

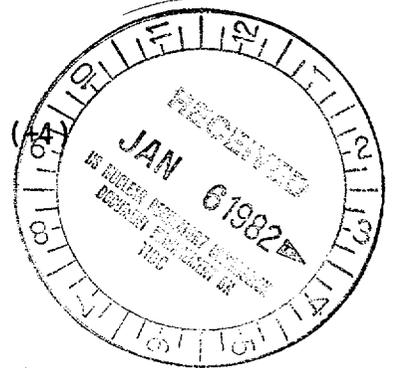
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Docket No. 50-317
50-318

Mr. A. E. Lundvall, Jr.
Vice President - Supply
Baltimore Gas & Electric Company
P.O. Box 1475
Baltimore, Maryland 21203

Dear Mr. Lundvall:

The Commission has issued the enclosed Amendment Nos. 64 and 48 to Facility Operating License Nos. DPR-53 and DPR-69 for Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated July 30, 1981.

These amendments revise the Technical Specifications to provide updated Limiting Conditions for Operation and Surveillance Requirements for dynamic component restraints (snubbers), delete the restriction on performance of the 60 month reserve battery capacity test, and assign the reserve battery to a capacity test load group.

With regard to mechanical snubbers, we understand that these units are presently not employed for support of safety-related equipment at Calvert Cliffs. Should you desire to use mechanical snubbers on safety-related equipment, please inform us 90-days prior to their planned installation.

A copy of the Safety Evaluation and the Notice of Issuance are also enclosed.

CP
1

Sincerely,

Original signed by:

David H. Jaffe, Project Manager
Operating Reactors Branch #3
Division of Licensing

8201190287 811228
PDR ADDCK 05000317
PDR

Enclosures:

- 1. Amendment No. 64 to DPR-53
- 2. Amendment No. 48 to DPR-69
- 3. Safety Evaluation
- 4. Notice of Issuance

cc: See next page

Handwritten notes and signatures:
CMT 12/23/81
JST
48 Notice of Issuance
AD Novak

OFFICE	ORB#3:DL	ORB#8:DL	ORAB	ORB#3:DL	ORB#3:DL	AD OR:DL	OELD
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DATE	11/30/81	12/1/81	12/15/81	12/15/81	12/15/81	12/15/81	12/17/81

Baltimore Gas and Electric Company

cc:

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cc w/enclosure(s) and incoming
dated: 7/30/81

Administrator, Power Plant Siting Program
Energy and Coastal Zone Administration
Department of Natural Resources
Tawes State Office Building
Annapolis, MD 21204

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555



BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 64
License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated July 30, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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PDR ADDCK 05000317
P PDR

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-53 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 64, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

For *Charles M. Trammell*
Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: December 28, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 64

FACILITY OPERATING LICENSE NO. DPR-53

DOCKET NO. 50-317

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove

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3/4 7-25
3/4 7-26

3/4 8-10
B 3/4 7-5
B 3/4 7-6
6-20

Insert

VII
3/4 7-25
3/4 7-26
3/4 7-26a
3/4 7-26b
3/4 8-10
B 3/4 7-5
B 3/4 7-6
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3/4.7.8 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.8.1 All snubbers listed in Table 3.7-4 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4. (MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES.)

ACTION: With one or more snubbers inoperable, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status, and perform an engineering evaluation* per Specification 4.7.8.b. and c. on the supported component or declare the supported system inoperable and follow the appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.8.1 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

a. Visual Inspections

Visual inspections shall be performed in accordance with the following schedule:

<u>No. Inoperable Snubbers per Inspection Period</u>	<u>Subsequent Visual** Inspection Period#</u>
0	18 months + 25%
1	12 months + 25%
2	6 months + 25%
3, 4	124 days + 25%
5, 6, 7	62 days + 25%
8 or more	31 days + 25%

The snubbers may be categorized into two groups: Those accessible and those inaccessible during reactor operation. Each group may be inspected independently in accordance with the above schedule.

* A documented, visual inspection shall be sufficient to meet the requirements for an engineering evaluation. Additional analyses, as needed, shall be completed in a reasonable period of time.

** The inspection interval shall not be lengthened more than two steps at a time.

The provisions of Specification 4.0.2 are not applicable.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

b. Visual Inspection Acceptance Criteria

Visual inspections shall verify (1) that there are no visible indications of damage or impaired OPERABILITY, and (2) that the snubber installation exhibits no visual indications of detachment from foundations or supporting structures. Snubbers which appear inoperable as a result of visual inspections may be determined OPERABLE for the purpose of establishing the next visual inspection interval, providing that (1) the cause of the rejection is clearly established and remedied for that particular snubber and for other snubbers that may be generically susceptible; and/or (2) the affected snubber is functionally tested in the as found condition and determined OPERABLE per Specification 4.7.8.d, as applicable. When the fluid port of a hydraulic snubber* is found to be uncovered, the snubber shall be determined inoperable unless it can be determined OPERABLE via functional testing for the purpose of establishing the next visual inspection interval.

For the snubber(s) found inoperable, an engineering evaluation shall be performed on the component(s) which are supported by the snubber(s). The scope of this engineering evaluation shall be consistent with the licensee's engineering judgment and may be limited to a visual inspection of the supported component(s). The purpose of this engineering evaluation shall be to determine if the component(s) supported by the snubber(s) were adversely affected by the inoperability of the snubber(s) in order to ensure that the supported component remains capable of meeting the designed service.

c. Functional Tests

At least once per 18 months during shutdown, a representative sample of 10% of the snubbers in use in the plant shall be functionally tested either in place or in a bench test.** For each snubber that does not meet the functional test acceptance criteria of Specification 4.7.8.d, an additional 5% of the snubbers shall be functionally tested until no more failures are found or until all snubbers have been functionally tested.

* This requirement is also applicable to snubbers served by a common hydraulic reservoir.

** The Steam Generator snubbers 1-63-13 through 1-63-28 need not be functionally tested until the refueling outage following June 30, 1985.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

Snubbers identified in Table 3.7-4 as "Especially Difficult to Remove" or in "High Exposure Zones" shall also be included in the representative sample.*

In addition to the regular sample, snubbers which failed the previous functional test shall be retested during the next test period. If a spare snubber has been installed in place of a failed snubber, then both the failed snubber (if it is repaired and installed in another position) and the spare snubber shall be retested during the next test period. Failure of these snubbers shall not entail functional testing of additional snubbers.

If any snubber selected for functional testing either fails to lock up or fails to move, i.e., frozen in place, the cause will be evaluated and if caused by manufacturer or design deficiency all generically susceptible snubbers of the same design subject to the same defect shall be functionally tested. This testing requirement shall be independent of the requirements stated above for snubbers not meeting the functional test acceptance criteria.

For the snubber(s) found inoperable, an engineering evaluation shall be performed on the component(s) which are supported by the snubber(s). The scope of this engineering evaluation shall be consistent with the licensee's engineering judgment and may be limited to a visual inspection of the supported component(s). The purpose of this engineering evaluation shall be to determine if the component(s) supported by the snubber(s) were adversely affected by the inoperability of the snubber(s) in order to ensure that the supported component remains capable of meeting the designed service.

d. Hydraulic Snubbers Functional Test Acceptance Criteria

The hydraulic snubber functional test shall verify that:

1. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.
2. Snubber bleed, or release rate, where required, is within the specified range in compression or tension. For snubbers specifically required to not displace under continuous load, the ability of the snubber to withstand load without displacement shall be verified.

* Permanent or other exemptions from functional testing for individual snubbers in these categories may be granted by the Commission only if a justifiable basis for exemption is presented and/or snubber life destructive testing was performed to qualify snubber operability for all design conditions at either the completion of their fabrication or at a subsequent date.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

e. Snubber Service Life Monitoring*

A record of the service life of each snubber, the date at which the designated service life commences and the installation and maintenance records on which the designated service life is based shall be maintained as required by Specification 6.10.2.m.

At least once per 18 months, the installation and maintenance records for each snubber listed in Table 3.7-4 shall be reviewed to verify that the indicated service life has not been exceeded or will not be exceeded prior to the next scheduled snubber service life review.** If the indicated service life will be exceeded prior to the next scheduled snubber service life review, the snubber service life shall be re-evaluated or the snubber shall be replaced or reconditioned so as to extend its service life beyond the date of the next scheduled service life review. This reevaluation, replacement, or reconditioning shall be indicated in the records.

f. Snubbers Served by a Common Hydraulic Reservoir

Snubbers served by a common hydraulic reservoir are indicated by a bracket in Table 3.7-4. All reservoirs serving more than one snubber shall be inspected to ensure adequate hydraulic level:

1. Within 7 days after reactor startup following a major outage or following any maintenance in the immediate vicinity of these snubbers, reservoirs, or associated hydraulic piping; and
2. Every 31 days \pm 25%.

* The Snubber Service Life Program shall be fully implemented by January 1, 1983.

**The provisions of Specification 4.0.2 are applicable.

PLANT SYSTEMS

BASES

environment. The operation of this system and the resultant effect on offsite dosage calculations was assumed in the accident analyses.

3/4.7.8 SNUBBERS

All snubbers are required OPERABLE to ensure that the structural integrity of the reactor coolant system and all other safety related systems is maintained during and following a seismic or other event initiating dynamic loads. Snubbers excluded from this inspection program are those installed on non-safety related systems and then only if their failure or failure of the system on which they are installed, would have no adverse effect on any safety-related system.

The visual inspection frequency is based upon maintaining a constant level of snubber protection to systems. Therefore, the required inspection interval varies inversely with the observed snubber failures and is determined by the number of inoperable snubbers found during an inspection. Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection. However, the results of such early inspections performed before the original required time interval has elapsed (nominal time less 25%) may not be used to lengthen the required inspection interval. Any inspection whose results require a shorter inspection interval will override the previous schedule.

When the cause of the rejection of a snubber is clearly established and remedied for that snubber and for any other snubbers that may be generically susceptible, and verified by inservice functional testing, that snubber may be exempted from being counted as inoperable. Generically susceptible snubbers are those which are (1) of a specific make or model, (2) of the same design, and (3) similarly located or exposed to the same environmental conditions such as temperature, radiation, and vibration. These characteristics of the snubber installation shall be evaluated to determine if further functional testing of similar snubber installations is warranted.

When a snubber is found inoperable, an engineering evaluation is performed, in addition to the determination of the snubber mode of failure, in order to determine if any safety-related component or system has been adversely affected by the inoperability of the snubber. The engineering evaluation shall determine whether or not the snubber mode of failure has imparted a significant effect or degradation on the supported component or system.

To provide assurance of snubber functional reliability, a representative sample of the installed snubbers will be functionally tested during plant shutdowns at 18 month intervals. Observed failures of these sample snubbers shall require functional testing of additional units.

The service life of a snubber is evaluated via manufacturer input and information through consideration of the snubber service conditions and associated installation and maintenance records (newly installed snubber, seal replaced, spring replaced, in high radiation area, in high temperature area, etc. . . .). The requirement to monitor the snubber service life is included to ensure that the

PLANT SYSTEMS

BASES

snubbers periodically undergo a performance evaluation in view of their age and operating conditions. The service life program is designed to uniquely reflect the conditions at Calvert Cliffs. The criteria for evaluating service life shall be determined, and documented, by the licensee. Records will provide statistical bases for future consideration of snubber service life. The requirements for the maintenance of records and the snubber service life review are not intended to affect plant operation.

3/4.7.9 SEALED SOURCE CONTAMINATION

The limitations on removable contamination for sources requiring leak testing, including alpha emitters, is based on 10 CFR 70.39(c) limits for plutonium. This limitation will ensure that leakage from byproduct, source, and special nuclear material sources will not exceed allowable intake values.

3/4.7.10 WATERTIGHT DOORS

This specification is provided to ensure the protection of safety related equipment from the effects of water or steam escaping from ruptured pipes or components in adjoining rooms.

3/4.7.11 FIRE SUPPRESSION SYSTEMS

The OPERABILITY of the fire suppression systems ensures that adequate fire suppression capability is available to confine and extinguish fires occurring in any portion of the facility where safety related equipment is located. The fire suppression system consists of the water system, spray and/or sprinklers, Halon and fire hose stations. The collective capability of the fire suppression systems is adequate to minimize potential damage to safety related equipment and is a major element in the facility fire protection program.

In the event that portions of the fire suppression systems are inoperable, alternate backup fire fighting equipment is required to be made available in the affected areas until the inoperable equipment is restored to service. Where a continuous fire watch is required in lieu of fire protection equipment and habitability due to heat or radiation is a concern, the fire watch should be stationed in a habitable area as close as possible to the inoperable equipment.

In the event the fire suppression water system becomes inoperable, immediate corrective measures must be taken since this system provides the major fire suppression capability of the plant. The requirement for a twenty-four hour report to the Commission provides for prompt evaluation of the acceptability of the corrective measures to provide adequate fire suppression capability for the continued protection of the nuclear plant.

ELECTRICAL POWER SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

- e. With one 125-volt D.C. battery inoperable during surveillance testing of the battery per Specification 4.8.2.3.2.e., operation may continue provided the associated bus is being powered by the Reserve Battery and an OPERABLE charger.
- f. With single cells having a voltage decrease of more than 0.10 volts from the previous performance discharge test (4.8.2.3.2.e.) value, but still ≥ 2.10 volts per surveillance requirement 4.8.2.3.2.b.1., either restore/replace cells or replace the affected battery with the Reserve Battery within 24 hours or be in HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.8.2.3.1 Each D.C. bus train shall be determined OPERABLE and energized at least once per 7 days by verifying correct breaker alignment and indicated power availability:

4.8.2.3.2 Each 125-volt battery bank and charger and the Reserve Battery shall be demonstrated OPERABLE;

a. At least once per 7 days by verifying that:

- 1. The electrolyte level of each pilot cell is between the minimum and maximum level indication marks.
- 2. The pilot cell specific gravity, corrected to 77°F and full electrolyte level is ≥ 1.200 .
- 3. The pilot cell voltage is ≥ 2.10 volts.
- 4. The overall battery voltage is ≥ 125 volts.

b. At least once per 92 days by verifying that:

- 1. The voltage of each connected cell is ≥ 2.10 volts under float charge and has not decreased more than 0.10 volts from the value observed during the latest performance discharge test (4.8.2.3.2.e).
- 2. The specific gravity, corrected to 77°F and full electrolyte level, of each connected cell is ≥ 1.200 and has not decreased more than 0.02 from the value observed during the previous test.
- 3. The electrolyte level of each connected cell is between the minimum and maximum level indication marks.

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- c. At least once per 18 months by verifying that:
1. The cells, cell plates and battery racks show no visual indication of physical damage or deterioration.
 2. The cell-to-cell and terminal connections are clean, tight, and coated with anti-corrosion material.
- d. At least once per 18 months by verifying that the battery capacity, with the charger disconnected, is adequate to either:
1. Supply and maintain in OPERABLE status all of the actual emergency loads for at least 2 hours when the battery is subjected to a battery service test, or
 2. Supply a dummy load of the following profile for at least 2 hours while maintaining the battery terminal voltage ≥ 100 volts:
 - a) Batteries 11, 21 and Reserve:
First minute ≥ 827 amperes
Next 1 minute ≥ 461 amperes
Next 117 minutes ≥ 251 amperes
Next 1 minute ≥ 325 amperes
 - b) Batteries 12 and 22:
First minute ≥ 193 amperes
Next 119 minutes ≥ 160 amperes
- At the completion of this battery test, the battery charger* shall be demonstrated capable of recharging the battery at a rate of ≤ 400 amperes while supplying normal D. C. loads. The battery shall be charged to at least 95% capacity in ≤ 24 hours.
- e. At least once per 60 months by verifying that the battery capacity is at least 80% of the manufacturer's rating when subjected to a performance discharge test. This performance discharge test shall be performed subsequent to the satisfactory completion of the required battery service test.

*Not applicable to the charger associated with the Reserve Battery.

ADMINISTRATIVE CONTROLS

6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. ALL REPORTABLE OCCURRENCES submitted to the Commission.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to Operating Procedures.
- g. Records of radioactive shipments.
- h. Records of sealed source and fission detector leak tests and results.
- i. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.

ADMINISTRATIVE CONTROLS

- d. Records of radiation exposure for all individuals entering radiation control areas.
- e. Records of gaseous and liquid radioactive material released to the environs.
- f. Records of transient or operational cycles for those facility components identified in Table 5.7.1.
- g. Records of training and qualification for current members of the plant staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities identified in the NRC approved QA Manual as lifetime records.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the POSRC and the OSSRC.
- l. Records of Environmental Qualification which are covered under the provisions of paragraph 6.13.
- m. Records of the service lives of all snubbers listed on Table 3.7-4 including the date at which the service life commences and associated installation and maintenance records.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20:

- a. A high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Special or Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 46
License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated July 30, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

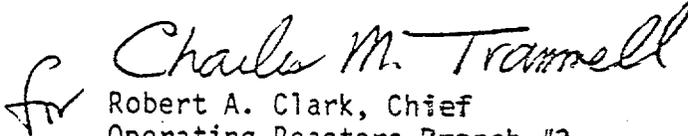
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-69 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 46, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

The image shows a handwritten signature in cursive that reads "Charles M. Trammell". To the left of the signature is a small, stylized handwritten mark that appears to be "RAC".

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: December 28, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 46

FACILITY OPERATING LICENSE NO. DPR-69

DOCKET NO. 50-317

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove

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3/4 7-25
3/4 7-26

3/4 8-10
B 3/4 7-5
B 3/4 7-6
6-20

Insert

VII
3/4 7-25
3/4 7-26
3/4 7-26a
3/4 7-26b
3/4 8-10
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3/4.7.8 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.8.1 All snubbers listed in Table 3.7-4 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4. (MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES.)

ACTION: With one or more snubbers inoperable, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status, and perform an engineering evaluation* per Specification 4.7.8.b. and c. on the supported component or declare the supported system inoperable and follow the appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.8.1 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

a. Visual Inspections

Visual inspections shall be performed in accordance with the following schedule:

<u>No. Inoperable Snubbers per Inspection Period</u>	<u>Subsequent Visual** Inspection Period#</u>
0	18 months + 25%
1	12 months + 25%
2	6 months + 25%
3, 4	124 days + 25%
5, 6, 7	62 days + 25%
8 or more	31 days + 25%

The snubbers may be categorized into two groups: Those accessible and those inaccessible during reactor operation. Each group may be inspected independently in accordance with the above schedule.

* A documented, visual inspection shall be sufficient to meet the requirements for an engineering evaluation. Additional analyses, as needed, shall be completed in a reasonable period of time.

** The inspection interval shall not be lengthened more than two steps at a time.

The provisions of Specification 4.0.2 are not applicable.

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SURVEILLANCE REQUIREMENTS (Continued)

b. Visual Inspection Acceptance Criteria

Visual inspections shall verify (1) that there are no visible indications of damage or impaired OPERABILITY, and (2) that the snubber installation exhibits no visual indications of detachment from foundations or supporting structures. Snubbers which appear inoperable as a result of visual inspections may be determined OPERABLE for the purpose of establishing the next visual inspection interval, providing that (1) the cause of the rejection is clearly established and remedied for that particular snubber and for other snubbers that may be generically susceptible; and/or (2) the affected snubber is functionally tested in the as found condition and determined OPERABLE per Specification 4.7.8.d, as applicable. When the fluid port of a hydraulic snubber* is found to be uncovered, the snubber shall be determined inoperable unless it can be determined OPERABLE via functional testing for the purpose of establishing the next visual inspection interval.

For the snubber(s) found inoperable, an engineering evaluation shall be performed on the component(s) which are supported by the snubber(s). The scope of this engineering evaluation shall be consistent with the licensee's engineering judgment and may be limited to a visual inspection of the supported component(s). The purpose of this engineering evaluation shall be to determine if the component(s) supported by the snubber(s) were adversely affected by the inoperability of the snubber(s) in order to ensure that the supported component remains capable of meeting the designed service.

c. Functional Tests

At least once per 18 months during shutdown, a representative sample of 10% of the snubbers in use in the plant shall be functionally tested either in place or in a bench test.** For each snubber that does not meet the functional test acceptance criteria of Specification 4.7.8.d, an additional 5% of the snubbers shall be functionally tested until no more failures are found or until all snubbers have been functionally tested.

* This requirement is also applicable to snubbers served by a common hydraulic reservoir.

** The Steam Generator snubbers 2-63-11 through 2-63-26 need not be functionally tested until the refueling outage following June 30, 1985.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

Snubbers identified in Table 3.7-4 as "Especially Difficult to Remove" or in "High Exposure Zones" shall also be included in the representative sample.*

In addition to the regular sample, snubbers which failed the previous functional test shall be retested during the next test period. If a spare snubber has been installed in place of a failed snubber, then both the failed snubber (if it is repaired and installed in another position) and the spare snubber shall be retested during the next test period. Failure of these snubbers shall not entail functional testing of additional snubbers.

If any snubber selected for functional testing either fails to lock up or fails to move, i.e., frozen in place, the cause will be evaluated and if caused by manufacturer or design deficiency all generically susceptible snubbers of the same design subject to the same defect shall be functionally tested. This testing requirement shall be independent of the requirements stated above for snubbers not meeting the functional test acceptance criteria.

For the snubber(s) found inoperable, an engineering evaluation shall be performed on the component(s) which are supported by the snubber(s). The scope of this engineering evaluation shall be consistent with the licensee's engineering judgment and may be limited to a visual inspection of the supported component(s). The purpose of this engineering evaluation shall be to determine if the component(s) supported by the snubber(s) were adversely affected by the inoperability of the snubber(s) in order to ensure that the supported component remains capable of meeting the designed service.

d. Hydraulic Snubbers Functional Test Acceptance Criteria

The hydraulic snubber functional test shall verify that:

1. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.
2. Snubber bleed, or release rate, where required, is within the specified range in compression or tension. For snubbers specifically required to not displace under continuous load, the ability of the snubber to withstand load without displacement shall be verified.

* Permanent or other exemptions from functional testing for individual snubbers in these categories may be granted by the Commission only if a justifiable basis for exemption is presented and/or snubber life destructive testing was performed to qualify snubber operability for all design conditions at either the completion of their fabrication or at a subsequent date.

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SURVEILLANCE REQUIREMENTS (Continued)

e. Snubber Service Life Monitoring*

A record of the service life of each snubber, the date at which the designated service life commences and the installation and maintenance records on which the designated service life is based shall be maintained as required by Specification 6.10.2.m.

At least once per 18 months, the installation and maintenance records for each snubber listed in Table 3.7-4 shall be reviewed to verify that the indicated service life has not been exceeded or will not be exceeded prior to the next scheduled snubber service life review.** If the indicated service life will be exceeded prior to the next scheduled snubber service life review, the snubber service life shall be re-evaluated or the snubber shall be replaced or reconditioned so as to extend its service life beyond the date of the next scheduled service life review. This reevaluation, replacement, or reconditioning shall be indicated in the records.

f. Snubbers Served by a Common Hydraulic Reservoir

Snubbers served by a common hydraulic reservoir are indicated by a bracket in Table 3.7-4. All reservoirs serving more than one snubber shall be inspected to ensure adequate hydraulic level:

1. Within 7 days after reactor startup following a major outage or following any maintenance in the immediate vicinity of these snubbers, reservoirs, or associated hydraulic piping; and
2. Every 31 days \pm 25%.

* The Snubber Service Life Program shall be fully implemented by January 1, 1983.

**The provisions of Specification 4.0.2 are applicable.

ELECTRICAL POWER SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

- e. With one 125-volt D.C. battery inoperable during surveillance testing of the battery per Specification 4.8.2.3.2.e., operation may continue provided the associated bus is being powered by the Reserve Battery and an OPERABLE charger.
- f. With single cells having a voltage decrease of more than 0.10 volts from the previous performance discharge test (4.8.2.3.2.e.) value, but still ≥ 2.10 volts per surveillance requirement 4.8.2.3.2.b.1., either restore/replace cells or replace the affected battery with the Reserve Battery within 24 hours or be in HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.8.2.3.1 Each D.C. bus train shall be determined OPERABLE and energized at least once per 7 days by verifying correct breaker alignment and indicated power availability.

4.8.2.3.2 Each 125-volt battery bank and charger and the Reserve Battery shall be demonstrated OPERABLE;

a. At least once per 7 days by verifying that:

- 1. The electrolyte level of each pilot cell is between the minimum and maximum level indication marks.
- 2. The pilot cell specific gravity, corrected to 77°F and full electrolyte level is ≥ 1.200 .
- 3. The pilot cell voltage is ≥ 2.10 volts.
- 4. The overall battery voltage is ≥ 125 volts.

b. At least once per 92 days by verifying that:

- 1. The voltage of each connected cell is ≥ 2.10 volts under float charge and has not decreased more than 0.10 volts from the value observed during the latest performance discharge test (4.8.2.3.2.e).
- 2. The specific gravity, corrected to 77°F and full electrolyte level, of each connected cell is ≥ 1.200 and has not decreased more than 0.02 from the value observed during the previous test.
- 3. The electrolyte level of each connected cell is between the minimum and maximum level indication marks.

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- c. At least once per 18 months by verifying that:
1. The cells, cell plates and battery racks show no visual indication of physical damage or deterioration.
 2. The cell-to-cell and terminal connections are clean, tight, and coated with anti-corrosion material.
- d. At least once per 18 months by verifying that the battery capacity, with the charger disconnected, is adequate to either:
1. Supply and maintain in OPERABLE status all of the actual emergency loads for at least 2 hours when the battery is subjected to a battery service test, or
 2. Supply a dummy load of the following profile for at least 2 hours while maintaining the battery terminal voltage ≥ 100 volts:
 - a) Batteries 11, 21 and Reserve:
First minute ≥ 827 amperes
Next 1 minute ≥ 461 amperes
Next 117 minutes ≥ 251 amperes
Next 1 minute ≥ 325 amperes
 - b) Batteries 12 and 22:
First minute ≥ 193 amperes
Next 119 minutes ≥ 160 amperes
- At the completion of this battery test, the battery charger* shall be demonstrated capable of recharging the battery at a rate of ≤ 400 amperes while supplying normal D. C. loads. The battery shall be charged to at least 95% capacity in ≤ 24 hours.
- e. At least once per 60 months by verifying that the battery capacity is at least 80% of the manufacturer's rating when subjected to a performance discharge test. This performance discharge test shall be performed subsequent to the satisfactory completion of the required battery service test.

*Not applicable to the charger associated with the Reserve Battery.

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environment. The operation of this system and the resultant effect on offsite dosage calculations was assumed in the accident analyses.

3/4.7.8 SNUBBERS

All snubbers are required OPERABLE to ensure that the structural integrity of the reactor coolant system and all other safety related systems is maintained during and following a seismic or other event initiating dynamic loads. Snubbers excluded from this inspection program are those installed on non-safety related systems and then only if their failure or failure of the system on which they are installed, would have no adverse effect on any safety-related system.

The visual inspection frequency is based upon maintaining a constant level of snubber protection to systems. Therefore, the required inspection interval varies inversely with the observed snubber failures and is determined by the number of inoperable snubbers found during an inspection. Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection. However, the results of such early inspections performed before the original required time interval has elapsed (nominal time less 25%) may not be used to lengthen the required inspection interval. Any inspection whose results require a shorter inspection interval will override the previous schedule.

When the cause of the rejection of a snubber is clearly established and remedied for that snubber and for any other snubbers that may be generically susceptible, and verified by inservice functional testing, that snubber may be exempted from being counted as inoperable. Generically susceptible snubbers are those which are (1) of a specific make or model, (2) of the same design, and (3) similarly located or exposed to the same environmental conditions such as temperature, radiation, and vibration. These characteristics of the snubber installation shall be evaluated to determine if further functional testing of similar snubber installations is warranted.

When a snubber is found inoperable, an engineering evaluation is performed, in addition to the determination of the snubber mode of failure, in order to determine if any safety-related component or system has been adversely affected by the inoperability of the snubber. The engineering evaluation shall determine whether or not the snubber mode of failure has imparted a significant effect or degradation on the supported component or system.

To provide assurance of snubber functional reliability, a representative sample of the installed snubbers will be functionally tested during plant shutdowns at 18 month intervals. Observed failures of these sample snubbers shall require functional testing of additional units.

The service life of a snubber is evaluated via manufacturer input and information through consideration of the snubber service conditions and associated installation and maintenance records (newly installed snubber, seal replaced, spring replaced, in high radiation area, in high temperature area, etc. . . .). The requirement to monitor the snubber service life is included to ensure that the

PLANT SYSTEMS

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snubbers periodically undergo a performance evaluation in view of their age and operating conditions. The service life program is designed to uniquely reflect the conditions at Calvert Cliffs. The criteria for evaluating service life shall be determined, and documented, by the licensee. Records will provide statistical bases for future consideration of snubber service life. The requirements for the maintenance of records and the snubber service life review are not intended to affect plant operation.

3/4.7.9 SEALED SOURCE CONTAMINATION

The limitations on removable contamination for sources requiring leak testing, including alpha emitters, is based on 10 CFR 70.39(c) limits for plutonium. This limitation will ensure that leakage from byproduct, source, and special nuclear material sources will not exceed allowable intake values.

3/4.7.10 WATERTIGHT DOORS

This specification is provided to ensure the protection of safety related equipment from the effects of water or steam escaping from ruptured pipes or components in adjoining rooms.

3/4.7.11 FIRE SUPPRESSION SYSTEMS

The OPERABILITY of the fire suppression systems ensures that adequate fire suppression capability is available to confine and extinguish fires occurring in any portion of the facility where safety related equipment is located. The fire suppression system consists of the water system, spray and/or sprinklers, Halon and fire hose stations. The collective capability of the fire suppression systems is adequate to minimize potential damage to safety related equipment and is a major element in the facility fire protection program.

In the event that portions of the fire suppression systems are inoperable, alternate backup fire fighting equipment is required to be made available in the affected areas until the inoperable equipment is restored to service. Where a continuous fire watch is required in lieu of fire protection equipment and habitability due to heat or radiation is a concern, the fire watch should be stationed in a habitable area as close as possible to the inoperable equipment.

In the event the fire suppression water system becomes inoperable, immediate corrective measures must be taken since this system provides the major fire suppression capability of the plant. The requirement for a twenty-four hour report to the Commission provides for prompt evaluation of the acceptability of the corrective measures to provide adequate fire suppression capability for the continued protection of the nuclear plant.

ADMINISTRATIVE CONTROLS

6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. All REPORTABLE OCCURRENCES submitted to the Commission.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to Operating Procedures.
- g. Records of radioactive shipments.
- h. Records of sealed source and fission detector leak tests and results.
- i. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.

ADMINISTRATIVE CONTROLS

- d. Records of radiation exposure for all individuals entering radiation control areas.
- e. Records of gaseous and liquid radioactive material released to the environs.
- f. Records of transient or operational cycles for those facility components identified in Table 5.7.1.
- g. Records of training and qualification for current members of the plant staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities identified in the NRC approved QA Manual as lifetime records.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the POSRC and the OSSRC.
- l. Records of Environmental Qualification which are covered under the provisions of paragraph 6.13.
- m. Records of the service lives of all snubbers listed on Table 3.7-4 including the date at which the service life commences and associated installation and maintenance records.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20:

- a. A high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Special or Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 64 AND 46 TO

FACILITY OPERATING LICENSE NOS. DPR-53 AND DPR-69

BALTIMORE GAS AND ELECTRIC COMPANY

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-317 AND 50-318

Introduction

By application for license amendment dated July 30, 1981, the Baltimore Gas and Electric Company (BG&E) requested changes to the Technical Specifications (TS) for Calvert Cliffs Units 1 and 2. The proposed changes provide updated Limiting Conditions for Operation (LCO'S) and Surveillance Requirements for dynamic component restraints (snubbers).

The July 30, 1981 application was in response to an NRC letter dated November 20, 1980, which provided model TS and requested BG&E to provide comparable TS. In the course of reviewing the July 30, 1981 application, we have found it necessary to make changes in the proposed TS. These changes have been discussed with Baltimore Gas and Electric and found to be acceptable.

In addition to TS changes associated with snubbers, we herein consider TS changes associated with the station reserve battery. The TS for the station reserve battery had been issued by License Amendments 58 and 40 for Calvert Cliffs Units 1 and 2, respectively.

Certain TS, associated with station battery surveillance requirements, were not addressed at that time and are considered herein.

Discussion and Evaluation

By letter dated November 20, 1980, the NRC provided BG&E with model TS that incorporated a revised surveillance program for snubbers. The November 20, 1980 letter also requested that BG&E submit proposed TS that conform to the model TS. BG&E responded to the November 20, 1980 NRC request by submitting proposed TS in their application for license amendment dated July 30, 1981. The proposed TS are addressed herein.

Limiting Conditions For Operation

The proposed LCO, TS 3.7.8.1, allows 72 hours to restore an inoperable safety-related snubber to operable status. After 72 hours, the system supported by the snubber is declared inoperable and the associated TS become applicable. The existing LCO, TS 3.7.8.1, requires inoperable snubbers to be made operable within 72 hours after which the reactor is to be in cold shutdown within 36 hours. The proposed TS, which is the same as the model TS, allows remedial action to be taken at a level which reflects the safety importance of the affected system, and is acceptable.

The proposed TS 3.7.8.1 also requires an "engineering evaluation" to be performed prior to returning a snubber to operable status. An engineering evaluation, at a minimum, consists of a documented, visual, inspection of the component supported by the inoperable snubber to assure that no observed damage has occurred to the component. This requirement sufficiently conforms to the model TS and is acceptable.

Surveillance Requirement-Visual Inspection

The existing visual inspection program, T S 4.7.8.1a, and b requires inspection of all snubbers and their installation. For snubbers with seal material made from ethylene propylene, which has been shown to be compatible with operating environments, the inspection interval is 4 months. For other seal material, not demonstrated to be compatible with operating environment, the inspection interval is 31 days. The proposed visual inspection requirements, T S 4.7.8.1a and b, do not discriminate between ethylene propylene and other seal materials. (All snubber seals presently employed at Calvert Cliffs utilize ethylene propylene.) The surveillance interval, however, provides for a sliding scale which is established based upon the number of snubbers found to be inoperable during an inspection. The maximum interval is 18 months, + 25%, if no failures occur. The minimum interval is 31 days +25%, if 8 or more failures are observed. The proposed visual inspection strategy encourages the licensee to adopt proven maintenance methods and materials with the goal of maximizing the inspection interval. The acceptance criteria for the visual inspection, contained in proposed TS 4.7.8.1a, is substantially the same as the existing criteria in TS 4.7.8.1a and 1b. One change from existing criteria is that snubbers which do not meet the criteria for visual inspection can be eliminated from the "failure lists" for the purposes of establishing the inspection interval. Such elimination is provided by successful functional testing and/or by establishing an obvious reason (and remedial action) for the snubber failure.

The proposed T S sufficiently conform to the mode TS and are acceptable.

Surveillance Requirements-Functional Test

The proposed functional test TS contained in TS 4.7.8.c, are substantially the same as existing TS 4.7.8.c. Snubbers in the following categories, however, had been exempted from periodic functional testing and would now require such testing: (1) snubbers located in high (radiation) exposure zones, (2) snubbers that are especially difficult to remove, and (3) snubbers with greater than 50,000 lbs capacity. With regard to functional testing of the 50,000 lbs capacity snubbers, the proposed TS would exempt these snubbers from functional testing until the refueling following June 30, 1985.

The proposed functional tests are substantially in conformance with the model TS. With regard to the proposed temporary exemption for functional testing of the 50,000 lbs. capacity snubbers, we understand that these snubbers must be modified to allow in place functional testing. Our review of the licensee's schedule for modification of these snubbers indicates that the proposed temporary exemption is acceptable.

Surveillance Requirements-Snubber Service Life Monitoring

Proposed TS 4.7.8.1e requires that the licensee establish a record keeping and analysis effort in order to periodically assess snubber service life. If during an 18 month review, snubbers are identified which exceed their service life, or will exceed their service life prior to the next review, the snubber service life shall be reevaluated, the snubber reconditioned, or the snubber replaced. The licensee has further requested that implementation of the life program be delayed until January 1, 1983.

Our review of the service life TS indicates that it is in substantial agreement with the model TS. We agree that a delay in implementation of this program until January 1, 1983 is appropriate since (1) key service life factors must be identified by the licensee, (2) the administrative features of the program must be established, and (3) significant input from the snubber vendor is expected to be required.

In summary, the LCO and Surveillance Requirements are substantially in conformance with the snubber model TS issued in November 1980. The proposed TS will improve the overall reliability of the snubbers at Calvert Cliffs by including snubbers in the surveillance program which had previously been exempted from testing. In addition, the proposed TS will establish a snubber service life review program which we believe will significantly improve the licensee's awareness of factors associated with extended service life of snubbers.

With regard to the station reserve battery, on November 2, 1981 the NRC issued License Amendments 58 and 40, for Calvert Cliffs Units 1 and 2, which approved the installation and use of the reserve battery. In our Safety Evaluation in support of License Amendments 58 and 40 we stated that the reserve battery would be subjected to the same surveillance requirements as the existing 125-volt D.C. batteries; however no specific test was specified to determine reserve battery capacity. The licensee subsequently proposed the assignment of the reserve battery to the same "dummy" load group as used for establishing the battery capacity of batteries 11 and 21. In addition, the restriction that the 60 month capacity test should be conducted during shutdown is no longer appropriate. Since the reserve battery can be utilized to replace any of the four 125 volt DC batteries, a single 125 volt DC battery can be taken out of service at any time to perform the capacity test or for other reasons. Accordingly, it is appropriate to delete the existing restriction on performance of the 60 month battery capacities test.

The deletion of the restriction on capacity testing and the specification of the load group for the reserve battery capacity test are consistent with our Safety Evaluation of November 2, 1981. These changes do not affect the reliability of the 125 volt D.C. batteries or the availability of emergency D.C. power.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 28, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSION
DOCKET NOS. 50-317 AND 318
BALTIMORE GAS AND ELECTRIC COMPANY
NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 64 and 46 to Facility Operating Licenses Nos. DPR-53 and DPR-69, issued to Baltimore Gas and Electric Company, which revised Technical Specifications for operation of the Calvert Cliffs Nuclear Power Plant, Units Nos. 1 and 2. The amendments are effective as of the date of issuance.

These amendments revise the Technical Specifications to provide updated Limiting Conditions for Operation and Surveillance Requirements for dynamic component restraints (snubbers), delete the restriction on performance of the 60 month reserve battery capacity test, and assign the reserve battery to a capacity test load group.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of the amendments was not required since the amendments do not involve a significant hazards consideration.

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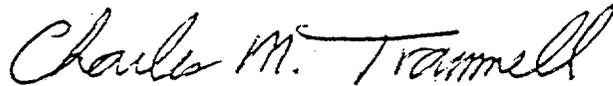
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The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendments.

For further details with respect to this action, see (1) the application for amendment dated July 30, 1981, (2) Amendment Nos. 64 and 46 to License Nos. DPR-53 and DPR-69, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D.C. and at the Calvert County Library, Prince Frederick, Maryland. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 28th day of December, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION



Charles M. Trammell, Acting Chief
Operating Reactors Branch #3
Division of Licensing