

December 30, 1985

Docket Nos. 50-317
and 50-318

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Mr. A. E. Lundvall, Jr.
Vice President - Supply
Baltimore Gas & Electric Company
P. O. Box 1475
Baltimore, Maryland 21203

Dear Mr. Lundvall:

The Commission has issued the enclosed Amendment Nos. 110 and 93 to Facility Operating License Nos. DPR-53 and DPR-69 for Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated November 1, 1985.

The amendments change the Calvert Cliffs Units 1 and 2 Technical Specifications (TS) as follows: (1) TS 6.2.1 and 6.2.2, "Organization", are revised via TS Figures 6.2-1 and 6.2-2 to reflect an extensive corporate reorganization; (2) the titles of a number of individuals, as reflected in TS Section 6.0, "Administrative Controls", are changed as a result of the reorganization; and (3) TS 6.5.1, "Plant Operations and Safety Review Committee (POSRC)", is changed to reflect the elimination of one member from the POSRC.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

/S/

David H. Jaffe, Project Manager
PWR Project Directorate #8
Division of PWR Licensing-B

Enclosures:

1. Amendment No. 110 to DPR-53
2. Amendment No. 93 to DPR-69
3. Safety Evaluation

cc w/enclosure:
See next page

PBD#8
PMKreutzer
12/19/85

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DJaffe
12/19/85

PBD#8
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Mr. A. E. Lundvall, Jr.
Baltimore Gas & Electric Company

Calvert Cliffs Nuclear Power Plant

cc:
Mr. William T. Bowen, President
Calvert County Board of
Commissioners
Prince Frederick, Maryland 20768

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U.S. Nuclear Regulatory Commission
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 110
License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated November 1, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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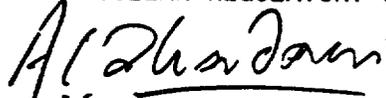
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-53 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 110, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Ashok C. Thadani, Director
PWR Project Directorate #8
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 30, 1985

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Manager Nuclear Operations shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of both units (4 members) or any personnel required for other essential functions during a fire emergency.

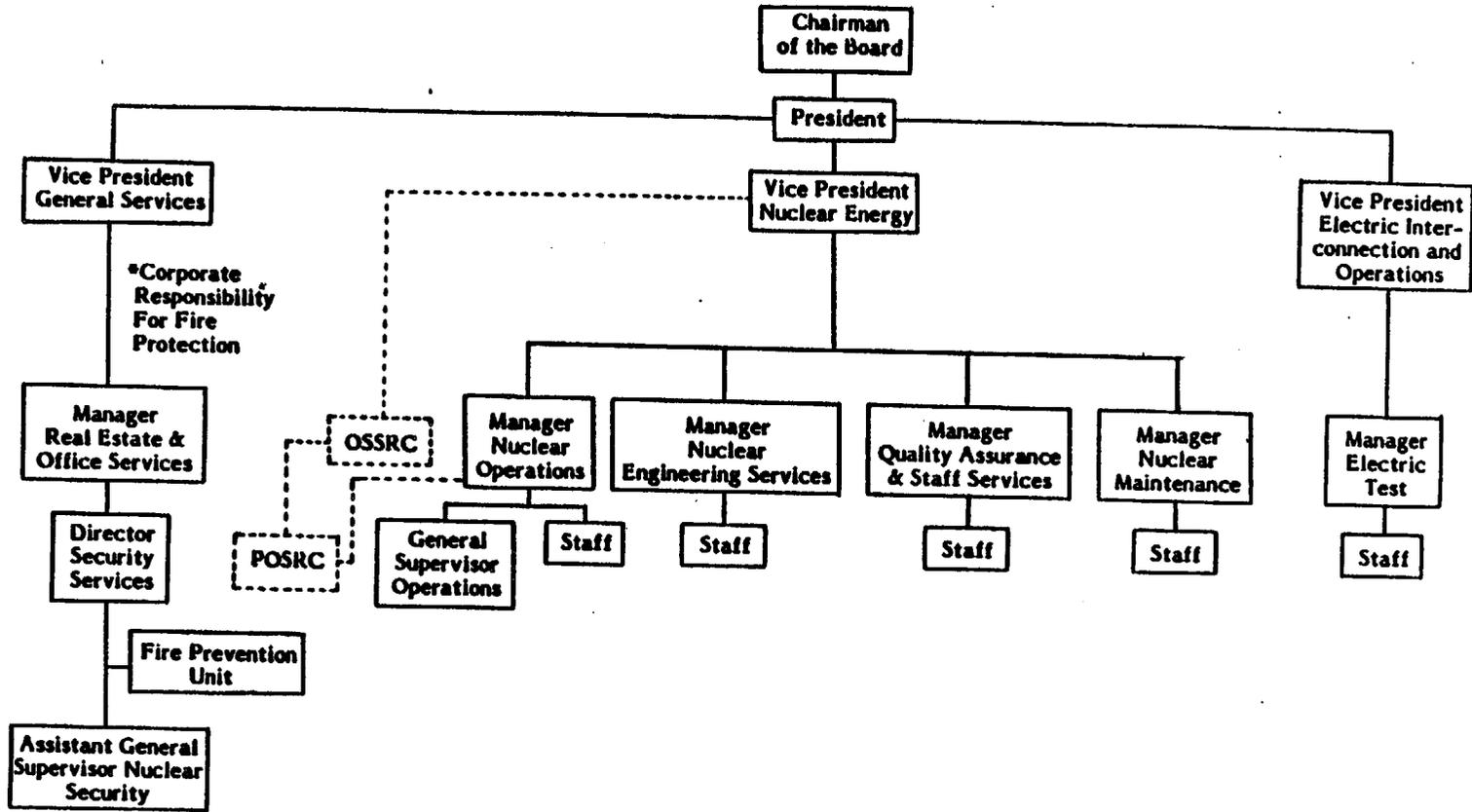
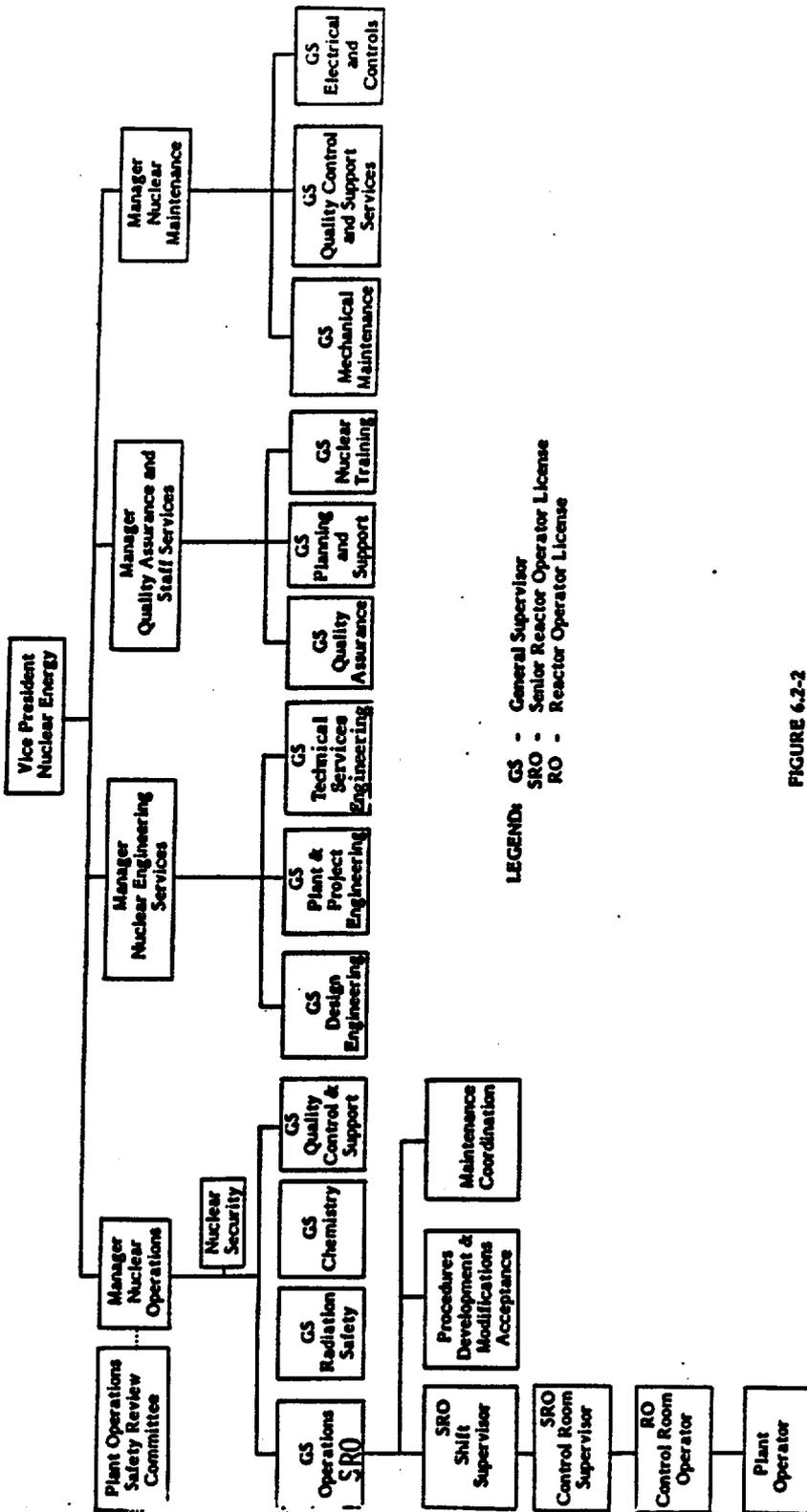


FIGURE 6.2-1
 MANAGEMENT ORGANIZATION CHART
 CALVERT CLIFFS NUCLEAR POWER PLANT
 BALTIMORE GAS & ELECTRIC COMPANY



LEGEND: GS - General Supervisor
 SRO - Senior Reactor Operator License
 RO - Reactor Operator License

FIGURE 6.2-2
 ORGANIZATION CHART (TWO UNIT OPERATION)
 CALVERT CLIFFS NUCLEAR POWER PLANT
 BALTIMORE GAS & ELECTRIC COMPANY

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION #

Condition of Unit 1 - Unit 2 in MODES 1, 2, 3 or 4

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	2*
OL**	3	3
Non-Licensed	3	3
Shift Technical Advisor	1##	1##

Condition of Unit 1 - Unit 2 in MODES 5 or 6

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	1*
OL**	3	2
Non-Licensed	3	3
Shift Technical Advisor	1##	0

TABLE 6.2-1 (Continued)

*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling, supervising CORE ALTERATIONS during fuel reloading.

**Assumes each individual is licensed on each unit.

#Shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2.1.

##The STA shall be qualified/serve as follows:

- (1) With one unit in MODE 5 or 6, and the other unit in MODE 1, 2, 3 or 4, the SOL holder other than the Shift Supervisor shall serve as the STA.
- (2) With one unit defueled and the other unit in MODE 1, 2, 3 or 4, the STA shall be an SOL holder in addition to the one SOL required.
- (3) With both units in MODE 1, 2, 3 or 4, the STA shall be an SOL holder in addition to the two SOLs required.

As an alternative to the above, the STA may be an individual with the following minimum qualifications: a bachelor degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the Radiation Safety Engineer who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and (2) the Shift Technical Advisor who shall have a Bachelor's Degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the General Supervisor - Nuclear Training and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55, as applicable.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the General Supervisor - Quality Control and Support and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975.

6.5 REVIEW AND AUDIT

6.5.1 PLANT OPERATIONS AND SAFETY REVIEW COMMITTEE (POSRC)

FUNCTION

6.5.1.1 The POSRC shall function to advise the Manager-Nuclear Operations on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The POSRC shall be composed of the:

Chairman:	Manager-Nuclear Operations
Member:	General Supervisor - Operations
Member:	General Supervisor - Electrical and Controls
Member:	General Supervisor - Chemistry
Member:	General Supervisor - Mechanical Maintenance
Member:	General Supervisor - Technical Services Engineering
Member:	General Supervisor - Radiation Safety
Member:	General Supervisor - Plant and Project Engineering

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the POSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in POSRC activities at any one time.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The POSRC shall meet at least once per calendar month and as convened by the POSRC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the POSRC shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The POSRC shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Manager - Nuclear Operations to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President Nuclear Energy and to the Chairman of the Off Site Safety Review Committee.
- f. Review of all REPORTABLE EVENTS.
- g. Review of facility operations to detect potential safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Off Site Safety Review Committee.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Off Site Safety Review Committee.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Off Site Safety Review Committee.
- k. Review of any accidental, unplanned or uncontrolled radioactive release that exceeds 25% of the limits of Specification 3.11.1.2, 3.11.2.2 or 3.11.2.3, including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and for forwarding of these reports to the Manager-Nuclear Operations and the Off Site Safety Review Committee.
- l. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL.

ADMINISTRATIVE CONTROLS

AUTHORITY

6.5.1.7 The Plant Operations and Safety Review Committee shall:

- a. Recommend to the Manager-Nuclear Operations written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President-Nuclear Energy and the Off Site Safety Review Committee of disagreement between the POSRC and the Manager-Nuclear Operations; however, the Manager-Nuclear Operations shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The POSRC shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Nuclear Energy and Chairman of the Off Site Safety Review Committee.

6.5.2 OFF SITE SAFETY REVIEW COMMITTEE (OSSRC)

FUNCTION

6.5.2.1 The Off Site Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy and non-destructive examination
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.2.2 The OSSRC shall be composed of at least seven members, including the Chairman. Members of the OSSRC may be from the Nuclear Energy Division or other BG&E organization or from organizations external to BG&E and shall collectively have expertise in all of the areas of 6.5.2.1.

QUALIFICATIONS

6.5.2.3 The Chairman, members and alternate members of the OSSRC shall be appointed in writing by the Vice President - Nuclear Energy and shall have an academic degree in engineering or a physical science, or the equivalent, and in addition shall have a minimum of five years technical experience in one or more areas given in 6.5.2.1. No more than two alternates shall participate as voting members in OSSRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the OSSRC Chairman to provide expert advice to the OSSRC.

MEETING FREQUENCY

6.5.2.5 The OSSRC shall meet at least once per six months.

QUORUM

6.5.2.6 The quorum of the OSSRC necessary for the performance of the OSSRC review and audit functions of these Technical Specifications shall consist of more than half the OSSRC membership or at least four members, whichever is greater. This quorum shall include the Chairman or his appointed alternate and the OSSRC members, including appointed alternates, meeting the requirements of Specification 6.5.2.3. No more than a minority of the quorum shall have line responsibility for operation of the plant.

ADMINISTRATIVE CONTROLS

REVIEW

6.5.2.7 The OSSRC shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provisions of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes in Technical Specifications or this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. All REPORTABLE EVENTS.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meetings minutes of the POSRC.

ADMINISTRATIVE CONTROLS

AUDITS

6.5.2.8.1 Audits of facility activities shall be performed under the cognizance of the OSSRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualification of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR Part 50, at least once per 24 months.
- e. Deleted.
- f. The Safeguards Contingency Plan and implementing procedures at least once per 12 months in accordance with 10 CFR 73.40(d).
- g. Any other area of facility operation considered appropriate by the OSSRC or the Vice President-Nuclear Energy.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.
- k. The radiological environmental monitoring program and the results thereof at least once per 12 months.
- l. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months.
- m. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months.
- n. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months.

ADMINISTRATIVE CONTROLS

6.5.2.8.2 Review of facility activities shall be performed under the cognizance of the OSSRC. These reviews shall encompass:

- a. The Facility Emergency Plan and implementing procedures at least once per 12 months in accordance with 10 CFR Part 50.54(t).

AUTHORITY

6.5.2.9 The OSSRC shall report to and advise the Vice President-Nuclear Energy on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of OSSRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each OSSRC meeting shall be prepared, approved and forwarded to the Vice President-Nuclear Energy within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice President-Nuclear Energy within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Vice President-Nuclear Energy and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the POSRC and the results of this review shall be submitted to the OSSRC and the Vice President-Nuclear Energy.

ADMINISTRATIVE CONTROLS

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least HOT STANDBY within one hour.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Vice President-Nuclear Energy and the OSSRC shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the POSRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the OSSRC and the Vice President-Nuclear Energy within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. The amount of overtime worked by plant staff members performing safety related functions must be limited in accordance with the NRC Policy Statement on working Hours (Generic Letter No. 82-12).
- h. PROCESS CONTROL PROGRAM implementation.
- i. OFFSITE DOSE CALCULATION MANUAL implementation.

6.8.2 Each procedure and administrative policy of 6.8.1 above and changes thereto shall be reviewed by the POSRC and approved by the Manager-Nuclear Operations prior to implementation and reviewed periodically as set forth in administrative procedures.

ADMINISTRATIVE CONTROLS

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the POSRC and approved by the Manager-Nuclear Operations within 14 days of implementation.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

6.9.1.2 The startup report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

ADMINISTRATIVE CONTROLS

- b. A high radiation area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.12.1.a, above, and in addition locked barricades shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained by the Supervisor-Radiation Control Operations and the Operations Shift Supervisor on duty under their separate administrative control.

6.13 SYSTEM INTEGRITY

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and
2. Leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

6.14 IODINE MONITORING

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

1. Training of personnel,
2. Procedures for monitoring, and
3. Provisions for maintenance of sampling and analysis equipment.

6.15 PROCESS CONTROL PROGRAM (PCP)

6.15.1 The PCP shall be approved by the Commission prior to implementation.

6.15.2 Licensee initiated changes to the PCP:

1. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made. This submittal shall contain:
 - a. An evaluation supporting the premise that the change did not reduce the overall conformance of the solidified waste product to existing criteria for solid wastes; and

ADMINISTRATIVE CONTROLS

- b. A reference to the date and the POSRC meeting number in which the change(s) were reviewed and found acceptable to the POSRC.
2. Shall become effective upon review by the POSRC and approval of the Manager-Nuclear Operations.

6.16 OFFSITE DOSE CALCULATION MANUAL (ODCM)

6.16.1 The ODCM shall be approved by the Commission prior to implementation.

6.16.2 Licensee initiated changes to the ODCM:

1. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made effective. This submittal shall contain:
 - a. Sufficient information to support the rationale for the change. Information submitted should consist of a package of those pages of the ODCM to be changed with each page numbered and provided with a change number and/or change date together with appropriate analyses or evaluations justifying the change(s);
 - b. A determination that the change will not reduce the accuracy or reliability of dose calculations or setpoint determinations; and
 - c. Documentation of the fact that the change has been reviewed and found acceptable by the POSRC.
2. Shall become effective upon review by the POSRC and approval of the Manager-Nuclear Operations.

6.17 MAJOR CHANGES TO RADIOACTIVE LIQUID, GASEOUS AND SOLID WASTE TREATMENT SYSTEMS

6.17.1 Licensee initiated major changes to the radioactive waste systems (liquid, gaseous and solid) shall be reported to the Commission in the Semi-annual Radioactive Effluent Release Report for the period in which the modification to the waste system is completed. The discussion of each change shall contain:

- a. A description of the equipment, components and processes involved.
- b. Documentation of the fact that the change including the safety analysis was reviewed and found acceptable by the POSRC.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 93
License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated November 1, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-69 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 93, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Ashok C. Thadani, Director
PWR Project Directorate #8
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 30, 1985

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6.2 ORGANIZATION

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FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of both units (4 members) or any personnel required for other essential functions during a fire emergency.

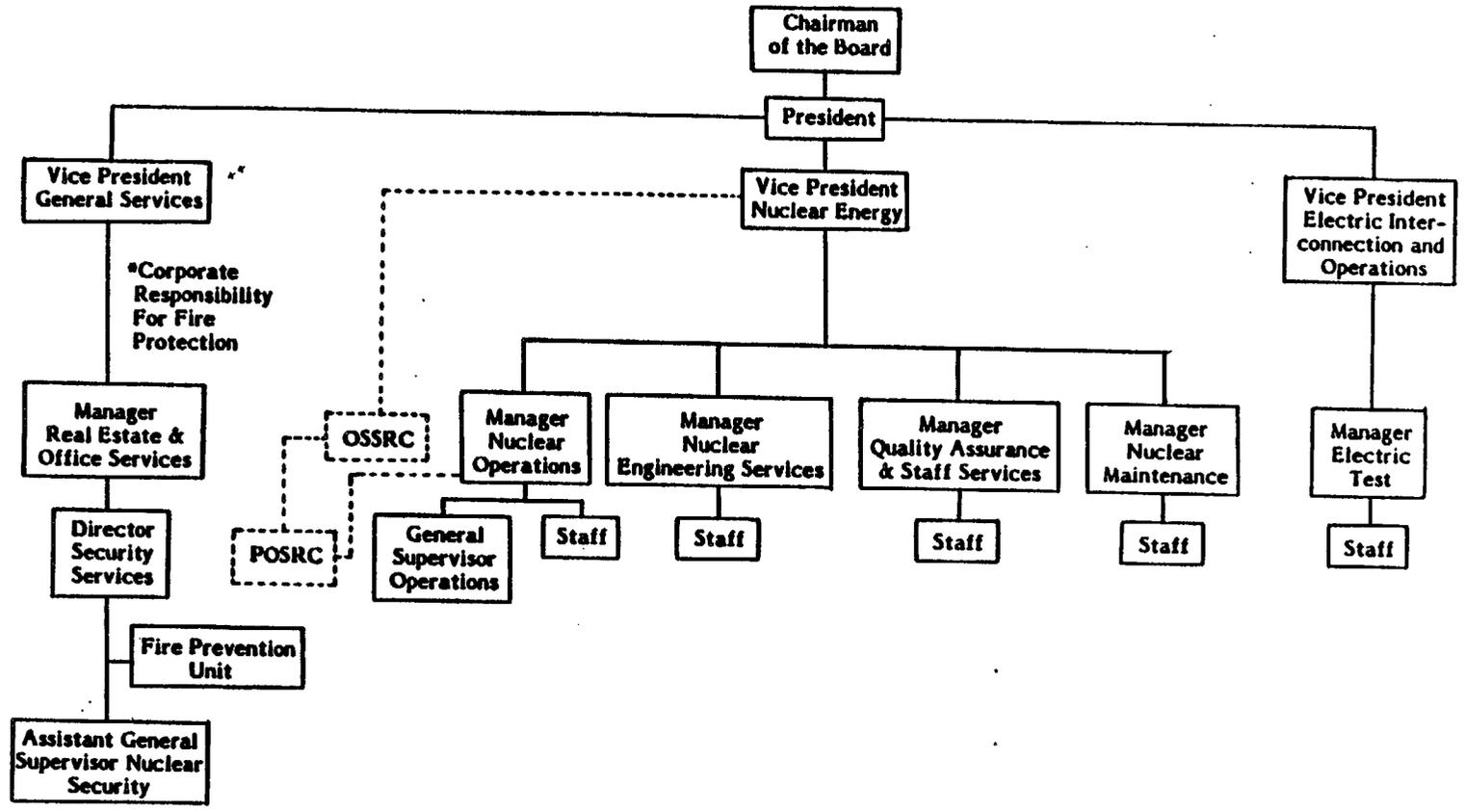
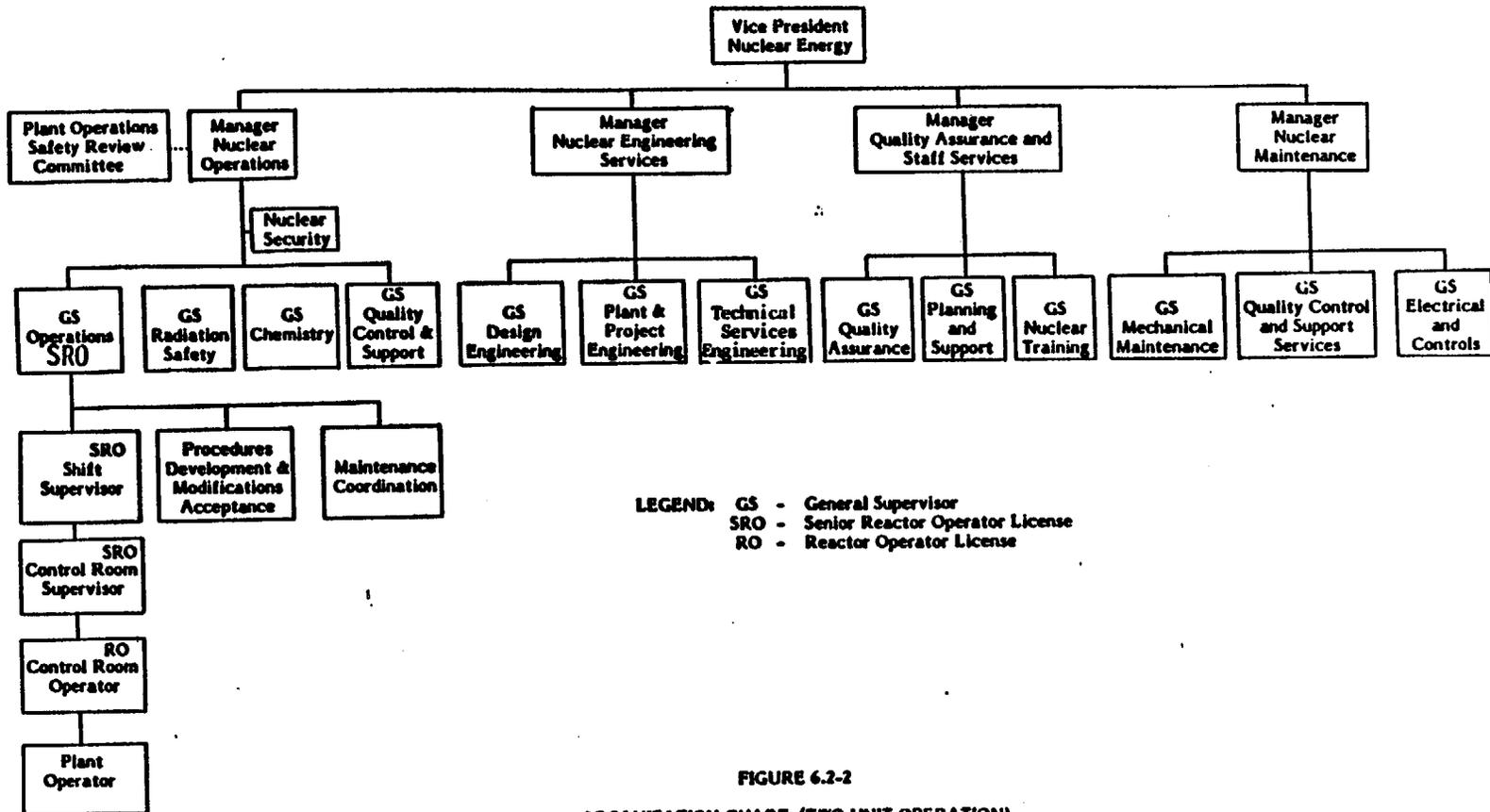


FIGURE 6.2-1
MANAGEMENT ORGANIZATION CHART
CALVERT CLIFFS NUCLEAR POWER PLANT
BALTIMORE GAS & ELECTRIC COMPANY



LEGEND: GS - General Supervisor
 SRO - Senior Reactor Operator License
 RO - Reactor Operator License

FIGURE 6.2-2
 ORGANIZATION CHART (TWO UNIT OPERATION)
 CALVERT CLIFFS NUCLEAR POWER PLANT
 BALTIMORE GAS & ELECTRIC COMPANY

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION #

Condition of Unit 2 - Unit 1 in MODES 1, 2, 3 or 4

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	2*
OL**	3	3
Non-Licensed	3	3
Shift Technical Advisor	1##	1##

Condition of Unit 2 - Unit 1 in MODES 5 or 6

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	1*
OL**	3	2
Non-Licensed	3	3
Shift Technical Advisor	1##	0

TABLE 6.2-1 (Continued)

*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling, supervising CORE ALTERATIONS during fuel reloading.

**Assumes each individual is licensed on each unit.

#Shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2.1.

##The STA shall be qualified/serve as follows:

- (1) With one unit in MODE 5 or 6, and the other unit in MODE 1, 2, 3 or 4, the SOL holder other than the Shift Supervisor shall serve as the STA.
- (2) With one unit defueled and the other unit in MODE 1, 2, 3 or 4, the STA shall be an SOL holder in addition to the one SOL required.
- (3) With both units in MODE 1, 2, 3 or 4, the STA shall be an SOL holder in addition to the two SOLs required.

As an alternative to the above, the STA may be an individual with the following minimum qualifications: a bachelor degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the Radiation Safety Engineer who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and (2) the Shift Technical Advisor who shall have a Bachelor's Degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the General Supervisor - Nuclear Training and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55, as applicable.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the General Supervisor - Quality Control and Support and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975.

6.5 REVIEW AND AUDIT

6.5.1 PLANT OPERATIONS AND SAFETY REVIEW COMMITTEE (POSRC)

FUNCTION

6.5.1.1 The POSRC shall function to advise the Manager-Nuclear Operations on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The POSRC shall be composed of the:

Chairman:	Manager-Nuclear Operations
Member:	General Supervisor - Operations
Member:	General Supervisor - Electrical and Controls
Member:	General Supervisor - Chemistry
Member:	General Supervisor - Mechanical Maintenance
Member:	General Supervisor - Technical Services Engineering
Member:	General Supervisor - Radiation Safety
Member:	General Supervisor - Plant and Project Engineering

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the POSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in POSRC activities at any one time.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The POSRC shall meet at least once per calendar month and as convened by the POSRC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the POSRC shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The POSRC shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Manager-Nuclear Operations to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President-Nuclear Energy and to the Chairman of the Off Site Safety Review Committee.
- f. Review of all REPORTABLE EVENTS.
- g. Review of facility operations to detect potential safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Off Site Safety Review Committee.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Off Site Safety Review Committee.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Off Site Safety Review Committee.
- k. Review of any accidental, unplanned or uncontrolled radioactive release that exceeds 25% of the limits of Specification 3.11.1.2, 3.11.2.2 or 3.11.2.3, including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and for forwarding of these reports to the Manager-Nuclear Operations and the Off Site Safety Review Committee.
- l. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL.

ADMINISTRATIVE CONTROLS

AUTHORITY

- 6.5.1.7 The Plant Operations and Safety Review Committee shall:
- a. Recommend to the Manager-Nuclear Operations written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
 - b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (d) above constitutes an unreviewed safety question.
 - c. Provide written notification within 24 hours to the Vice President - Nuclear Energy and the Off Site Safety Review Committee of disagreement between the POSRC and the Manager-Nuclear Operations; however, the Manager-Nuclear Operations shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The POSRC shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Nuclear Energy and Chairman of the Off Site Safety Review Committee.

6.5.2 OFF SITE SAFETY REVIEW COMMITTEE (OSSRC)

FUNCTION

6.5.2.1 The Off Site Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy and non-destructive examination
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.2.2 The OSSRC shall be composed of at least seven members, including the Chairman. Members of the OSSRC may be from the Nuclear Energy Division or other BG&E organization or from organizations external to BG&E.

QUALIFICATIONS

6.5.2.3 The Chairman, members and alternate members of the OSSRC shall be appointed in writing by the Vice President - Nuclear Energy and shall have an academic degree in engineering or a physical science, or the equivalent, and in addition shall have a minimum of five years technical experience in one or more areas given in 6.5.2.1. No more than two alternates shall participate as voting members in OSSRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the OSSRC Chairman to provide expert advice to the OSSRC.

MEETING FREQUENCY

6.5.2.5 The OSSRC shall meet at least once per six months.

QUORUM

6.5.2.6 The quorum of the OSSRC necessary for the performance of the OSSRC review and audit functions of these Technical Specifications shall consist of more than half the OSSRC membership or at least four members, whichever is greater. This quorum shall include the Chairman or his appointed alternate and the OSSRC members, including appointed alternates, meeting the requirements of Specification 6.5.2.3. No more than a minority of the quorum shall have line responsibility for operation of the plant.

ADMINISTRATIVE CONTROLS

REVIEW

6.5.2.7 The OSSRC shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes in Technical Specifications or this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. All REPORTABLE EVENTS.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meetings minutes of the POSRC.

ADMINISTRATIVE CONTROLS

AUDITS

6.5.2.8.1 Audits of facility activities shall be performed under the cognizance of the OSSRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualification of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR Part 50, at least once per 24 months.
- e. Deleted.
- f. The Safeguards Contingency Plan and implementing procedures at least once per 12 months in accordance with 10 CFR 73.40(d).
- g. Any other area of facility operation considered appropriate by the OSSRC or the Vice President-Nuclear Energy.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.
- k. The radiological environmental monitoring program and the results thereof at least once per 12 months.
- l. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months.
- m. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months.
- n. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months.

ADMINISTRATIVE CONTROLS

6.5.2.8.2 Review of facility activities shall be performed under the cognizance of the OSSRC. These reviews shall encompass:

- a. The Facility Emergency Plan and implementing procedures at least once per 12 months in accordance with 10 CFR Part 50.54(t).

AUTHORITY

6.5.2.9 The OSSRC shall report to and advise the Vice President - Nuclear Energy on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of OSSRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each OSSRC meeting shall be prepared, approved and forwarded to the Vice President - Nuclear Energy within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice President - Nuclear Energy within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Vice President - Nuclear Energy and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the POSRC and the results of this review shall be submitted to the OSSRC and the Vice President - Nuclear Energy.

ADMINISTRATIVE CONTROLS

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least HOT STANDBY within one hour.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Vice President - Nuclear Energy and the OSSRC shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the POSRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the OSSRC and the Vice President - Nuclear Energy within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. The amount of overtime worked by plant staff members performing safety related functions must be limited in accordance with the NRC Policy Statement on working Hours (Generic Letter No. 82-12).
- h. PROCESS CONTROL PROGRAM implementation.
- i. OFFSITE DOSE CALCULATION MANUAL implementation.

6.8.2 Each procedure and administrative policy of 6.8.1 above and changes thereto shall be reviewed by the POSRC and approved by the Manager-Nuclear Operations prior to implementation and reviewed periodically as set forth in administrative procedures.

ADMINISTRATIVE CONTROLS

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the POSRC and approved by the Manager-Nuclear Operations within 14 days of implementation.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

6.9.1.2 The startup report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

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- b. A high radiation area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.12.1.a, above, and in addition locked barricades shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained by the Supervisor-Radiation Control Operations and the Operations Shift Supervisor on duty under their separate administrative control.

6.13 SYSTEM INTEGRITY

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and
2. Leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

6.14 IODINE MONITORING

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

1. Training of personnel,
2. Procedures for monitoring, and
3. Provisions for maintenance of sampling and analysis equipment.

6.15 PROCESS CONTROL PROGRAM (PCP)

6.15.1 The PCP shall be approved by the Commission prior to implementation.

6.15.2 Licensee initiated changes to the PCP:

1. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made. This submittal shall contain:

ADMINISTRATIVE CONTROLS

- a. An evaluation supporting the premise that the change did not reduce the overall conformance of the solidified waste product to existing criteria for solid wastes; and
 - b. A reference to the date and the POSRC meeting number in which the change(s) were reviewed and found acceptable to the POSRC.
2. Shall become effective upon review by the POSRC and approval of the Manager-Nuclear Operations.

6.16 OFFSITE DOSE CALCULATION MANUAL (ODCM)

6.16.1 The ODCM shall be approved by the Commission prior to implementation.

6.16.2 Licensee initiated changes to the ODCM:

1. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made effective. This submittal shall contain:
 - a. Sufficient information to support the rationale for the change. Information submitted should consist of a package of those pages of the ODCM to be changed with each page numbered and provided with a change number and/or change date together with appropriate analyses or evaluations justifying the change(s);
 - b. A determination that the change will not reduce the accuracy or reliability of dose calculations or setpoint determinations; and
 - c. Documentation of the fact that the change has been reviewed and found acceptable by the POSRC.
2. Shall become effective upon review by the POSRC and approval of the Manager-Nuclear Operations.

6.17 MAJOR CHANGES TO RADIOACTIVE LIQUID, GASEOUS AND SOLID WASTE TREATMENT SYSTEMS

6.17.1 Licensee initiated major changes to the radioactive waste systems (liquid, gaseous and solid) shall be reported to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the modification to the waste system is completed. The discussion of each change shall contain:

- a. A description of the equipment, components and processes involved.
- b. Documentation of the fact that the change including the safety analysis was reviewed and found acceptable by the POSRC.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 110 AND 93

TO FACILITY OPERATING LICENSE NOS. DPR-53 AND DPR-69

BALTIMORE GAS AND ELECTRIC COMPANY

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-317 AND 50-318

Introduction

By application for license amendment dated November 1, 1985, Baltimore Gas & Electric Company (BG&E) requested changes to the Technical Specifications (TS) for Calvert Cliffs Units 1 and 2 as follows: (1) TS 6.2.1 and 6.2.2, "Organization", would be changed via TS Figures 6.2-1 and 6.2-2 to reflect an extensive corporate reorganization; (2) the titles of a number of individuals, as reflected in TS Section 6.0, "Administrative Controls", would be changed as a result of the reorganization; and (3) TS 6.5.1, "Plant Operations and Safety Review Committee (POSRC)", would be changed to reflect the elimination of one member from the POSRC.

Discussion and Evaluation

The reorganization proposed by BG&E represents a significant change in the off-site and on-site organizational structure as reflected in TS 6.2.1 and 6.2.2, respectively. The Vice President-Supply and Vice President-Engineering and Construction positions would be eliminated. A new position of Vice President-Nuclear Energy would be created which would encompass the present responsibilities of the above two eliminated positions with respect to BG&E's nuclear program. Reporting to the Vice President-Nuclear Energy would be four managers: Manager-Nuclear Operations (The Manager-Nuclear Operations would assume the current responsibilities of the Plant Superintendent with respect to nuclear safety); Manager-Nuclear Engineering Services (The Manager-Nuclear Engineering Services would be established to consolidate all of the engineering support staff into a single department); Manager-Quality Assurance and Staff Services (The Manager-Quality Assurance and Staff Services would assume the responsibilities of the present Manager-Quality Assurance with exception of the Quality Control function); and Manager-Nuclear Maintenance (The Manager-Nuclear Maintenance would be responsible for the electrical, instrument, and control, and mechanical maintenance/modification crafts and technicians and their associated quality control inspectors.)

The proposed reorganization would be expected to improve management at Calvert Cliffs by allowing key functions to report to a single BG&E Vice President while maintaining independent oversight. Since an equivalent or

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improved level of management would be maintained at Calvert Cliffs, no degradation in the maintenance or testing of safety-related equipment or structures would occur; thus, the proposed changes to TS 6.2.1 and 6.2.2 are acceptable.

Consistent with the proposed changes to TS 6.2.1 and 6.2.2, the licensee has proposed a number of changes to TS Section 6.0 to reflect changes in job titles. In all cases, the job function remains the same and only the title would change. These proposed changes to Section 6.0 are administrative in nature in light of the proposed changes concerning TS 6.2.1 and 6.2.2 and are thus, acceptable.

Finally, the licensee has proposed to eliminate one member from the POSRC as specified in TS 6.5.1. This committee provides a substantial degree of oversight concerning safety-related activities related to plant operation; TS 6.5.1.6 enumerates these responsibilities. At the present time, TS 6.5.1.2 requires that the POSRC be composed of eight members and a chairman. The licensee has proposed that the POSRC be composed of seven members and a chairman.

The proposed elimination of a POSRC member does not eliminate any technical discipline necessary to carry out the function of the POSRC or in any other way hamper the function of the POSRC. Since the POSRC will continue to maintain adequate oversight concerning activities that affect safety-related equipment or structures, the change in TS 6.5.1.2 which reduces the POSRC membership from eight to seven members is acceptable.

Environmental Consideration

These amendments relate to changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 30, 1985

Principal Contributors:

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