

AUG 2 1984

Docket Nos. 50-317
and 50-318

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Mr. A. E. Lundvall, Jr.
Vice President - Supply
Baltimore Gas & Electric Company
P. O. Box 1475
Baltimore, Maryland 21203

Dear Mr. Lundvall:

The Commission has issued the enclosed Amendment Nos. 96 and 77 to Facility Operating License Nos. DPR-53 and DPR-69 for Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications in partial response to your application dated April 9, 1984. The remaining issues will be addressed in future correspondence.

The amendments revise the Technical Specifications to reflect: (1) changes to the administrative requirements of TS 6.2, "Organization", associated with corporate organizational changes, (2) installation of additional fire protection instrumentation, and (3) a change in the administrative control of access to high radiation areas.

A copy of the related Safety Evaluation is enclosed. The notice of issuance will be included in the Commission's next monthly Federal Register notice.

Sincerely,

Original signed by

David H. Jaffe, Project Manager
Operating Reactors Branch #3
Division of Licensing

Enclosures:

1. Amendment No. 96 to DPR-53
2. Amendment No. 77 to DPR-69
3. Safety Evaluation

cc w/enclosure:
See next page

8408240015 840802
PDR ADDCK 05000317
P PDR

ORB#3:DL
PMKreutzer
7/25/84

ORB#3:DL
DJaffe/pn
7/26/84

ORB#3:DL
JRMiller
7/27/84

OELD
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7/27/84

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Jed
Signed 8/2 Jed

AD:OR:DL
GCLainas
8/1/84



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 96
License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated April 8, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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PDR ADDCK 05000317
P PDR

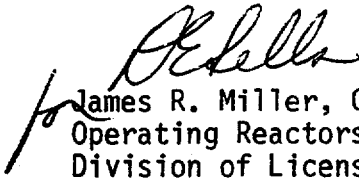
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-53 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 96, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective within 30 days of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 2, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 96

FACILITY OPERATING LICENSE NO. DPR-53

DOCKET NO. 50-317

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are provided to maintain document completeness.

Pages

3/4 3-45

3/4 3-46

6-2

6-3

6-21

TABLE 3.3-11
FIRE DETECTION INSTRUMENTS

UNIT 1

<u>INSTRUMENT LOCATION</u>	<u>MINIMUM INSTRUMENTS OPERABLE</u>		
	<u>HEAT</u>	<u>FLAME</u>	<u>SMOKE</u>
Spent Fuel Pool Heat Exchanger Room 320			3
Main Control Room 405			6
Control Room Vent Duct "A"			2
Main Plant Exhaust Equip Room 524			8
Control Room HVAC Equip Room 512			4
Passage and Filter Room 323			3
Unit 1 Cont SW Elec Pen Area*	4		
Unit 1 Cont NE Elec Pen Area*	4		
Unit 1 Cont East RCPS*	16		
Unit 1 Cont West RCPS*	16		
Control Room Vent Duct "B"			1
West Passage 319 Elev 27'-0"			6
E/W Corridor 104, 100 and 106 - Elev (-) 10'-0"			5
Intake Structure			48
Unit 1 Waste Proc Control Room 111			1
Coolant Waste Rec/Mon TK Pp Room 110			2
11 Diesel Generator**	2		
12 Diesel Generator**	2		
Unit 1 Cable Tunnel Elev 83'-0"			4
Cable Chase 1A			1
Cable Chase 1B			1
Unit 1 C.S.R. & Cable Chase 1C**	2		10
Unit 1 Personnel Access Area Room 525			3
Unit 1 Switchgear Elev 27'-0" Room 317**			6
Unit 1 Switchgear Elev 45'-0" Room 430**			8
Unit 1 Elec Equip Room 529			3
Unit 1 East Elec Pen Room 429			3
Unit 1 West Elec Pen Room 423			3
Unit 1 Refueling Water TK Pump Room 439			2
Unit 1 East Piping Pen Rooms 227 and 316		3	5
Unit 1 Purge Air Supply Room 318			2
Unit 1 West Piping Pen Rooms 221 and 326		2	3
Unit 1 Letdown Heat Exchanger Room 324			1
Unit 1 Volume Control TK Room 218			1
Unit 1 ECCS Pump Rooms 118 and 122			7
Unit 1 Coolant Waste Rec TK Room 114 and 112		4	
Unit 1 ECCS Pump Rooms 119 and 122			7
Unit 1 Elev 27'-0" Swgr Room Vent Duct	1		
Unit 1 Elev 45'-0" swgr Room Vent Duct	1		

*Detection instruments located within the containment are not required to be OPERABLE during the performance of Type A Containment Leakage Rate Tests.

**Detectors which automatically actuate fire suppression systems.

TABLE 3.3-11 (Continued)

FIRE DETECTION INSTRUMENTS

UNIT 1

<u>INSTRUMENT LOCATION</u>	<u>MINIMUM INSTRUMENTS OPERABLE</u>		
	<u>HEAT</u>	<u>FLAME</u>	<u>SMOKE</u>
Main Steam Piping Room 315A			6
Hot Machine Shop 223			4
Battery Room 304 and 301			3
Misc. Waste Monitor Tank Room 113			1
Charging Pump Room 115			3
East Piping Room 428			7
North South Corridor 410			4
Spent Fuel Pool 530		5	17
Radiation Chem., Lab Office, Rm. 513, 518 and 519, Corridor 521, 522 and 534	1		16
Cask and Equipment Loading Area Rm. 419, 420, 425, and 426		3	22
Spent Fuel Vent Equip. Room 520			2
Component Cool Room 228			8
Radiation Exchange Vent Equip. Room 225			4
Boric Hold Tank & Pump Room 217			2
Reactor Cooling Pump Room 216			1
Hot Instrument Shop Room 222			2
Service Water Room 226		3	6
East Piping Room 224			10
Corridor 200, 209, and 210			13
Solid Waste Room 418 and 417		2	3
Spent Resin Metering Tank Room 441			1
Waste Gas Equipment Room 207			1
Auxiliary Feed Tank Room 603			2
Misc. Waste Equipment Room 536			3
Corridor 308			6
N/S Corridor Room 410			4
N/S Corridor Room 308			6
Degasifier Pump Room 220			1
Waste Gas Compressor Room 208			2

TABLE 3.3-11 (Continued)

FIRE DETECTION INSTRUMENTS

UNIT 1

<u>INSTRUMENT LOCATION</u>	<u>MINIMUM INSTRUMENTS OPERABLE</u>		
	<u>HEAT</u>	<u>FLAME</u>	<u>SMOKE</u>
Main Steam Piping Room 315A			6
Hot Machine Shop 223			4
Battery Room 304 and 301			3
Misc. Waste Monitor Tank Room 113			1
Charging Pump Room 115			3
East Piping Room 428			7
North South Corridor 410			4
Spent Fuel Pool 530		5	17
Radiation Chem., Lab Office, Rm. 513, 518 and 519, Corridor 521, 522 and 534	1		16
Cask and Equipment Loading Area Rm. 419, 420, 425, and 426		3	22
Spent Fuel Vent Equip. Room 520			2
Component Cool Room 228			8
Radiation Exchange Vent Equip. Room 225			4
Boric Hold Tank & Pump Room 217			2
Reactor Cooling Pump Room 216			1
Hot Instrument Shop Room 222			2
Service Water Room 226		3	6
East Piping Room 224			10
Corridor 200, 209, and 210			13
Solid Waste Room 418 and 417		2	3
Spent Resin Metering Tank Room 441			1
Waste Gas Equipment Room 207			1
Auxiliary Feed Tank Room 603			2
Misc. Waste Equipment Room 536			3
Corridor 308			6
N/S Corridor Room 410			4
N/S Corridor Room 308			6
Degasifier Pump Room 220			1
Waste Gas Compressor Room 208			2

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Plant Superintendent shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of both units (4 members) or any personnel required for other essential functions during a fire emergency.

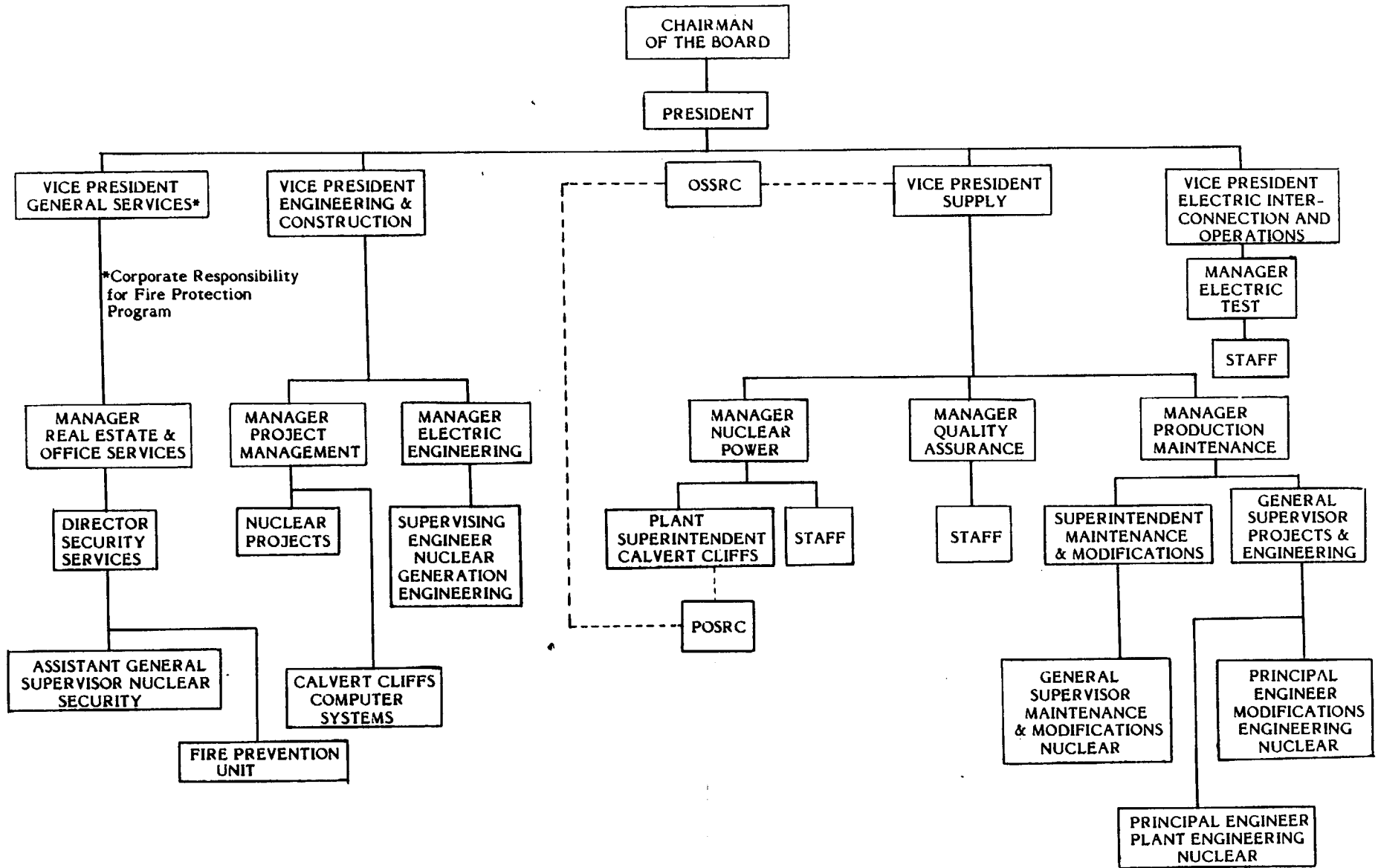


FIGURE 6.2-1
 MANAGEMENT ORGANIZATION CHART
 CALVERT CLIFFS NUCLEAR POWER PLANT
 BALTIMORE GAS & ELECTRIC COMPANY

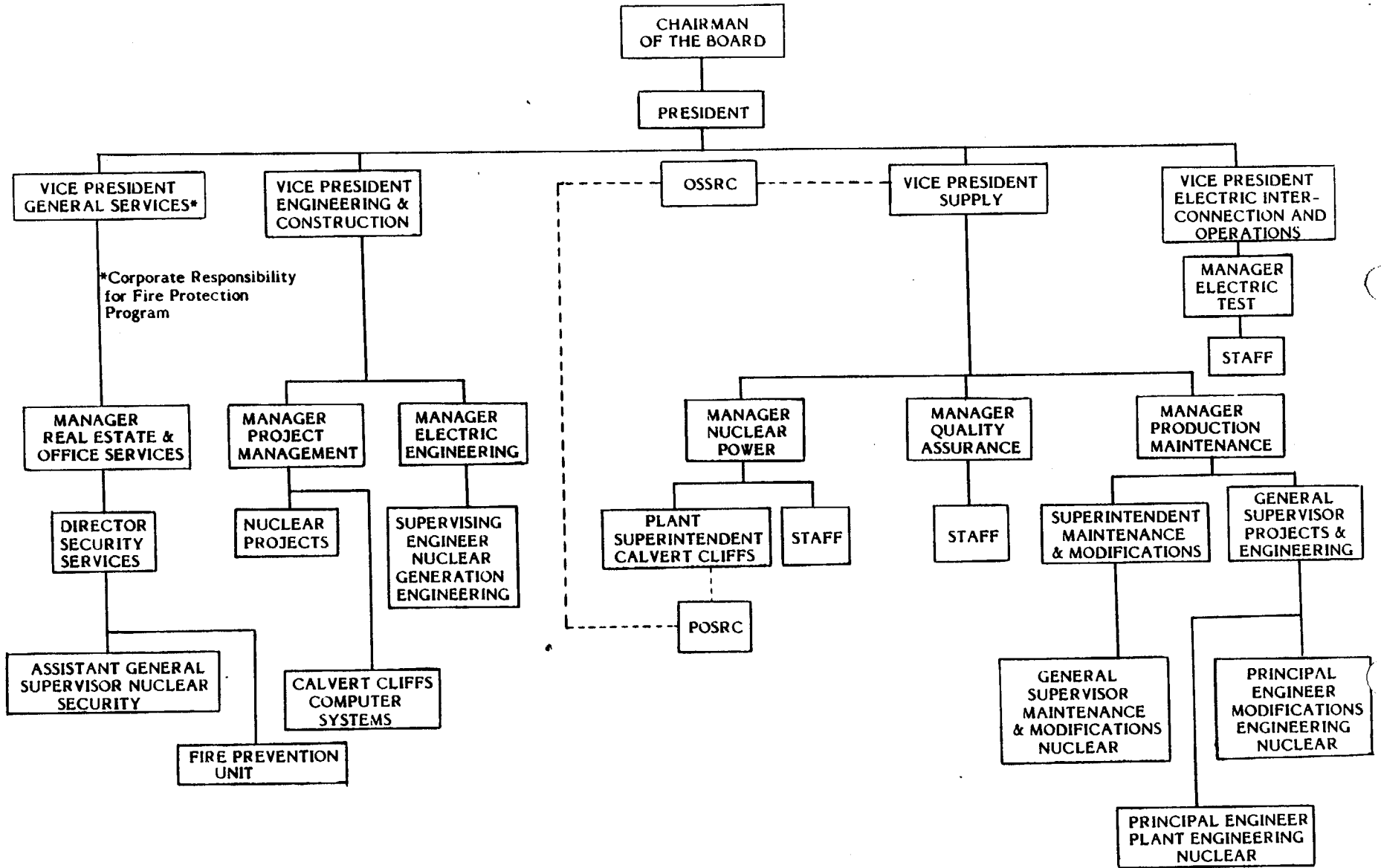


FIGURE 6.2-1
 MANAGEMENT ORGANIZATION CHART
 CALVERT CLIFFS NUCLEAR POWER PLANT
 BALTIMORE GAS & ELECTRIC COMPANY

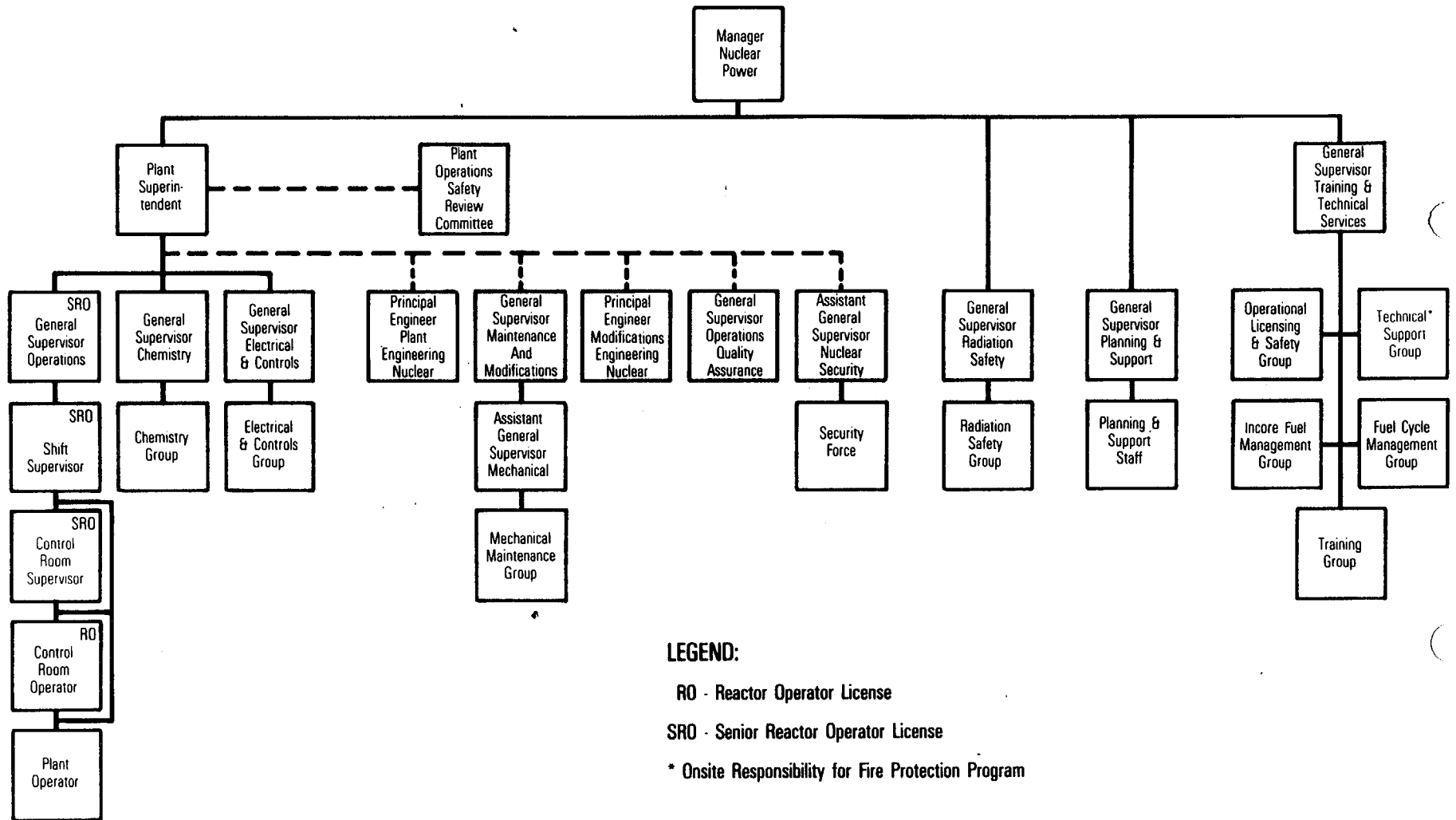


Figure 6.2.2
Organization Chart (Two Unit Operation) - Calvert Cliffs Nuclear Power Plant
Baltimore Gas and Electric Company

TABLE 6.2-1
MINIMUM SHIFT CREW COMPOSITION #

Condition of Unit 1 - Unit 2 in MODES 1, 2, 3 or 4

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	2*
OL**	3	3
Non-Licensed	3	3
Shift Technical Advisor	1##	1##

Condition of Unit 1 - Unit 2 in MODES 5 or 6

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	1*
OL**	3	2
Non-Licensed	3	3
Shift Technical Advisor	1##	0

TABLE 6.2-1
MINIMUM SHIFT CREW COMPOSITION #

Condition of Unit 1 - Unit 2 in MODES 1, 2, 3 or 4

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	2*
OL**	3	3
Non-Licensed	3	3
Shift Technical Advisor	1##	1##

Condition of Unit 1 - Unit 2 in MODES 5 or 6

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	1*
OL**	3	2
Non-Licensed	3	3
Shift Technical Advisor	1##	0

ADMINISTRATIVE CONTROLS

- b. A high radiation area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.12.1.a, above, and in addition locked barricades shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained by the Supervisor-Radiation Control and the Operations Shift Supervisor on duty under their separate administrative control.

6.13 ENVIRONMENTAL QUALIFICATION

6.13.1 By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of: Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class IE Electrical Equipment in Operating Reactors" (DOR Guidelines); or NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment", December 1979. Copies of these documents are attached to Order for Modification of Licenses DPR-53 and DPR-69 dated October 24, 1980.

6.13.2 By no later than December 1, 1980, complete and auditable records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

6.14 SYSTEM INTEGRITY

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and
2. Leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

ADMINISTRATIVE CONTROLS

6.15 IODINE MONITORING

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

1. Training of personnel,
2. Procedures for monitoring, and
3. Provisions for maintenance of sampling and analysis equipment.

ADMINISTRATIVE CONTROLS

6.15 IODINE MONITORING

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

1. Training of personnel,
2. Procedures for monitoring, and
3. Provisions for maintenance of sampling and analysis equipment.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 77
License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated April 9, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

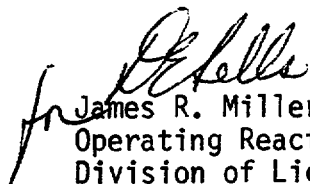
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-69 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 77, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective within 30 days of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 8, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 77

FACILITY OPERATING LICENSE NO. DPR-69

DOCKET NO. 50-318

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are provided to maintain document completeness.

Pages

3/4 3-45

3/4 3-46

6-2

6-3

6-21

TABLE 3.3-11
FIRE DETECTION INSTRUMENTS

UNIT 2

<u>INSTRUMENT LOCATION</u>	<u>MINIMUM INSTRUMENTS OPERABLE*</u>		
	<u>HEAT</u>	<u>FLAME</u>	<u>SMOKE</u>
Unit 2 East Elec Pen Room 409			3
Unit 2 West Elec Pen Room 414			3
Unit 2 Switchgear Elev 27'-0" Room 311**			6
Unit 2 Switchgear Elev 45'-0" Room 407**			8
Unit 2 Elec Equip Room 532			3
Unit 2 Cont SE Elec Pen Area*	4		
Unit 2 Cont NW Elec Pen Area*	4		
Unit 2 Cont East RCPS*	16		
Unit 2 Cont West RCPS*	16		
Unit 2 Main Plant Exh Equip Room 526			8
Unit 2 Personnel Access Area Room 527			3
Cable Tunnel U-2 Elev 83'-0"			4
Cable Chase 2A			1
Cable Chase 2B			1
Unit 2 C.S.R. & Cable Chase 2C**	2		10
Unit 2 Letdown Heat Exchanger Room 322			1
Unit 2 Volume Control Tank Room 214			1
Unit 2 Cool Waste Rec TK Room 107 and 109		4	
Unit 2 ECCS Pump Rooms 101 and 120			7
Unit 2 Pump Room 108 Elev (-) 10'-0"			1
Unit 2 Intake Structure			48
Unit 2 Elev 27'-0" Swgr Room Vent Duct	1		
Unit 2 Elev 45'-0" Swgr Room Vent Duct	1		
Unit 2 ECCS Pp Rooms 102 and 120			7
21 Diesel Generator **	2		
Unit 2 Refueling Water Tk Pp Room 440.			2
Unit 2 East Pp Pen Rooms 206 and 310		3	5
Unit 2 Purge Air Supply Room 312			2
Unit 2 West Piping Pen Rooms 211 and 321		2	3

*Detection instruments located within the containment are not required to be OPERABLE during the performance of Type A Containment Leakage Rate Tests.
**Detectors which automatically actuate fire suppression systems.

TABLE 3.3-11 (Continued)

FIRE DETECTION INSTRUMENTS

<u>INSTRUMENT LOCATION</u>	<u>MINIMUM INSTRUMENT OPERABLE</u>		
	<u>HEAT</u>	<u>FLAME</u>	<u>SMOKE</u>
Main Steam Piping Room 309			6
East Piping Area Room 203			10
Charging Pump Room 105			3
Battery Room 307 and 305			3
Misc. Waste Monitor Tank Room			1
East Piping Area Room 408			7
Component Cooling Room 201			9
Radiation Exchange Equip. Room 204			4
Boric Acid Tank and Pump Room 215			2
Reactor Cooling Pump Room 216A			2
Service Water Room 205		3	6
Auxiliary Feed Pump Room 605			2
Degasifier Pump Room			1

TABLE 3.3-11 (Continued)

FIRE DETECTION INSTRUMENTS

<u>INSTRUMENT LOCATION</u>	<u>MINIMUM INSTRUMENT OPERABLE</u>		
	<u>HEAT</u>	<u>FLAME</u>	<u>SMOKE</u>
Main Steam Piping Room 309			6
East Piping Area Room 203			10
Charging Pump Room 105			3
Battery Room 307 and 305			3
Misc. Waste Monitor Tank Room			1
East Piping Area Room 408			7
Component Cooling Room 201			9
Radiation Exchange Equip. Room 204			4
Boric Acid Tank and Pump Room 215			2
Reactor Cooling Pump Room 216A			2
Service Water Room 205		3	6
Auxiliary Feed Pump Room 605			2
Degasifier Pump Room			1

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Plant Superintendent shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of both units (4 members) or any personnel required for other essential functions during a fire emergency.

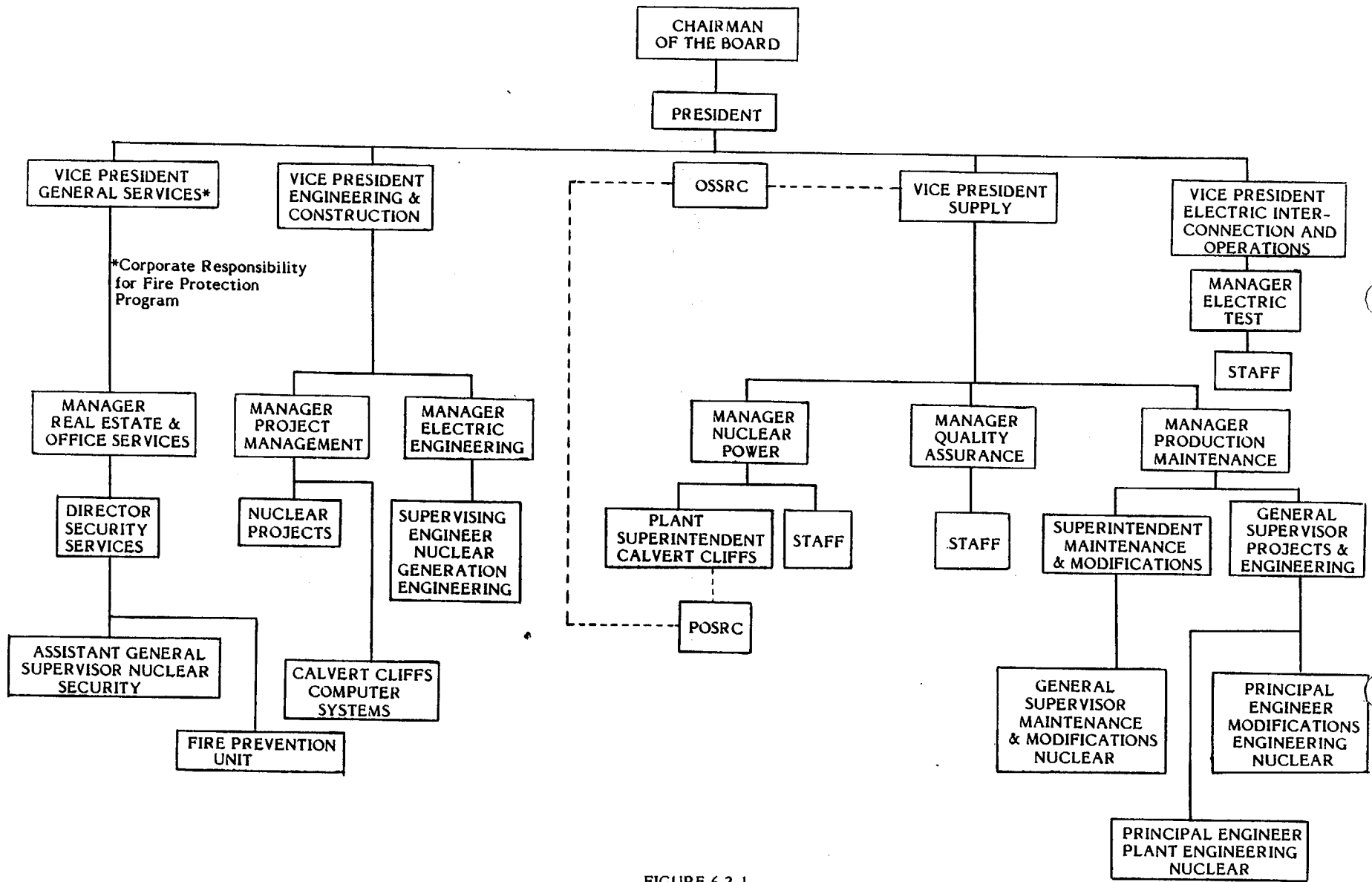


FIGURE 6.2-1
 MANAGEMENT ORGANIZATION CHART
 CALVERT CLIFFS NUCLEAR POWER PLANT
 BALTIMORE GAS & ELECTRIC COMPANY

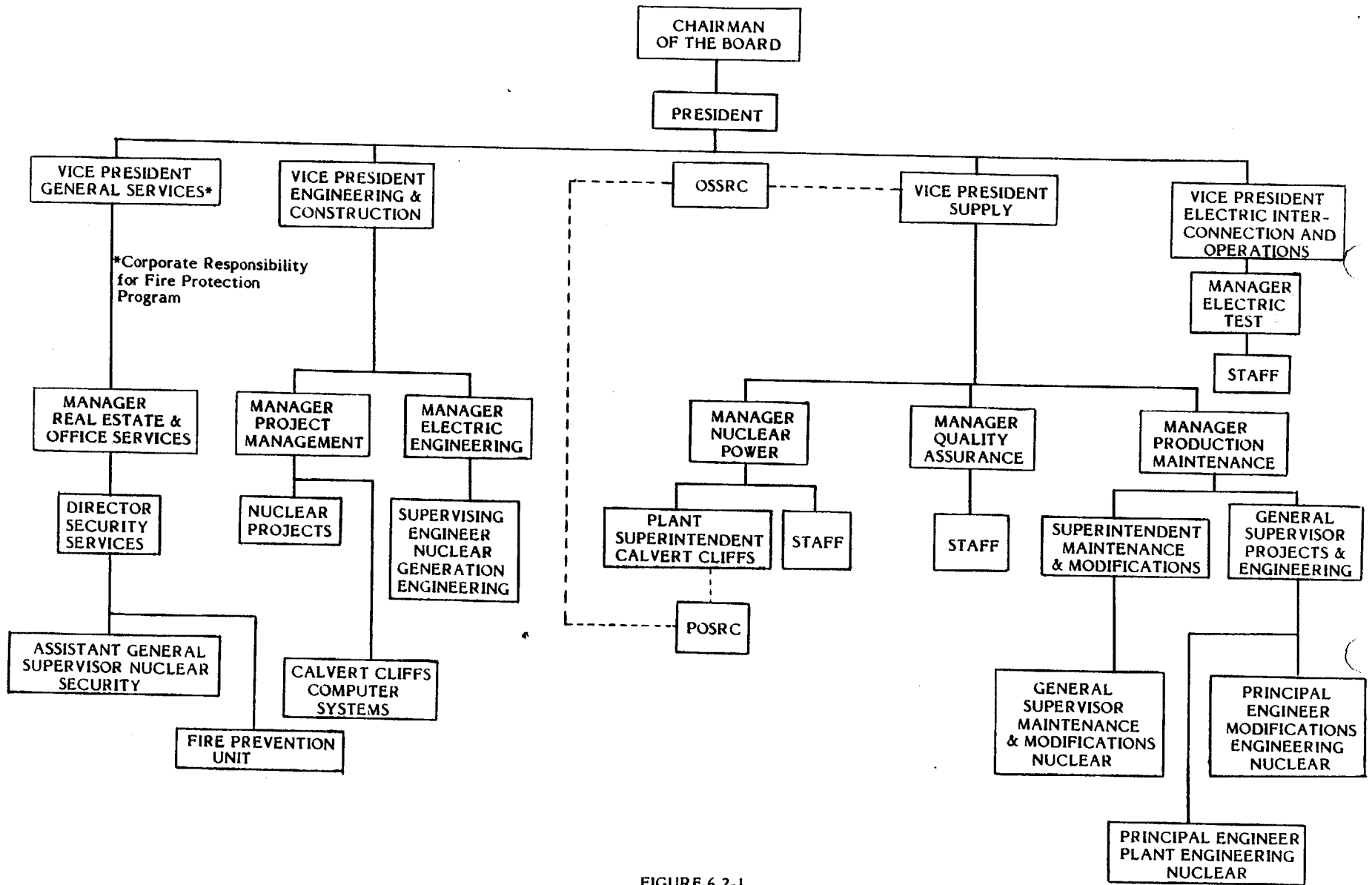


FIGURE 6.2-1
 MANAGEMENT ORGANIZATION CHART
 CALVERT CLIFFS NUCLEAR POWER PLANT
 BALTIMORE GAS & ELECTRIC COMPANY

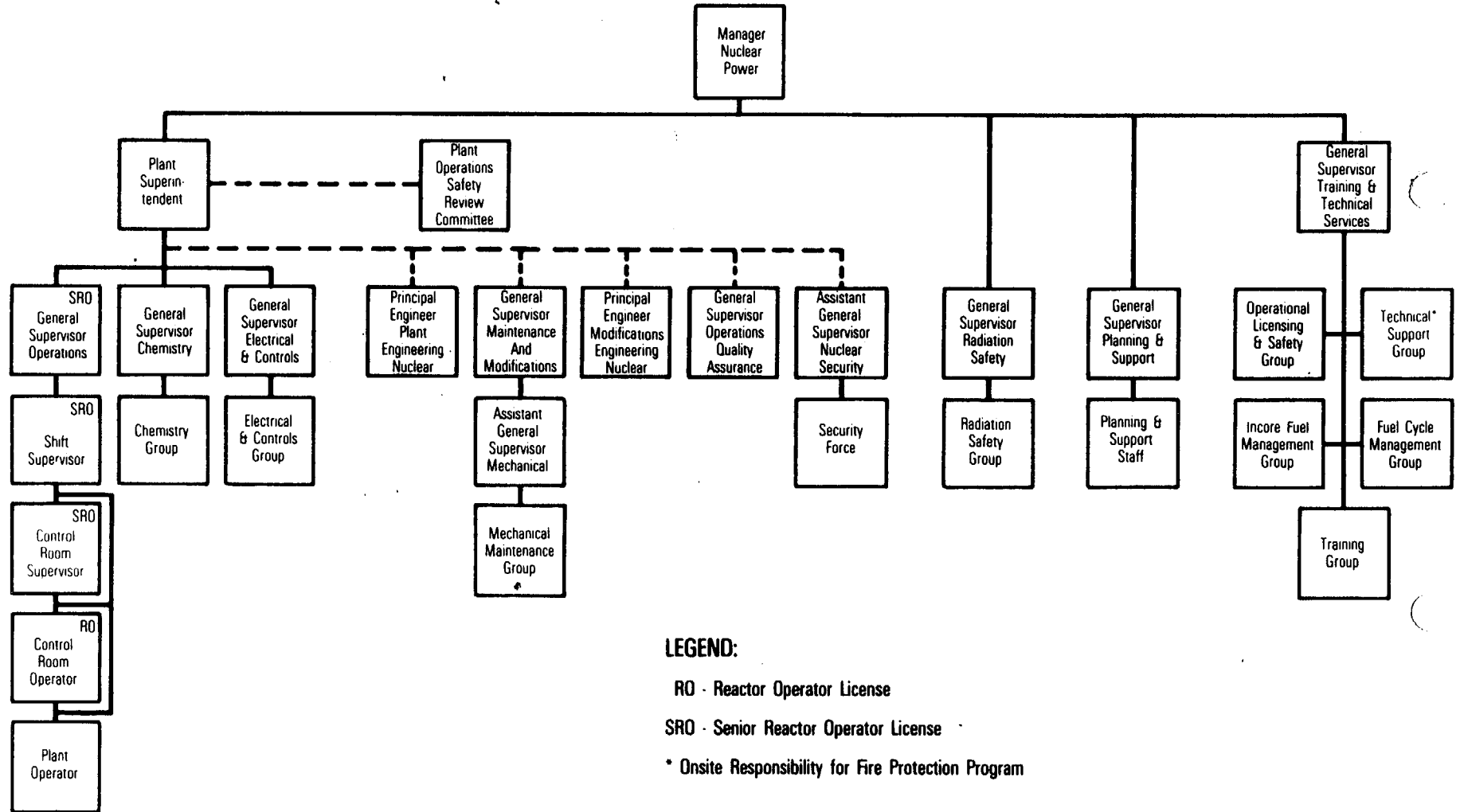


Figure 6.2.2
Organization Chart (Two Unit Operation) - Calvert Cliffs Nuclear Power Plant
Baltimore Gas and Electric Company

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION #

Condition of Unit 2 - Unit 1 in MODES 1, 2, 3 or 4

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	2*
OL**	3	3
Non-Licensed	3	3
Shift Technical Advisor	1##	1##

Condition of Unit 2 - Unit 1 in MODES 5 or 6

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	1*
OL**	3	2
Non-Licensed	3	3
Shift Technical Advisor	1##	0

TABLE 6.2-1
MINIMUM SHIFT CREW COMPOSITION #

Condition of Unit 2 - Unit 1 in MODES 1, 2, 3 or 4

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	2*
OL**	3	3
Non-Licensed	3	3
Shift Technical Advisor	1##	1##

Condition of Unit 2 - Unit 1 in MODES 5 or 6

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	1*
OL**	3	2
Non-Licensed	3	3
Shift Technical Advisor	1##	0

ADMINISTRATIVE CONTROLS

- b. A high radiation area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.12.1.a, above, and in addition locked barricades shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained by the Supervisor-Radiation Control and the Operations Shift Supervisor on duty under their separate administrative control.

6.13 ENVIRONMENTAL QUALIFICATION

6.13.1 By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of: Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class IE Electrical Equipment in Operating Reactors" (DOR Guidelines); or NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment", December 1979. Copies of these documents are attached to Order for Modification of Licenses DPR-53 and DPR-69 dated October 24, 1980.

6.13.2 By no later than December 1, 1980, complete and auditable records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

6.14 SYSTEM INTEGRITY

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and
2. Leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

ADMINISTRATIVE CONTROLS

6.15 IODINE MONITORING

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

1. Training of personnel,
2. Procedures for monitoring, and
3. Provisions for maintenance of sampling and analysis equipment.

ADMINISTRATIVE CONTROLS

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The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 96 AND 77

TO FACILITY OPERATING LICENSE NOS. DPR-53 AND DPR-69

BALTIMORE GAS AND ELECTRIC COMPANY

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NOS. 1 AND 2

DOCKET NOS. 50-317 AND 50-318

Introduction

By application for license amendments dated April 9, 1984, Baltimore Gas and Electric (BG&E) requested changes to the Technical Specifications (TS) for Calvert Cliffs Units 1 and 2. The proposed amendments would change the Unit 1 and Unit 2 TS to reflect: (1) proposed changes to the administrative requirements of TS 6.2, "Organization", associated with corporate organizational changes, (2) installation of additional fire protection instrumentation, and (3) a proposed change in the administrative control of access to high radiation areas. These changes to the TS are in partial response to the application dated April 9, 1984. The remaining issues addressed in the April 9, 1984 application will be addressed in future correspondence.

Discussion and Evaluation

BG&E has proposed two changes to the organizational requirements of TS 6.2, "Organization". The first modification would change the title of the "Senior Control Room Operator" (SCRO) to "Control Room Supervisor" in TS Figure 6.2.2. This change is necessary to distinguish the SCRO who is on duty in the control room, who will be referred to as the Control Room Supervisor, from other SCROs who may be assigned to other duties at the facility.

The second proposed modification would change the corporate responsibility for the BG&E Fire Prevention Unit as shown in TS Figure 6.2-1. The Fire Prevention Unit, which originally reported to the General Supervisor-Finance, now reports to the Manager-Real Estate and Office Services. Because of this organizational responsibility shift, the corporate responsibility for the Fire Protection Program is now assigned to the Vice President-General Services.

In both the proposed changes to TS 6.2, the safety functions of the individual or organizational units involved have not been changed and therefore the proposed changes are only administrative in nature. Accordingly, the proposed changes to TS 6.2 are acceptable.

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The second change to the TS results from the installation of additional fire protection instrumentation. This instrumentation, described in proposed changes to TS Table 3.3-11, "Fire Detection Instruments", includes local heat, flame and/or smoke detectors, and an alarm system. The annunciators in the control room provide an audio-visual alarm which indicates the location of the affected area. Since the fire detection instruments listed in TS Table 3.3-11 are subject to Limiting Conditions for Operation, TS 3.3.3.7, "Fire Detection Instrumentation", the inclusion of the new instruments in TS Table 3.3-11 represents an additional limitation. Since the proposed change to TS Table 3.3-11 represents appropriate additional limitation via TS 3.3.3.7, the proposed changes to the TS are acceptable.

Finally, the licensee has requested a change to TS 6.12, "High Radiation Areas", concerning the control of keys to high radiation areas where the dose rate is greater than 1000 mrem/hr. At the present time TS 6.12.1.b requires that these keys be maintained under the administrative control of the shift supervisor on duty. The licensee has requested that these keys be maintained, "...by the Supervisor-Radiation Control and the Operations Shift Supervisor on duty under their separate administrative control.

Typically, the few plant status changes that affect dose rates (e.g., mode changes, fuel movement, ion exchanger line-ups, etc.) are internally communicated to the Radiation Control Unit. Similarly, the Radiation Control Unit informs Operations of non-routine entries into locked high radiation areas. Since the internal communication is assured, the Radiation Control Supervisor can also effectively administer the keys to the locked high radiation areas and ensure control over unnecessary exposure. Accordingly, this proposed change to the TS is acceptable.

Environmental Consideration

Changes (1) and (3) of the amendments involve changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, with respect to these items, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Sec. 51.22(c)(10). Change (2) of the amendments involves a change in the installation or use of a facility component located within the restricted area and a change in surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, with respect to change (2), these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Sec. 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 2, 1984

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