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 November 16, 1974

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Docket No. 50-317

Baltimore Gas and Electric Company
ATTN: Mr. John W. Gore, Jr.
Vice President
Gas and Electric Building
Charles Center
Baltimore, Maryland 21203

Amendment No. 4
Change No. 3
License No. DPR-53

Gentlemen:

By letter dated November 14, 1974, Baltimore Gas and Electric Company (BG&E) requested that its Operating License No. CPR-53 for Calvert Cliffs Unit No. 1 be amended to modify the Technical Specifications for the period during which the reactor core internals are being inspected for the purpose of ascertaining the cause of a recent increase in the pressure drop across the core.

BG&E stated that it is desirable to handle fuel elements in air and provided an evaluation of a postulated fuel handling accident under the conditions for which it has proposed to be allowed to operate during the inspection period. In discussions between the Licensee and the AEC Licensing Project Manager concerning BG&E's requested amendments, we have learned that BG&E intends to remove a sufficient number of fuel elements from the reactor to permit access to the lower part of the core with a boroscope that will be used to inspect the core internals. BG&E estimates that approximately 12 elements may be required to be removed to provide the required access.

BG&E believes that it is desirable to inspect the fuel elements it removes from the core in a dry condition. In order to do this it proposes to leave the fuel pool dry to provide the location for inspecting the fuel. After inspection, the fuel assembly will be transferred through the fuel pool transfer tube to the spent fuel storage pool, which is also to be kept dry and stored dry in the spent fuel storage racks. The reactor core will be covered with borated water at refueling boron concentrations.

LB

BG&E's safety analysis of a postulated fuel handling accident assumes the release of all fission products in the gas gap of all of the rods in a fuel assembly.

Because of the low power level at which the plant has operated during the low power physics tests, the activity in the fuel assemblies is low and the amount of fission products available for release is relatively low. As a result, the consequences of a fuel handling accident will be much less severe than for the case of failure of an assembly that has operated at full reactor power that was evaluated in our review of the Calvert Cliffs License application. In fact they are on the order of the limits specified by 10 CFR Part 20.

BG&E has informed us that it will maintain the exposures to operating personnel during this operation to well within the range of exposures generally experienced in refueling shutdowns and as low as practicable.

On the basis of our consideration of the plans described by BG&E and of our evaluation of the safety analyses performed by BG&E we have concluded that there is no significant safety hazard consideration associated with the planned core and fuel inspection program proposed by BG&E. Accordingly, we will permit Technical Specifications 3.8.D, 3.8.A.6, 4.1, 4.3.C, 4.2, 4.5.B.5, 4.5.C.5, 4.5.F.4, 4.7-1.A.3, 4.7-1.B.1.c, 4.8, 4.9.B, 4.13.A and 6.2.2 of Appendix A of the license to be waived from 6 PM on November 16, 1974 until the inspection program is completed to the extent that the reactor pressure vessel head is reinstalled. During this interim period the remaining Technical Specifications in Appendix A will be supplemented by additional temporary Technical Specifications as provided in the enclosure.

We are hereby amending License No. DPR-53 by including Section 7 to Appendix A which will, for a specified period of time, delete certain Technical Specifications in Appendix A and supplement the remainder with Specifications needed to assure safe conditions during the planned inspection program. The amended Technical Specifications are presented as Change No. 3 to the Technical Specifications and is attached to the enclosed Amendment No. 4 to the Calvert Cliffs Unit Operating License.

Sincerely,

R. C. DeYoung, Assistant Director
for Light Water Reactors, Group 1
Directorate of Licensing

Enclosure:

see page 3

	L:LWR 1-3	L:LWR 1-3	L:AD/LWR 1		
SURNAME	GRivenbark:cls	ODParr	RCDeYoung		
DATE	11/15/74	11/18/74	11/18/74		

**Baltimore Gas and Electric
Company**

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Enclosures:

1. Amendment No. 4 to DPR-53
2. Federal Register Notice

**cc: Mr. James A. Biddison, Jr.
General Counsel
Gas and Electric Building
Charles Center
Baltimore, Maryland 21203**

**James C. Cawood, Jr., Esq.
Vice President
Chesapeake Environmental
Protection Association
4700 Auth Place
Camp Springs, Maryland**

**George F. Trowbridge, Esq.
Shaw, Pittman, Potts and
Trowbridge
910 17th Street, N. W.
Washington, D. C. 20006**

**Mr. Warren D. Hodges, Director
Department of State Planning
301 West Preston Street
Baltimore, Maryland 21201**

**Mr. Bernard Fowler, President
Board of County Commissioners
Prince Frederick, Maryland 20678**

**Mr. Robert J. Blanco, Region III
Environmental Protection Agency
Sixth and Walnut Streets
Philadelphia, Pennsylvania 19106**

**Mr. Bruce Blanchard
Environmental Projects Review
Department of the Interior
Room 5321
18th and C Streets, N. W.
Washington, D. C. 20240**

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**Baltimore Gas and Electric
Company**

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**Mr. Sheldon Myers
ATTN: Mr. Jack Anderson
Office of Federal Activities
EPA, Room W-541, Waterside Mall
401 M Street, S. W.
Washington, D. C. 20460**

**Colonel Howard Sargent
Executive Director of Civil Works
Office of the Chief of Engineers
Corps of Engineers
Department of Army
Forrestal Building, Room 4-G060
Washington, D. C. 20314**

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BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

**Amendment No. 4
License No. DPR-53**

1. The Atomic Energy Commission (the Commission) having found that:
 - A. The application for amendment by Baltimore Gas and Electric Company (the licensee) dated November 14, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended, and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2. C. (2) of Facility License No. DPR-53 is hereby amended to read as follows:

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"(2) Technical Specifications

The Technical Specifications contained in Appendix A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 3."

3. This license amendment is effective as of 6 PM of the date of its issuance.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed by
R. C. DeYoung

**R. C. DeYoung, Assistant Director
for Light Water Reactors, Group 1
Directorate of Licensing**

Date of Issuance: November 16, 1974

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**ATTACHMENT TO LICENSE AMENDMENT NO. 4
CHANGE NO. 3 TO THE TECHNICAL SPECIFICATIONS
FACILITY OPERATING LICENSE NO. DPR-53
BALTIMORE GAS AND ELECTRIC COMPANY
CALVERT CLIFFS UNIT 1
DOCKET NO. 50-317**

SECTION 7

7.0 INTERIM SPECIAL TECHNICAL SPECIFICATIONS

Technical Specifications 3.8.D, 3.8.A.6, 4.1, 4.3.C, 4.2, 4.5.B.5, 4.5.C.5, 4.5.F.4, 4.7-1.A.3, 4.7-1.B.1.c, 4.8, 4.9.B, 4.13.A, and 6.2.2 are temporarily waived in their entirety for the period from 6 PM on November 16, 1974 until the reactor pressure vessel head is reinstalled. At such time Section 7 of Appendix A shall be deleted from License No. DPR-53 and the Technical Specifications listed above as temporarily waived shall be in effect. During the period from 6 PM on November 16, 1974 until the reactor pressure vessel head is reinstalled the Technical Specifications in Appendix A, except for those as indicated above to be temporarily waived, shall be supplemented by the temporary Technical Specifications included in this Section 7.

- 7.1 The spent fuel cooling system may be inoperable when the spent fuel storage pool is dry.
- 7.2 Borated water at a concentration of 1720 ppm or greater shall be maintained over the core at a level as high as practicable above the hot and cold leg pipe nozzles. A maximum of 12 fuel assemblies may be removed from the core at any time and stored dry in the spent fuel pool storage racks. The temperature of the dry fuel elements shall be maintained at less than 220°F.
- 7.3 Calibration, testing, and checking of instrument channels, reactor protective system, and engineered safety features, system logic channels, and miscellaneous instrument systems and controls shall be performed as specified in Tables 4.1.1 through 4.1.3 except that those tests and checks that are listed as required to be performed each refueling are waived.

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7.4 Equipment and sampling tests shall be conducted as specified in Tables 4.2.1 and 4.2.2 except that tests and checks noted in items 3, 4, 10, 11, 20 and 21 are not required.

7.5 Airlocks shall be tested at 6-month intervals. However, when containment integrity is required, airlocks which are opened during such intervals shall be tested after each opening or every three days, whichever interval is longer. In any event, the airlocks shall be tested prior to exceeding a cold shutdown condition. For containment penetrations employing a continuous leakage monitoring system, type B tests, except for tests of airlocks, may, notwithstanding the test schedule specified under A.6 above, be performed every other reactor shutdown for refueling but in no case at intervals greater than 3 years.

7.6 Minimum shift manning requirements are those listed in Table 6.2-1 for initial fuel loading.

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UNITED STATES ATOMIC ENERGY COMMISSION

DOCKET NO. 50-317

BALTIMORE GAS AND ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO

FACILITY OPERATING LICENSE

Notice is hereby given that the U. S. Atomic Energy Commission (the Commission) has issued Amendment No. 4 to Facility Operating License No. DPR-53 issued to Baltimore Gas and Electric Company which revised Technical Specifications for operation of the Calvert Cliffs Nuclear Power Plant, Unit 1, located in Calvert County, Maryland. The amendment is effective as of its date of issuance.

The amendment modifies the Specification to delete some of the Appendix A Technical Specification Requirements and to supplement the remainder with Specification needed to assure safe conditions during an inspection program that is to be performed for the purpose of determining the cause of an increase in the pressure drop across the reactor core.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

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For further details with respect to this action, see (1) the application for amendment, dated November 15, 1974, (2) Amendment No. 4 to License No. DPR-53, with any attachments, and (3) the related safety evaluation contained in the Commission's letter to Baltimore Gas and Electric Company. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Calvert County Library, Prince Frederick, Maryland 20678.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Atomic Energy Commission, Washington, D. C. 20545, Attention: Deputy Director for Reactor Projects, Directorate of Licensing - Regulation.

Dated at Bethesda, Maryland, this *18th* day of November 1974.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed by
G. W. Rivenbark

George W. Rivenbark, Acting Chief
Light Water Reactors
Branch 1-3
Directorate of Licensing

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