associated modifications, BG&E changed a valve in this line from an automatic containment isolation valve to a locked closed manual containment isolation valve.

The valves listed in TS Table 3.6-1 (as referenced by TS 3.6.4.1) are containment isolation valves which must close upon detection of accident conditions in order to maintain containment integrity. Valves 1(2)-SV-6529 (Units 1 and 2) had been automatic isolation valves which had been routinely tested to assure a response time of less than or equal to 7 seconds. These valves were modified by removal of the automatic isolation signal and replacement of the actuating hand switches in the control room with key-locked hand switches. In addition, the valves were further controlled by removal of control power with the valves in the closed position.

This proposed change does not increase the probability of occurrence of an accident or malfunction of equipment important to safety previously evaluated in the Updated Final Safety Analysis Report (FSAR). This is because of dual protection for ensuring that the valves are maintained in the closed position, during reactor operation, i.e., the fuses being removed and the key which must be inserted and operated to open the valves under administrative control.

This proposed change does not result in any reduction in the margin of safety. Section 3/4 6.4 of the TS Bases states that, "The Operability of the containment isolation valves ensures that the containment atmosphere will be isolated from the outside environment in the event of a release of radioactive material to the containment atmosphere or pressurization of the containment." By administratively controlling the valves as previously described, the valves are maintained closed when containment integrity is required with the exception of brief periods in the event they are required to be opened to perform the function which is associated with post-accident sampling. The surveillance requirements for containment isolation valves will adequately ensure the valves remain operable. In addition, since these valves would be closed at the initiation of any accident, this change will not result in an accident not previously considered in the FSAR.

Based upon the above, we conclude that proposed change to TS 3.6.4.1, as reflected in TS Table 3.6-1, is acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendments.

Conclusion

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
(2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: September 22, 1983

Principal Contributor:
D. H. Jaffe