DCS MS-016

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Docket Nos. 50-317
and 50-318

Mr. A. E. Lundvall, Jr.
Vice President - Supply
Baltimore Gas & Electric Company
P. O. Box 1475
Baltimore, Maryland 21203

Dear Mr. Lundvall:

The Commission has issued the enclosed Amendment Nos. 85 and 68 to Facility Operating License Nos. DPR-53 and DPR-69 for Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated April 27, 1983.

These amendments to the Technical Specifications (1) reflect changes in the on-site organization, (2) provide clarification for surveillance of certain sealed sources, and (3) delete the operability and surveillance requirements for a safety related snubber which has been removed from service (Unit 1 only).

A copy of the Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular monthly Federal Register Notice.

Sincerely,
Original signed by

David H. Jaffe, Project Manager
Operating Reactors Branch #3
Division of Licensing

Enclosures:
1. Amendment No. 85 to DPR-53
2. Amendment No. 68 to DPR-69
3. Safety Evaluation

cc: See next page

Clerk to type requested immediately before signing

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OFFICE	ORB#3:DL	ORB#3:DL	ORAB	ORB#3:DL	AD:OR:DL	OELD	
SURNAME	PMKreutzer	DJaffe/vm	GHolahan	RAClark	GCLainas		
DATE	7/8/83	7/8/83	7/8/83	7/19/83	7/19/83	7/12/83	

NRC FORM 318 (10-80) NRCM 0240 OFFICIAL RECORD COPY USGPO: 1981—335-960

cc:
James A. Biddison, Jr.
General Counsel
Baltimore Gas and Electric Company
P. O. Box 1475
Baltimore, MD 21203

George F. Trowbridge, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

Mr. R. C. L. Olson, Principal Engineer
Nuclear Licensing Analysis Unit
Baltimore Gas and Electric Company
Room 922 - G&E Building
P. O. Box 1475
Baltimore, MD 21203

Mr. Leon B. Russell
Plant Superintendent
Calvert Cliffs Nuclear Power Plant
Maryland Routes 2 & 4
Lusby, MD 20657

Bechtel Power Corporation
Attn: Mr. J. C. Ventura
Calvert Cliffs Project Engineer
15740 Shady Grove Road
Gaithersburg, MD 20760

Combustion Engineering, Inc.
Attn: Mr. R. R. Mills, Manager
Engineering Services
P. O. Box 500
Windsor, CT 06095

Ms. Mary Harrison, President
Calvert County Board of County Commissioners
Prince Frederick, MD 20768

U. S. Environmental Protection Agency
Region III Office
Attn: Regional Radiation Representative
Curtis Building (Sixth Floor)
Sixth and Walnut Streets
Philadelphia, PA 19106

Mr. Ralph E. Architzel
Resident Reactor Inspector
NRC Inspection and Enforcement
P. O. Bos 437
Lusby, MD 20657

Mr. Charles B. Brinkman
Manager - Washington Nuclear Operations
Combustion Engineering, Inc.
7910 Woodmont Avenue
Bethesda, MD 20814

Mr. J. A. Tiernan, Manager
Nuclear Power Department
Calvert Cliffs Nuclear Power Plant
Maryland Routes 2 & 4
Lusby, MD 20657

Mr. W. J. Lippold, Supervisor
Nuclear Fuel Management
Baltimore Gas and Electric Company
Calvert Cliffs Nuclear Power Plant
P. O. Box 1475
Baltimore, Maryland 21203

Mr. R. E. Denton, General Supervisor
Training & Technical Services
Calvert Cliffs Nuclear Power Plant
Maryland Routes 2 & 4
Lusby, MD 20657

Mr. R. M. Douglass, Manager
Quality Assurance Department
Baltimore Gas & Electric Company
Fort Smallwood Road Complex
P. O. Box 1475
Baltimore, MD 21203

Mr. S. M. Davis, General Supervisor
Operations Quality Assurance
Calvert Cliffs Nuclear Power Plant
Maryland Routes 2 & 4
Lusby, MD 20657

Administrator, Power Plant Siting Program
Energy and Coastal Zone Administration
Department of Natural Resources
Tawes State Office Building
Annapolis, MD 21204

Regional Administrator
Nuclear Regulatory Commission, Region I
Office of Executive Director for Operations
631 Park Avenue
King of Prussia, Pennsylvania 19406

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 85
License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated April 27, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with TO CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-53 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 85, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: July 19, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 85

FACILITY OPERATING LICENSE NO. DPR-53

DOCKET NO. 50-317

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages
3/4 7-31
3/4 7-63
3/4 7-64
6-3
6-6

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
1-36-1	SUCTION #11 AUX. FEED PUMP 12'	A	No	No
1-36-1A	SUCTION #11 AUX. FEED PUMP 12'	A	No	No
1-38-5	PRESSURIZER SAMPLE LINES 24'	I	Yes	No
1-38-6	PRESSURIZER SAMPLE LINES 37'	I	Yes	No
1-24-1	DIESEL GENERATOR #12 EXHAUST 92'	A	No	No
1-24-2	DIESEL GENERATOR #11 EXHAUST 92'	A	No	No

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
1-24-3	EMERGENCY DIESEL #12 EXHAUST 61'	A	No	No
1-24-3A	EMERGENCY DIESEL #12 EXHAUST 61'	A	No	No
1-24-4	EMERGENCY DIESEL #11 EXHAUST 61'	A	No	No
1-24-4A	EMERGENCY DIESEL #11 EXHAUST 61'	A	No	No
1-24-5	DIESEL GENERATOR #21 EXHAUST 92'	A	No	No
1-24-6	DIESEL GENERATOR #21 EXHAUST 62'	A	No	No
1-24-6A	DIESEL GENERATOR #21 EXHAUST 62'	A	No	No
1-28-1	UNIT 1 AFW PUMP ROOM 22'	A	No	No
1-28-2	UNIT 1 AFW PUMP ROOM 22'	A	No	No
1-28-3	UNIT 1 AFW PUMP ROOM 22'	A	No	No
1-41-1	SUCTION #13 CHARGING PUMP -10'	A	No	No
1-41-2	AUX. SPRAY 65'	I	Yes	No
1-41-3	AUX. SPRAY 65'	I	Yes	No

3/4.7.9 SEALED SOURCE CONTAMINATION

LIMITING CONDITION FOR OPERATION

3.7.9.1 Each sealed source containing radioactive material either in excess of 100 microcuries of beta and/or gamma emitting material or 5 microcuries of alpha emitting material shall be free of $\geq$ 0.005 microcuries of removable contamination.

APPLICABILITY: At all times.

ACTION:

a. Each sealed source with removable contamination in excess of the above limit shall be immediately withdrawn from use and:

   1. Either decontaminated and repaired, or

   2. Disposed of in accordance with Commission Regulations.

b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.9.1.1 Test Requirements - Each sealed source shall be tested for leakage and/or contamination by:

a. The licensee, or

b. Other persons specifically authorized by the Commission or an Agreement State.

The test method shall have a detection sensitivity of at least 0.005 microcuries per test sample.

4.7.9.1.2 Test Frequencies - Each category of sealed sources (excluding startup sources and fission detectors previously subjected to core flux) shall be tested at the frequencies described below.

a. Sources in use - At least once per six months for all sealed sources containing radioactive material:

SURVEILLANCE REQUIREMENTS (Continued)

1. With a half-life greater than 30 days (excluding Hydrogen 3), and

2. In any form other than gas.

b. Stored sources not in use - Each sealed source and fission detector shall be tested prior to use or transfer to another licensee unless tested within the previous six months. Sealed sources transferred without a certificate indicating the last test date shall be tested prior to being placed into use.

c. Startup sources and fission detectors - Each sealed startup source and fission detector shall be tested within 31 days prior to being subjected to core flux or installed in the core and following repair or maintenance to the source or detector.

4.7.9.1.3 Reports - A report shall be prepared and submitted to the Commission on an annual basis if sealed source or fission detector leakage tests reveal the presence of $\geq$ 0.005 microcuries of removable contamination.

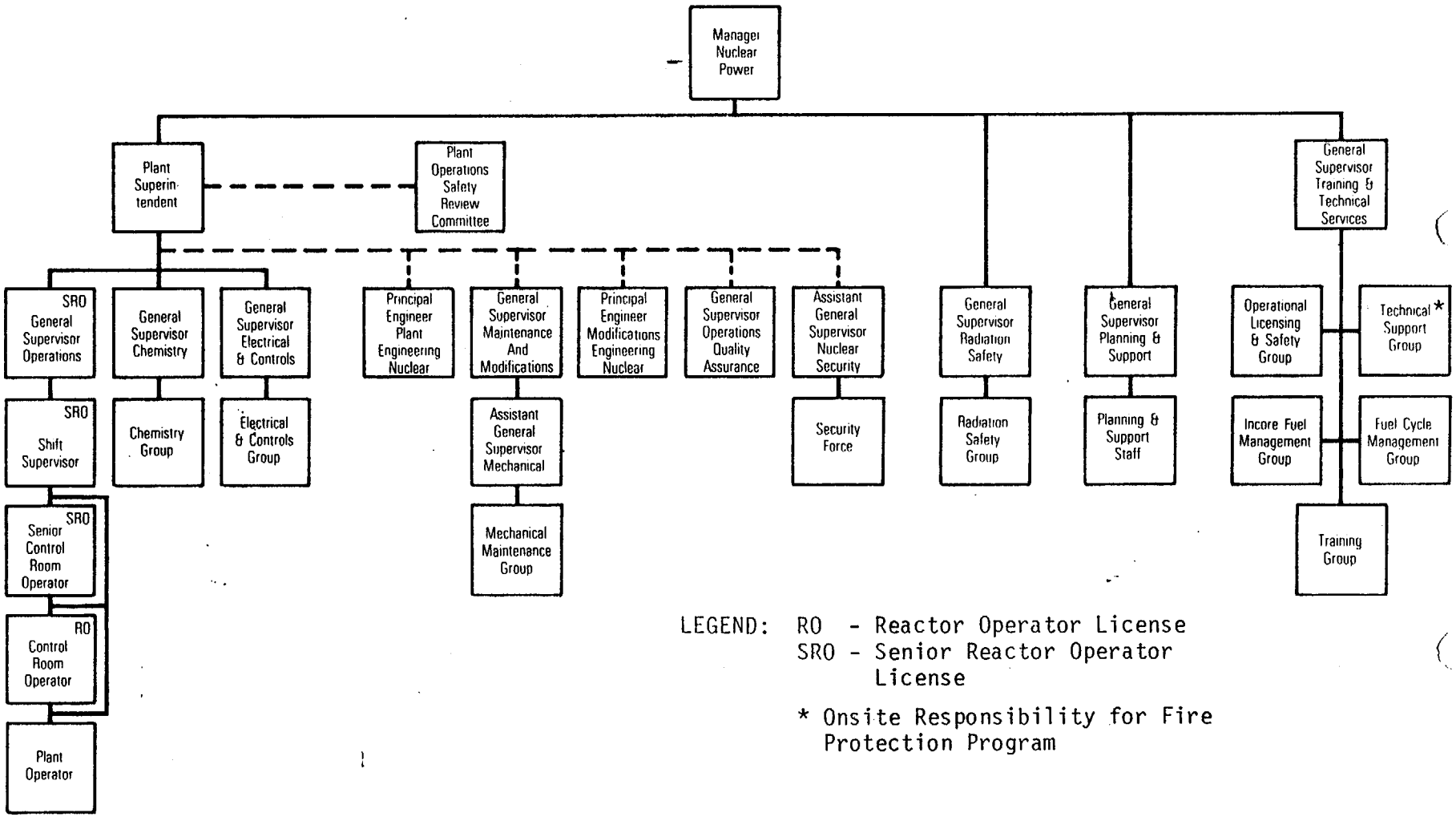

Figure 6.2.2 Organization Chart (Two Unit Operation) - Calvert Cliffs Nuclear Power Plant Baltimore Gas and Electric Company

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION #

Condition of Unit 1 - Unit 2 in MODES 1, 2, 3 or 4

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	2*
OL**	3	3
Non-Licensed	3	3
Shift Technical Advisor	1##	1##

Condition of Unit 1 - Unit 2 in MODES 5 or 6

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	1*
OL**	3	2
Non-Licensed	3	3
Shift Technical Advisor	1##	0

TABLE 6.2-1 (Continued)

*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling, supervising CORE ALTERATIONS during fuel reloading.

**Assumes each individual is licensed on each unit.

#Shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2.1.

##With one unit in MODE 5 or 6, and the other unit in MODE 1, 2, 3 or 4, the SOL holder other than the Shift Supervisor may serve as STA. With one unit defueled and the other unit in MODE 1, 2, 3 or 4, the STA must be an SOL holder in addition to the one SOL required. With both units in MODE 1, 2, 3 or 4, the STA must be an SOL holder in addition to the two SOL's required.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Radiation Safety Engineer who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and (2) the Shift Technical Advisor who shall have a Bachelor's Degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the General Supervisor - Training and Technical Services for the Nuclear Power Department and the General Foreman for the Production Maintenance Department and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55, as applicable.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the General Supervisor - Training and Technical Services and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975.

6.5 REVIEW AND AUDIT

6.5.1 PLANT OPERATIONS AND SAFETY REVIEW COMMITTEE (POSRC)

FUNCTION

6.5.1.1 The POSRC shall function to advise the Plant Superintendent on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The POSRC shall be composed of the:

Chairman:	Plant Superintendent
Member:	General Supervisor - Operations
Member:	General Supervisor - Electrical and Controls
Member:	General Supervisor - Chemistry
Member:	Principal Engineer - Plant Engineering Nuclear
Member:	General Supervisor - Maintenance and Modifications
Member:	Principal Engineer - Incore Fuel Management
Member:	General Supervisor - Radiation Safety
Member:	General Supervisor - Training and Technical Services

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the POSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in POSRC activities at any one time.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 68
License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated April 27, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-69 is hereby amended to read as follows:

   2 Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 68 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: July 19, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 68

FACILITY OPERATING LICENSE NO. DPR-69

DOCKET NO. 50-318

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 7-55
3/4 7-56
6-3
6-6

PLANT SYSTEMS

3/4.7.9 SEALED SOURCE CONTAMINATION

LIMITING CONDITION FOR OPERATION

3.7.9.1 Each sealed source containing radioactive material either in excess of 100 microcuries of beta and/or gamma emitting material or 5 microcuries of alpha emitting material shall be free of $\geq$ 0.005 microcuries of removable contamination.

APPLICABILITY: At all times.

ACTION:

a. Each sealed source with removable contamination in excess of the above limit shall be immediately withdrawn from use and:

   1. Either decontaminated and repaired, or

   2. Disposed of in accordance with Commission Regulations.

b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.9.1.1 Test Requirements - Each sealed source shall be tested for leakage and/or contamination by:

a. The licensee, or

b. Other persons specifically authorized by the Commission or an Agreement State.

The test method shall have a detection sensitivity of at least 0.005 microcuries per test sample.

4.7.9.1.2 Test Frequencies - Each category of sealed sources (excluding startup sources and fission detectors previously subjected to core flux) shall be tested at the frequencies described below.

a. Sources in use - At least once per six months for all sealed sources containing radioactive material:

SURVEILLANCE REQUIREMENTS (Continued)

1. With a half-life greater than 30 days (excluding Hydrogen 3), and

2. In any form other than gas.

b. Stored sources not in use - Each sealed source and fission detector shall be tested prior to use or transfer to another licensee unless tested within the previous six months. Sealed sources transferred without a certificate indicating the last test date shall be tested prior to being placed into use.

c. Startup sources and fission detectors - Each sealed startup source and fission detector shall be tested within 31 days prior to being subjected to core flux or installed in the core and following repair or maintenance to the source or detector.

4.7.9.1.3 Reports - A report shall be prepared and submitted to the Commission on an annual basis if sealed source or fission detector leakage tests reveal the presence of $\geq$ 0.005 microcuries of removable contamination.

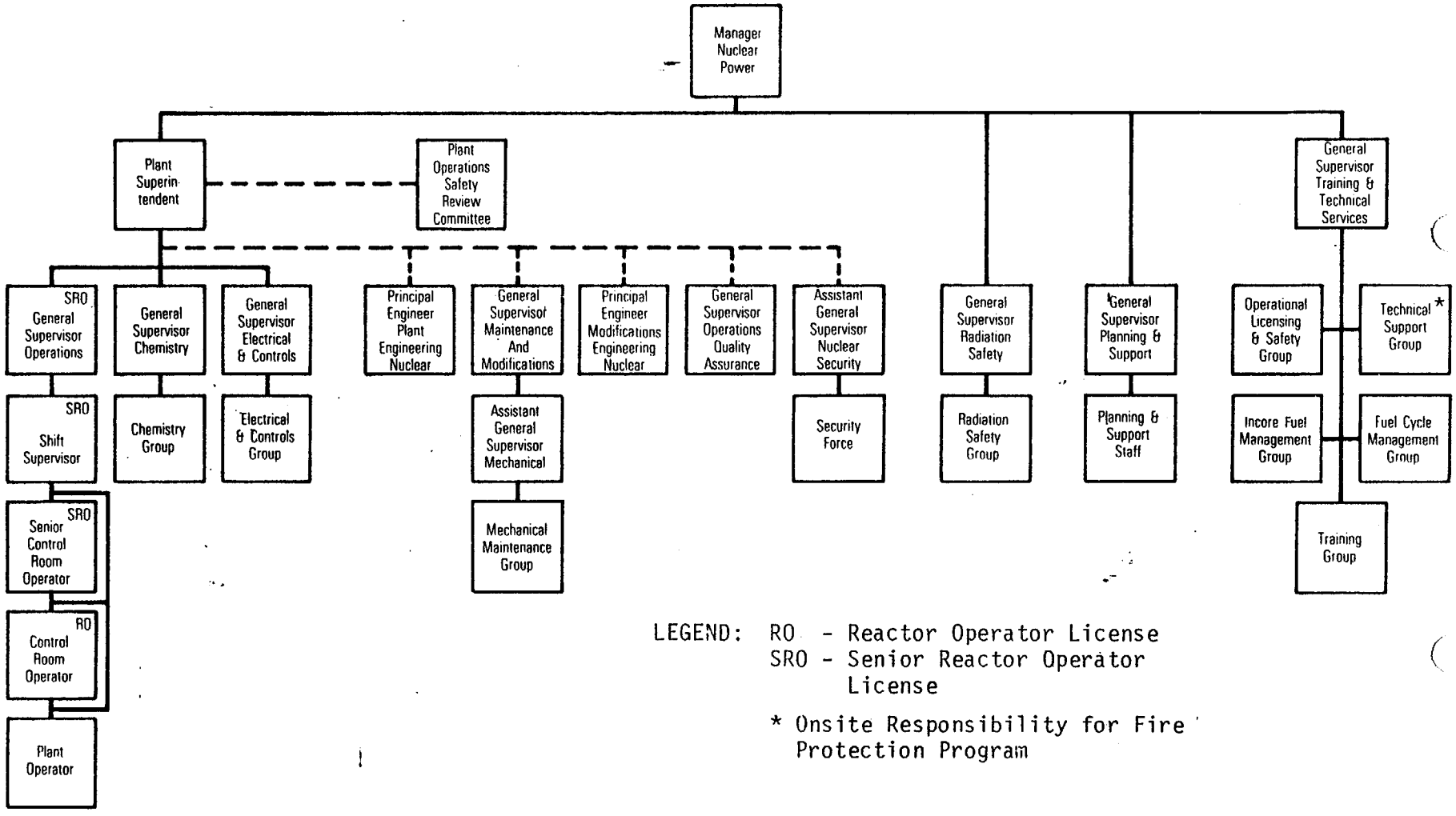

Figure 6.2.2 Organization Chart (Two Unit Operation) - Calvert Cliffs Nuclear Power Plant Baltimore Gas and Electric Company

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION #

Condition of Unit 2 - Unit 1 in MODES 1, 2, 3 or 4

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	2*
OL**	3	3
Non-Licensed	3	3
Shift Technical Advisor	1##	1##

Condition of Unit 2 - Unit 1 in MODES 5 or 6

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	1*
OL**	3	2
Non-Licensed	3	3
Shift Technical Advisor	1##	0

TABLE 6.2-1 (Continued)

*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling, supervising CORE ALTERATIONS during fuel reloading.

**Assumes each individual is licensed on each unit.

#Shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2.1.

##With one unit in MODE 5 or 6, and the other unit in MODE 1, 2, 3, or 4, the SOL holder other than the Shift Supervisor may serve as STA. With one unit defueled and the other unit in MODE 1, 2, 3 or 4, the STA must be an SOL holder in addition to the one SOL required. With both units in MODE 1, 2, 3 or 4, the STA must be an SOL holder in addition to the two SOLs required.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Radiation Safety Engineer who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and (2) the Shift Technical Advisor who shall have a Bachelor's Degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the General Supervisor - Training and Technical Services for the Nuclear Power Department and the General Foreman for the Production Maintenance Department and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55, as applicable.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the General Supervisor - Training and Technical Services and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975.

6.5 REVIEW AND AUDIT

6.5.1 PLANT OPERATIONS AND SAFETY REVIEW COMMITTEE (POSRC)

FUNCTION

6.5.1.1 The POSRC shall function to advise the Plant Superintendent on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The POSRC shall be composed of the:

Chairman:	Plant Superintendent
Member:	General Supervisor - Operations
Member:	General Supervisor - Electrical and Controls
Member:	General Supervisor - Chemistry
Member:	Principal Engineer - Plant Engineering Nuclear
Member:	General Supervisor - Maintenance and Modifications
Member:	Principal Engineer - Incore Fuel Management
Member:	General Supervisor - Radiation Safety
Member:	General Supervisor - Training and Technical Services

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the POSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in POSRC activities at any one time.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 85 AND 68 TO

FACILITY OPERATING LICENSES NOS. DPR-53 AND DPR-69

BALTIMORE GAS AND ELECTRIC COMPANY

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NOS. 1 & 2

DOCKET NOS. 50-317 AND 50-318

Introduction

By application dated April 27, 1983, Baltimore Gas and Electric Company (BG&E) requested changes to the Technical Specifications (TS) for Calvert Cliffs Units 1 and 2. The proposed changes to the TS would (1) reflect changes in the on-site organization, (2) provide clarification for surveillance of certain sealed sources, and (3) delete the operability and surveillance requirements for a safety-related snubber which has been removed from service (Unit 1 only).

Discussion and Evaluation

BG&E has proposed changes to Section 6.0 of the TS in two areas, namely Figure 6.2.2 Organization Chart - Calvert Cliffs Nuclear Power Plant, and the composition of the Plant Operations and Safety Review Committee (POSRC) membership as noted in Section 6.5.1.2. These TS changes are required to reflect changes in the on-site organization at Calvert Cliffs.

The position of Supervisor-Nuclear Fuel Management is to be eliminated at Calvert Cliffs. The Nuclear Fuel Management group will be divided into two parts, Incore Fuel Management and Fuel Cycle Management, both of which report to the General Supervisor-Training and Technical Services. A new position of General Supervisor-Planning and Support will be formed with a Planning and Support staff reporting to him, which will encompass the present responsibilities of the Supervisor-Administrative Services. The job title of General Foreman and Assistant General Foreman-Mechanical will be changed to General Supervisor-Maintenance and Modifications and Assistant General Supervisor-Mechanical, respectively. The job title of Supervisor-Calvert Cliffs Security will be changed to Assistant General Supervisor-Nuclear Security. Proposed TS Figure 6.2.2 reflects these organizational and title changes.

Since the position of Supervisor-Nuclear Fuel Management will be eliminated, the Principal Engineer of Incore Fuel Management will replace the Supervisor-Nuclear Fuel Management as a POSRC member. This proposed change is reflected in TS 6.5.1.2.

We have reviewed the proposed changes to the Calvert Cliffs on-site organization as reflected in TS Figure 6.2.2. We conclude that the changes will not degrade the technical or administrative functioning of this organization and are therefore acceptable. In addition, we find that the proposed change to TS 6.5.1.2 is acceptable in that the proposed change to the POSRC will not degrade the ability of the POSRC to advise the Plant Superintendent on all matters related to nuclear safety at Calvert Cliffs.

BG&E has also proposed changes to TS 3/4.7.9, "Sealed Source Contamination." The proposed changes would clarify the surveillance requirements for startup sources and fission detectors. At the present time, TS 4.7.9.1.1 requires startup sources and fission detectors to be tested for leakage and/or contamination "...within 30 days prior to being subjected to core flux...". This TS could be interpreted to require surveillance prior to each reactor startup, thus resulting in the need to remove these sources from the reactor. The proposed TS changes would transfer the words "...(excluding startup sources and fission detectors previously subject to core flux)..." from TS 4.7.9.1.2a to TS 4.7.9.1.2. This change extends the stated exclusion from "sources in use" to all categories of sealed sources including startup sources and fission detectors. In addition, the words "...or installation in the core..." would be inserted in TS 4.7.9.1.2c to provide the clarification concerning the timing for performance of leakage and/or contamination surveillance.

The Bases for TS 3.4.7.9 indicate the need for surveillance of sealed sources to "...ensure that leakage from byproduct, source, and special nuclear material sources will not exceed allowable intake values." Once these sources undergo surveillance for leakage and/or contamination and are installed in the reactor vessel, they are isolated by virtue of their location. It was never the intent of the NRC to require that these sources be removed for testing prior to each reactor startup. This position is reflected in versions of the Combustion Engineering Standard Technical Specifications (CE-STS) issued by the NRC subsequent to the version of the CE-STS for Calvert Cliffs. Accordingly, we conclude that the proposed changes to TS 3/4.7.9 are consistent with the current NRC approach concerning surveillance for sealed sources and are therefore acceptable.

Finally, BG&E has proposed a change to the snubber operability and surveillance requirements of Calvert Cliffs Unit 1 TS 3/4.7.8, "Snubbers". This change would delete snubber 1-38-4 from the list of snubbers in TS Table 3.7-4. This snubber was removed under an earlier modification which installed the Reactor Coolant System head vent and Pressurizer vent piping. The system upstream of and including the main solenoid valve is Seismic Class 1 allowing removal of the snubber, since the system is adequately supported and snubber 1-38-4 is no longer required. Since the snubber is no longer needed or installed, no reduction in the bases for any Technical Specifications would result from this change.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: July 19, 1983

Principal Contributor:
D. H. Jaffe