

March 17, 1978

Docket Nos. 50-317 ✓
and 50-318

Baltimore Gas and Electric Company
ATTN: Mr. A. E. Lundvall, Jr.
Vice President - Supply
P. O. Box 1475
Baltimore, Maryland 21203

Gentlemen:

DISTRIBUTION:

Docket(2) ✓
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The Commission has issued the enclosed Amendments Nos.30 and 15 to Facility Operating Licenses Nos. DPR-53 and DPR-69 for the Calvert Cliffs Nuclear Power Plant Units Nos. 1 and 2, respectively. The amendments are in accordance with your request dated June 17, 1977, and supplementing information dated November 9, 1977.

These amendments revise the Appendix A Technical Specifications by:

- (1) modifying the review process for revisions to the site security plan and the emergency plan for each unit (Items 1 and 2 of the letter dated June 17, 1977),
- (2) correcting the safety-related hydraulic snubber list to as-built condition for Unit No. 1 (Item 3), and
- (3) allowing limited battery maintenance during reactor operation for each unit (Items 4 and 5).

At the August 31, 1977, meeting on battery capacity testing, the NRC staff and BG&E representatives agreed to postpone evaluation of Items 6 and 7 of the letter of June 17, 1977, until such time as BG&E submits additional information.

Cond. D.
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Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 30 to DPR-53
- 2. Amendment No. 15 to DPR-69
- 3. Safety Evaluation
- 4. Notice

cc w/enclosures:
See next page

OFFICE →	ORB#4:DOR	ORB#4:DOR	SPTS	OELD	C-ORB#4:DOR
SURNAME →	RIngram <i>in</i>	EComer.dn <i>SEC</i>	JMcGough	<i>C. Woodhead</i>	RReid
DATE →	3/13/78	3/14/78	3/14/78	3/15/78	3/17/78

Baltimore Gas and Electric Company

cc w/enclosures:

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Region III Office
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Sixth & Walnut Streets
Philadelphia, Pennsylvania 19106

cc w/4 encls. & incoming dtd:
6/17/77 & 11/9/77

Administrator, Power Plant Siting Program
Energy and Coastal Zone Administration
Department of Natural Resources
Tawes State Office Building
Annapolis, Maryland 21401



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS & ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 30
License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated June 17, 1977, as supplemented November 9, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-53 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 30, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 17, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 30

FACILITY OPERATING LICENSE NO. DPR-53

DOCKET NO. 50-317

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

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3/4 7-29
3/4 7-30
3/4 7-31
3/4 7-34
3/4 7-37
3/4 7-38
3/4 7-39
3/4 7-45
3/4 7-61
3/4 7-61a (added)
3/4 8-8
6-7

ADMINISTRATIVE CONTROLS

AUTHORITY

6.5.1.7 The Plant Operations and Safety Review Committee shall:

- a. Recommend to the Chief Engineer written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Manager - Electric Production Department and the Off Site Safety Review Committee of disagreement between the POSRC and the Chief Engineer; however, the Chief Engineer shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The POSRC shall maintain written minutes of each meeting and copies shall be provided to the Manager - Electric Production Department and Chairman of the Off Site Safety Review Committee.

6.5.2 OFF SITE SAFETY REVIEW COMMITTEE (OSSRC)

FUNCTION

6.5.2.1 The Off Site Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The POSRC shall meet at least once per calendar month and as convened by the POSRC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the POSRC shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The POSRC shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Chief Engineer to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Manager - Electric Production Department and to the Chairman of the Off Site Safety Review Committee.
- f. Review all written reports requiring 24 hour notification to the Commission.
- g. Review of facility operations to detect potential safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Off Site Safety Review Committee.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Off Site Safety Review Committee.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Off Site Safety Review Committee.

ELECTRICAL POWER SYSTEMS

D.C. DISTRIBUTION - OPERATING

LIMITING CONDITION FOR OPERATION

3.8.2.3 The following D.C. bus trains shall be energized and OPERABLE:

- a. 125-volt D.C. bus No. 11, 125 volt D. C. battery bank No. 11 and a full capacity charger.
- b. 125-volt D.C. bus No. 12, 125-volt D. C. battery bank No. 12 and a full capacity charger.
- c. 125-volt D.C. bus No. 21, 125-volt D. C. battery bank No. 21 and a full capacity charger.
- d. 125-volt D.C. bus No. 22, 125-volt D. C. battery bank No. 22 and a full capacity charger.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. With one 125-volt D.C. bus inoperable, restore the inoperable bus to OPERABLE status within 2 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With one 125-volt D.C. battery and/or its charger inoperable, except during surveillance testing per Specifications 4.8.2.3.2.c.2, 4.8.2.3.2.d and 4.8.2.3.2.e, restore the inoperable battery and/or charger to OPERABLE status within 2 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With one 125-volt D.C. battery inoperable during surveillance testing of the battery per Specifications 4.8.2.3.2.c.2, 4.8.2.3.2.d and 4.8.2.3.2.e, operation may continue provided the associated bus is being powered by an OPERABLE charger.

SURVEILLANCE REQUIREMENTS

4.8.2.3.1 Each D.C. bus train shall be determined OPERABLE and energized at least once per 7 days by verifying correct breaker alignment and indicated power availability.

4.8.2.3.2 Each 125-volt battery bank and charger shall be demonstrated OPERABLE:

ELECTRICAL POWER SYSTEMS

A.C. DISTRIBUTION - SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.8.2.2 As a minimum, the following A.C. electrical busses shall be OPERABLE and energized from sources of power other than a diesel generator but aligned to an OPERABLE diesel generator:

- 1 - 4160 volt Emergency Bus
- 1 - 480 volt Emergency Bus
- 2 - 120 volt A.C. Vital Busses

APPLICABILITY: MODES 5 and 6

ACTION:

With less than the above complement of A.C. busses OPERABLE and energized, establish CONTAINMENT INTEGRITY within 8 hours.

SURVEILLANCE REQUIREMENTS

4.8.2.2 The specified A.C. busses shall be determined OPERABLE and energized from A.C. sources other than the diesel generators at least once per 7 days by verifying correct breaker alignment and indicated power availability.

TABLE 4.7-4

HYDRAULIC SNUBBER INSPECTION SCHEDULE

<u>NUMBER OF SNUBBERS FOUND INOPERABLE DURING INSPECTION OR DURING INSPECTION INTERVAL*</u>	<u>NEXT REQUIRED INSPECTION INTERVAL**</u>
0	18 months \pm 25%
1	12 months \pm 25%
2	6 months \pm 25%
3 or 4	124 days \pm 25%
5, 6, or 7	62 days \pm 25%
>8	31 days \pm 25%

* Snubbers may be categorized into two groups, "accessible" and "inaccessible". This categorization shall be based upon the snubber's accessibility for inspection during reactor operation. These two groups may be inspected independently according to the above schedule.

** The required inspection interval shall not be lengthened more than one step at a time.

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-83-75	AUXILIARY STEAM ISOLATION VALVE BYPASS 32'	A	No	No
1-83-76	AUXILIARY FEED PUMP STEAM SUPPLY FROM S.G. #12 40'	A	No	No
1-83-76A	AUXILIARY FEED PUMP STEAM SUPPLY FROM S.G. #12 40'	A	No	No
1-83-77	AUXILIARY FEED PUMP STEAM SUPPLY FROM S.G. #12 40'	A	No	No
1-83-78	#11 AUXILIARY FEED PUMP TURBINE STEAM SUPPLY 16'	A	No	No

*Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-4 provided that a revision to Table 3.7-4 is included with the next License Amendment request.

**Modification to this table due to changes in high radiation areas shall be submitted to the NRC as part of the next License Amendment request.

TABLE 3.7-4
SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-83-55	MAIN STEAM LINE ENCAPSULATION 27'	A	No	No
1-83-56	MAIN STEAM LINE ENCAPSULATION 27'	A	No	No
1-83-57	MAIN STEAM LINE ENCAPSULATION 27'	A	No	No
1-83-58	MAIN STEAM LINE ENCAPSULATION 27'	A	No	No
1-83-59	MAIN STEAM LINE ENCAPSULATION 27'	A	No	No
1-83-60	MAIN STEAM LINE ENCAPSULATION 27'	A	No	No
1-83-61	MAIN STEAM LINE ENCAPSULATION 27'	A	No	No
1-83-62	MAIN STEAM LINE ENCAPSULATION 27'	A	No	No
1-83-67	MAIN STEAM FROM S.G. #12 61'	I	Yes	No
1-83-69	MAIN STEAM FROM S.G. #12 61'	I	Yes	No
1-83-70	MAIN STEAM FROM S.G. #12 61'	I	Yes	No
1-83-71	MAIN STEAM FROM S.G. #12 61'	I	Yes	No
1-83-73	MSIV #11 HYDRAULIC SUPPLY 38'	A	No	No
1-83-74	MSIV #11 HYDRAULIC SUPPLY 38'	A	No	No

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-60-1	SERVICE WATER TO CONTAINMENT COOLER #12 42'	I	Yes	No
1-60-2	SERVICE WATER TO CONTAINMENT COOLER #12 43'	I	Yes	No
1-60-3	SERVICE WATER TO CONTAINMENT COOLER #12, 65'	I	Yes	No
1-60-4	SERVICE WATER TO CONTAINMENT COOLER #14 66'	I	Yes	No
1-60-4A	SERVICE WATER FROM CONTAINMENT COOLER #14 66'	I	Yes	No
1-60-5	SERVICE WATER FROM CONTAINMENT COOLER #14, 68'	I	Yes	No
1-60-5A	SERVICE WATER FROM CONTAINMENT COOLER #14 68'	I	Yes	No
1-60-6	SERVICE WATER FROM CONTAINMENT COOLER #12, 53'	I	Yes	No
1-60-7	SERVICE WATER FROM CONTAINMENT COOLER #13 64'	I	Yes	No

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-54-12	ERV-404 DISCHARGE 89'	I	Yes	No
1-54-14	RECIRC. RELIEF VALVE DISCH. TO QUENCH TANK 28'	I	Yes	No
1-54-15	RECIRC. RELIEF VALVE DISCH. TO QUENCH TANK 28'	I	Yes	No
1-54-16	RECIRC. RELIEF VALVE DISCH. TO QUENCH TANK 40'	I	Yes	No
1-54-17	RECIRC. RELIEF VALVE DISCH. TO QUENCH TANK 49'	I	Yes	No
1-54-18	ERV-404 RV 201 DISCH. HEADER 89'	I	Yes	No
1-54-19	RV 201 DISCHARGE 89'	I	Yes	No
1-54-20	RV 200, RV 201 SEAT DRAIN HEADER 83'	I	Yes	No
1-54-21	RV 200, RV201 SEAT DRAIN HEADER 77'	I	Yes	No
1-54-23	PRESSURIZER RELIEF 90'	I	Yes	No

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-52-47	LPSI TO LOOP 11B 46'	I	Yes	No
1-52-48	LPSI TO LOOP 11B 44'	I	Yes	No
1-52-49	LPSI TO LOOP 11B (PENETRATION) 42'	I	Yes	No
1-52-50	LPSI TO LOOP 11B (PENETRATION) 42'	I	Yes	No
1-52-50A	LPSI TO LOOP 11B (PENETRATION) 42'	I	Yes	No
1-52-51	SI TO LOOP 11B 48'	I	Yes	No
1-52-52	SI TO LOOP 11B 48'	I	Yes,	No
1-52-53	SI TO LOOP 12B 48'	I	Yes	No
1-52-54	SI TO LOOP 12B 48'	I	Yes	No
1-52-54A	SI TO LOOP 12B 48'	I	Yes	No
1-52-55	SI TO LOOP 12B 48'	I	Yes	No
1-52-56	SI TO LOOP 12B (UPSTREAM MOV 644) 48'	I	Yes	No

TABLE 3.7-4
SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-52-39	SI TO LOOP 12A 48'	I	Yes	No
1-52-39A	SI TO LOOP 12A 48'	I	Yes	No
1-52-40	LPSI TO SI LOOP 12A 46'	I	Yes	No
1-52-41	SI TO LOOP 12B (UPSTREAM MOV 644) 41'	I	Yes	No
1-52-42	SI TO LOOP 11B 48'	I	Yes	No
1-52-42A	SI TO LOOP 11B 48'	I	Yes	No
1-52-43	SI TO LOOP 11B (UPSTREAM MOV 624) 48'	I	Yes	No
1-52-43A	SI TO LOOP 11B (UPSTREAM MOV 624) 48'	I	Yes	No
1-52-44	SI TO LOOP 11B (UPSTREAM MOV 624) 48'	I	Yes	No
1-52-45	SI TANK 11A DISCHARGE 55'	I	Yes	No
1-52-46	SI TANK 11B DISCHARGE 57'	I	Yes	No
1-52-46A	SI TANK 11B DISCHARGE 57'	I	Yes	No

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-52-29	SI TANK 11A DISCHARGE 48'	I	Yes	No
1-52-30	HPSI TO LOOP 11A 46'	I	Yes	No
1-52-31	SI TO LOOP 11A 48'	I	Yes	No
1-52-32	SI TO LOOP 11A (UPSTREAM MOV 614) 48'	I	Yes	No
1-52-32A	SI TO LOOP 11A (UPSTREAM MOV 614) 48'	I	Yes	No
1-52-33	SI TANK 12A DISCHARGE 55'	I	Yes	No
1-52-34	SI TANK 12A DISCHARGE 56'	I	Yes	No
1-52-34A	SI TANK 12A DISCHARGE 56'	I	Yes	No
1-52-35	SI TO LOOP 12A (UPSTREAM MOV 634) 48'	I	Yes	No
1-52-36	SI TO LOOP 12A 48'	I	Yes	No
1-52-37	SI TO LOOP 12A 48'	I	Yes	No
1-52-37A	SI TO LOOP 12A 48'	I	Yes	No
1-52-38	SI TO LOOP 12A 48'	I	Yes	No

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-52-23	SI SUCTION FROM RWT 34'	I	Yes	No
1-52-24	SD COOLING UPSTREAM ISOLATION VALVE 25'	I	Yes	No
1-52-25	SD COOLING UPSTREAM ISOLATION VALVE 20'	I	Yes	No
1-52-25A	SD COOLING UPSTREAM ISOLATION VALVE 20'	I	Yes	No
1-52-25B	SD COOLING UPSTREAM ISOLATION VALVE 20'	I	Yes	No
1-52-26	SD COOLING UPSTREAM ISOLATION VALVE 31'	I	Yes	No
1-52-26A	SD COOLING UPSTREAM ISOLATION VALVE 31'	I	Yes	No
1-52-27	SD COOLING UPSTREAM ISOLATION VALVE 31'	I	Yes	No
1-52-28	SD COOLING UPSTREAM ISOLATION VALVE 31'	I	Yes	No
1-52-28A	SD COOLING UPSTREAM ISOLATION VALVE 31'	I	Yes	No

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-41-16	CHARGING LINE LOOP 11A 24'	I	Yes	No
1-41-17	CHARGING LINE LOOP 11A 24'	I	Yes	No
1-41-18	LETDOWN LINE 24'	I	Yes	No
1-45-1	FEEDWATER INLET TO S.G. #11 40'	I	Yes	No
1-45-1A	FEEDWATER INLET TO S.G. #11 40'	I	Yes	No
1-45-2	FEEDWATER INLET TO S.G. #12 50'	I	Yes	No
1-45-3	FEEDWATER INLET TO S.G. #12 55'	I	Yes	No
1-45-6	AUX. FEEDWATER INLET TO S.G. #12 62'	I	Yes	No
1-45-7	AUX. FEEDWATER INLET TO S.G. #11 65'	I	Yes	No
1-45-7A	AUX. FEEDWATER INLET TO S.G. #11 65'	I	Yes	No
1-45-8	STEAM SUPPLY #12 AUX. FEED PUMP 12'	A	No	No

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-41-4	AUX. SPRAY 65'	I	Yes	No
1-41-5	AUX. SPRAY 49'	I	Yes	No
1-41-6	CHARGING LINE LOOP 12B 46'	I	Yes	No
1-41-7	CHARGING LINE LOOP 12B 46'	I	Yes	No
1-41-8	CHARGING LINE LOOP 12B 46'	I	Yes	No
1-41-9	CHARGING LINE LOOP 12B 38'	I	Yes	No
1-41-10	CHARGING LINE LOOP 11A 34'	I	Yes	No
1-41-11	CHARGING LINE LOOP 11A 25'	I	Yes	No
1-41-12	CHARGING LINE LOOP 11A 24'	I	Yes	No
1-41-13	CHARGING LINE LOOP 11A 24'	I	Yes	No
1-41-14	CHARGING LINE LOOP 11A 46'	I	Yes	No
1-41-15	CHARGING LINE LOOP 11A 46'	I	Yes	No

CALVERT CLIFFS-UNIT 1

3/4 7-32

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-24-3	EMERGENCY DIESEL #12 EXHAUST 61'	A	No	No
1-24-3A	EMERGENCY DIESEL #12 EXHAUST 61'	A	No	No
1-24-4	EMERGENCY DIESEL #11 EXHAUST 61'	A	No	No
1-24-4A	EMERGENCY DIESEL #11 EXHAUST 61'	A	No	No
1-24-5	DIESEL GENERATOR #21 EXHAUST 92'	A	No	No
1-24-6	DIESEL GENERATOR #21 EXHAUST 62'	A	No	No
1-24-6A	DIESEL GENERATOR #21 EXHAUST 62'	A	No	No
1-41-1	SUCTION #13 CHARGING PUMP -10'	A	No	No
1-41-2	AUX. SPRAY 65'	I	Yes	No
1-41-3	AUX. SPRAY 65'	I	Yes	No

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-36-1	SUCTION #11 AUX. FEED PUMP 12'	A	No	No
1-36-1A	SUCTION #11 AUX. FEED PUMP 12'	A	No	No
1-36-3	DISCHARGE #11 AUX. FEED PUMP 12'	A	No	No
1-36-3A	DISCHARGE #11 AUX. FEED PUMP 12'	A	No	No
1-36-4	DISCHARGE #12 AUX. FEED PUMP 12'	A	No	No
1-36-4A	DISCHARGE #12 AUX. FEED PUMP 12'	A	No	No
1-38-1	PRESSURIZER SAMPLE LINES 13'	A	Yes	No
1-38-2	PRESSURIZER SAMPLE LINES 13'	A	Yes	No
1-38-4	PRESSURIZER SAMPLE LINES 13'	A	Yes	No
1-38-5	PRESSURIZER SAMPLE LINES 13'	A	Yes	No
1-24-1	DIESEL GENERATOR #12 EXHAUST 92'	A	No	No
1-24-2	DIESEL GENERATOR #11 EXHAUST 92'	A	No	No

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-15-2	SUCTION HEADER COMPONENT COOLING PUMPS 5'	A	No	No
1-15-3	COMPONENT COOLING PUMP SUCTION FROM COMPONENT COOLING HEAD TANK 5'	A	No	No
1-15-4	COMPONENT COOLING PUMP DISCHARGE HEADER 5'	A	No	No
1-15-5	COMPONENT COOLING PUMP #12 DISCHARGE 5'	A	No	No
1-15-6	COMPONENT COOLING PUMP DISCHARGE HEADER 5'	A	No	No
1-15-6A	COMPONENT COOLING PUMP DISCHARGE HEADER 5'	A	No	No
1-15-7	COMPONENT COOLING INLET #11 LEIDOWN HX 5'	A	No	No
1-15-8	COMPONENT COOLING FROM WASTE EVAPS. 45'	A	No	No
1-15-9	COMPONENT COOLING WATER FROM MISC. -10'	A	No	No
1-15-11	WASIE PROCESSING SYSEM -10'	A	No	No

TABLE 3.7-4
SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-11-20	SERVICE WATER PUMP #12 DISCHARGE 5'	A	No	No
1-11-21	SERVICE WATER PUMP #12 SUCTION 5'	A	No	No
1-11-22	SERVICE WATER PUMP #13 DISCHARGE 5'	A	No	No
1-12-1	DISCHARGE #12 COMPONENT COOLING HX 15'	A	No	No
1-12-2	DISCHARGE #12 SERVICE WATER HX 15'	A	No	No
1-12-3	DISCHARGE #11 SERVICE WATER HX 15'	A	No	No
1-12-4	INLET #11 COMPONENT COOLING HX 15'	A	No	No
1-12-5	INLET #12 COMPONENT COOLING HEAT EXCHANGER 15'	A	No	No
1-12-5A	INLET #12 COMPONENT COOLING HEAT EXCHANGER 15'	A	No	No
1-15-1	SUCTION HEADER COMPONENT COOLING PUMPS 5'	A	No	No

TABLE 3.7-4
SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-11-11	SERVICE WATER FROM CONTAINMENT COOLERS 5'	A	No	Yes
1-11-11A	SERVICE WATER PUMP SUCTION HDR 5'	A	No	Yes
1-11-12	SERVICE WATER FROM CONTAINMENT COOLERS 5'	A	No	Yes
1-11-13	SERVICE WATER FROM CONTAINMENT COOLERS 5'	A	No	No
1-11-14	SERVICE WATER FROM CONTAINMENT COOLERS 5'	A	No	No
1-11-16	SERVICE WATER PUMP DISCHARGE HEADER 5'	A	No	No
1-11-17	SERVICE WATER PUMP DISCHARGE HEADER 5'	A	No	No
1-11-18	SERVICE WATER PUMP DISCHARGE HEADER 5'	A	No	No
1-11-18A	SERVICE WATER PUMP DISCHARGE HEADER 5'	A	No	No
1-11-19	SERVICE WATER PUMP DISCHARGE HEADER 5'	A	No	No

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-11-1	SERVICE WATER PUMP #13 SUCTION 5'	A	No	No
1-11-2	SERVICE WATER PUMP #13 SUCTION 5'	A	No	No
1-11-3	SERVICE WATER PUMP #13 SUCTION 5'	A	No	No
1-11-4	SERVICE WATER PUMP #12 SUCTION 5'	A	No	No
1-11-5	SERVICE WATER PUMP #12 SUCTION 5'	A	No	No
1-11-6	SERVICE WATER PUMP #11 SUCTION 5'	A	No	No
1-11-7	SERVICE WATER PUMP #11 SUCTION 5'	A	No	No
1-11-8	SERVICE WATER PUMP #11 SUCTION 5'	A	No	No
1-11-9	SERVICE WATER PUMP #11 SUCTION 5'	A	No	No
1-11-10	SERVICE WATER HEADER FROM TURBINE BLDG. 5'	A	No	No



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS & ELECTRIC COMPANY

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15
License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated June 17, 1977, as supplemented November 9, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.2. of Facility Operating License No. DPR-69 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 15, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 17, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 15

FACILITY OPERATING LICENSE NO. DPR-69

DOCKET NO. 50-318

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 8-8

6-7

ELECTRICAL POWER SYSTEMS

A.C. DISTRIBUTION - SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.8.2.2 As a minimum, the following A.C. electrical busses shall be OPERABLE and energized from sources of power other than a diesel generator but aligned to an OPERABLE diesel generator:

- 1 - 4160 volt Emergency Bus
- 1 - 480 volt Emergency Bus
- 2 - 120 volt A.C. Vital Busses

APPLICABILITY: MODES 5 and 6

ACTION:

With less than the above complement of A.C. busses OPERABLE and energized, establish CONTAINMENT INTEGRITY within 8 hours.

SURVEILLANCE REQUIREMENTS

4.8.2.2 The specified A.C. busses shall be determined OPERABLE and energized from A.C. sources other than the diesel generators at least once per 7 days by verifying correct breaker alignment and indicated power availability.

ELECTRICAL POWER SYSTEMS

D.C. DISTRIBUTION - OPERATING

LIMITING CONDITION FOR OPERATION

3.8.2.3 The following D.C. bus trains shall be energized and OPERABLE:

- a. 125-volt D.C. bus No. 11, 125 volt D. C. battery bank No. 11 and a full capacity charger.
- b. 125-volt D.C. bus No. 12, 125-volt D. C. battery bank No. 12 and a full capacity charger.
- c. 125-volt D.C. bus No. 21, 125-volt D. C. battery bank No. 21 and a full capacity charger.
- d. 125-volt D.C. bus No. 22, 125-volt D. C. battery bank No. 22 and a full capacity charger.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. With one 125-volt D.C. bus inoperable, restore the inoperable bus to OPERABLE status within 2 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With one 125-volt D.C. battery and/or its charger inoperable, except during surveillance testing per Specifications 4.8.2.3.2.c.2, 4.8.2.3.2.d and 4.8.2.3.2.e, restore the inoperable battery and/or charger to OPERABLE status within 2 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With one 125-volt D.C. battery inoperable during surveillance testing of the battery per Specifications 4.8.2.3.2.c.2, 4.8.2.3.2.d and 4.8.2.3.2.e, operation may continue provided the associated bus is being powered by an OPERABLE charger.

SURVEILLANCE REQUIREMENTS

4.8.2.3.1 Each D.C. bus train shall be determined OPERABLE and energized at least once per 7 days by verifying correct breaker alignment and indicated power availability.

4.8.2.3.2 Each 125-volt battery bank and charger shall be demonstrated OPERABLE:

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The POSRC shall meet at least once per calendar month and as convened by the POSRC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the POSRC shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The POSRC shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Chief Engineer to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Manager - Electric Production Department and to the Chairman of the Off Site Safety Review Committee.
- f. Review all written reports requiring 24 hour notification to the Commission.
- g. Review of facility operations to detect potential safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Off Site Safety Review Committee.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Off Site Safety Review Committee.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Off Site Safety Review Committee.

ADMINISTRATIVE CONTROLS

AUTHORITY

6.5.1.7 The Plant Operations and Safety Review Committee shall:

- a. Recommend to the Chief Engineer written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Manager - Electric Production Department and the Off Site Safety Review Committee of disagreement between the POSRC and the Chief Engineer; however, the Chief Engineer shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The POSRC shall maintain written minutes of each meeting and copies shall be provided to the Manager - Electric Production Department and Chairman of the Off Site Safety Review Committee.

6.5.2 OFF SITE SAFETY REVIEW COMMITTEE (OSSRC)

FUNCTION

6.5.2.1 The Off Site Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 30 TO LICENSE NO. DPR-53 AND
AMENDMENT NO. 15 TO LICENSE NO. DPR-69
BALTIMORE GAS AND ELECTRIC COMPANY
CALVERT CLIFFS NUCLEAR POWER PLANT UNITS NOS. 1 AND 2
DOCKETS NOS. 50-317 AND 50-318

1.0 Introduction

By letter dated June 17, 1977, and supplemental information dated November 9, 1977, Baltimore Gas and Electric Company (BG&E) proposed changes to the Technical Specifications (TS) for the Calvert Cliffs Nuclear Power Plant (CCNPP) Units Nos. 1 and 2 to:

- (1) modify the review process for revisions to the site security plan for each unit (Items 1 and 2),
- (2) correct the safety-related hydraulic snubber list for Unit No. 1 (Item 3), and
- (3) allow limited battery maintenance during reactor operation for each unit (Items 4 and 5).

2.0 Discussion and Evaluation

2.1 Review Process for Revisions to the Site Security Plan

BG&E proposed a change to Specification 6.5.1.6.i that would change the review process for revisions to the site security plan for both CCNPP units. Items 1 and 2 of the BG&E letter dated June 17, 1977, requested that the Chairman of the Off-Site Safety and Review Committee (OSSRC) be replaced by the Manager of Electric Production as review authority for changes to the Security Plan and its implementing procedures. This change was requested because the Manager of Electric Production has the corporate responsibility for site security. Based

on our review, we determined that this change would not conform to the independent review criterion since the Manager-Electric Production has line responsibility for operation of CCNPP. Following discussions with BG&E, they proposed, by letter dated November 9, 1977, that the review process be handled by their Chief Nuclear Engineer who reports to the Vice President - Engineering and Construction. The Chief Nuclear Engineer's position, since it does not have line responsibility for operation of CCNPP, meets the independent review criterion.

Notwithstanding the foregoing, the TS change being made requires that Plant Operating Safety Review Committee (POSRC) recommended changes to the Plant Security Plan and implementing procedures be submitted to the OSSRC rather than to an individual. The OSSRC may then refer the recommended changes to an individual for further review if they deem this appropriate. BG&E will implement the approved change by appropriate administrative procedures.

Although not requested by BG&E, the same change in wording was also made in Specification 6.5.1.6.j. This change requires that recommended changes to the Emergency Plan and implementing procedures be submitted to the OSSRC instead of the Chairman of the OSSRC. These changes are consistent with the policy expressed in the current Standard Technical Specifications. These changes have been discussed with and agreed to by BG&E.

2.2 Hydraulic Snubber List Corrections

In Item 3 of the June 17, 1977 letter, BG&E presented additions, deletions and changes to the Safety Related Hydraulic Snubbers listed in Table 3.7-4 of the Unit No. 1 TS. The proposed change would make editorial changes to the list of safety-related snubbers to reflect the actual as-built conditions as determined during the recent re-fueling outage snubber inspection. Certain snubbers have been redesignated as to number and location, and other snubbers need to be added to the list. No existing snubbers are being physically relocated or removed. This proposed change is administrative in nature and has no effect on the operability of safety-related snubbers and therefore has no effect on plant safety.

2.3 Battery Maintenance During Reactor Operation

BG&E also proposed two minor changes (Items 4 and 5 of the June 17, 1977 letter) for the D.C. distribution specification

for both units. These minor changes would allow the performance of required surveillance on the station batteries, in particular inspecting and cleaning the battery terminal connections, without invoking an action statement. Currently Specification 3.8.2.3 allows the performance of certain required routine maintenance without shutting down the reactor. This change would add cleaning of the battery terminals to this category. Due to the short duration of this activity, the likelihood of placing the plant in a reduced emergency power source posture during a postulated accident is not significant. Further, periodic cleaning of battery terminals is an essential element to assuring a reliable supply of D.C. power. Accordingly, we find the proposed change acceptable.

3.0 Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

4.0 Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 17, 1978

UNITED STATES NUCLEAR REGULATORY COMMISSION
DOCKETS NOS. 50-317 AND 50-318
BALTIMORE GAS AND ELECTRIC COMPANY
NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 30 and 15 to Facility Operating Licenses Nos. DPR-53 and DPR-69 (respectively), issued to Baltimore Gas and Electric Company (the licensee), which revised the Technical Specifications for operation of the Calvert Cliffs Nuclear Power Plant Units Nos. 1 and 2 (the facilities) located in Calvert County, Maryland. The amendments are effective as of their date of issuance.

The amendments revised the Technical Specifications by: (1) modifying the review process for revisions to the site security plan and the emergency plan for each unit; (2) correcting the safety-related hydraulic snubber list to as-built condition for Unit No. 1; and (3) allowing limited battery maintenance during reactor operation for each unit.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the

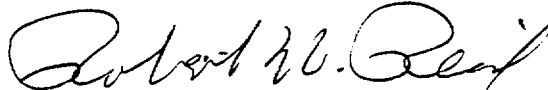
license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendments.

For further details with respect to this action, see (1) the application for amendments dated June 17, 1977, as supplemented November 9, 1977, (2) Amendment No. 30 to License No. DPR-53, and Amendment No. 15 to License No. DPR-69, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C., and at the Calvert County Library, Prince Frederick, Maryland. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 17th day of March 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors