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Vice President

January 31, 2001

U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Document Control Desk

Subject: Oconee Nuclear Station
Docket Numbers 50-269, 270, and 287
Technical Specification Bases (TSB) Changes

Please find attached revision to TSB 3.6.1, which was approved by Station Management on January 17, 2001 and implemented on January 25, 2001. The change revises the reference section of the Bases for 3.6.1, Containment, to reflect the Updated Final Safety Analysis Report (UFSAR) Chapter 18, Table 18-1. As part of the License Renewal, Containment Leak Rate Testing is credited with managing aging of steel components of the Reactor Building Containment for the period of extended operation.

Attachment 1 contains the new Technical Specification Bases page and Attachment 2 contains the markup version of the Bases page.

If any additional information is needed, please contact Larry E. Nicholson, (864-885-3292)

Very truly yours,

W. R. McCollum, Jr., Vice President
Oconee Nuclear Site

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Attachment 1

BASES

SURVEILLANCE
REQUIREMENTS
(continued)

SR 3.6.1.2

Maintaining the containment OPERABLE requires compliance with the Type B and C leakage rate test requirements of 10 CFR 50, Appendix J, Option A (Ref. 1), as modified by approved exemptions. As left leakage prior to the first startup after performing a required 10 CFR 50, Appendix J, Option A, leakage test is required to be $< 0.6 L_a$ for combined Type B and C leakage. At all other times between required leakage rate tests, the acceptance criteria is based on an overall Type A leakage limit of $\leq 1.0 L_a$. At $\leq 1.0 L_a$ the offsite dose consequences are bounded by the assumptions of the safety analysis. SR Frequencies are as required by Appendix J, Option A, as modified by approved exemptions. Thus, SR 3.0.2 (which allows Frequency extensions) does not apply. These periodic testing requirements verify that the containment leakage rate does not exceed the leakage rate assumed in the safety analysis.

SR 3.6.1.3

This SR ensures that the structural integrity of the containment will be maintained in accordance with the provisions of the Containment Tendon Surveillance Program. Testing and Frequency are as described in Specification 5.5.7, "Pre-stressed Concrete Containment Tendon Surveillance Program."

REFERENCES

1. 10 CFR 50, Appendix J, Option A and B.
 2. UFSAR, Sections 15.13 and 15.14.
 3. UFSAR, Section 6.2.
 4. 10 CFR 50.36.
 5. UFSAR Chapter 18, Table 18-1.
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Attachment 2

BASES

**SURVEILLANCE
REQUIREMENTS**
(continued)

SR 3.6.1.2

Maintaining the containment OPERABLE requires compliance with the Type B and C leakage rate test requirements of 10 CFR 50, Appendix J, Option A (Ref. 1), as modified by approved exemptions. As left leakage prior to the first startup after performing a required 10 CFR 50, Appendix J, Option A, leakage test is required to be $< 0.6 L_a$ for combined Type B and C leakage. At all other times between required leakage rate tests, the acceptance criteria is based on an overall Type A leakage limit of $\leq 1.0 L_a$. At $\leq 1.0 L_a$ the offsite dose consequences are bounded by the assumptions of the safety analysis. SR Frequencies are as required by Appendix J, Option A, as modified by approved exemptions. Thus, SR 3.0.2 (which allows Frequency extensions) does not apply. These periodic testing requirements verify that the containment leakage rate does not exceed the leakage rate assumed in the safety analysis.

SR 3.6.1.3

This SR ensures that the structural integrity of the containment will be maintained in accordance with the provisions of the Containment Tendon Surveillance Program. Testing and Frequency are as described in Specification 5.5.7, "Pre-stressed Concrete Containment Tendon Surveillance Program."

REFERENCES

1. 10 CFR 50, Appendix J, Option A and B.
2. UFSAR, Sections 15.13 and 15.14.
3. UFSAR, Section 6.2.
4. 10 CFR 50.36.
5. *UFSAR Chapter 18, Table 18-1.*