

DEC 20 1974

Docket No. 50-317

Baltimore Gas & Electric Co.  
ATTN: Mr. A. E. Lundvall, Jr.  
Vice President, Supply  
Gas and Electric Building  
Charles Center  
Baltimore, Maryland 21203

Amendment No. 6  
Change No. 5  
License No. DPR-53

Gentlemen:

By letter dated December 19, 1974, Baltimore Gas and Electric Company (BG&E) requested that its Operating License No. DPR-53 for Calvert Cliffs Unit 1 be amended to modify the Technical Specifications for the present period during which the reactor is shut down for the purpose of recoupling several control element assemblies to their respective extension shafts.

Following a recent plant shutdown to inspect the core internals, several control element assemblies were not properly recoupled when the reactor vessel was reclosed. This condition was detected during test operations that followed the outage and the reactor was shut down and the vessel head was removed to determine the cause for and to couple the uncoupled control element assemblies. Prior to removing the vessel head, the reactor coolant boron concentration was increased to the refueling boron concentration and the coolant temperature was decreased to less than 210°F. This condition is defined in the Technical Specifications as "Refueling Shutdown". The Technical Specifications also define as a "Refueling Operation" any operation involving movement of core components which could affect core reactivity when the vessel head is unbolted or removed.

Most, but not all, of the Technical Specifications that require checks, measurements, and tests during "Refueling Shutdown" do so to assure that the tests are made at refueling intervals of approximately one year rather than (1) each time "any operation involving movement of core components which could effect core reactivity is performed with the vessel head unbolted or removed," or (2) each time "the reactor coolant is at refueling boron concentration and the average coolant temperature is less than 210°F". Temporary deletion of those Technical Specification requirements that were specified for each refueling shutdown solely for the purpose of providing certain tests, checks, and measurements at one to two year intervals will not circumvent the intent of the Specifications as these checks, tests, and measurements will be performed when the actual refueling of the reactor occurs. On this basis, we conclude that temporary deletion of these Specifications does not involve a significant safety hazard.

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consideration. Some Technical Specification requirements were specified for each refueling shutdown because they can only be performed when the plant is shut down or the vessel head is removed. Checks and tests of this type were performed within the past 30 days during the previous plant shutdown to inspect the core internals and these checks and tests will be required to be made when the actual refueling occurs. On this basis we conclude that temporary deletion of these Specifications does not involve a significant safety hazard consideration. A few of the Technical Specification requirements were specified for each refueling shutdown because they specify checks and tests that should be performed each time the reactor vessel head is removed. We will not delete the requirement for these tests.

Accordingly, we will permit Technical Specifications 4.1, 4.2, 4.3.C, 4.5.B.5, 4.5.C.5, 4.5.F.4, 4.6.C, 4.6.D, 4.7-1.A.2, 4.7-1.A.3, 4.7-1.B.1.c, 4.8, 4.9.B, and 4.13.A to be waived until the reactor is next taken out of the refueling shutdown condition. At the same time, we will supplement the remaining Appendix A Technical Specifications with additional temporary Technical Specifications as provided in the enclosure to assure that those specific checks and tests that should be performed each time the reactor vessel head is removed and that are part of the Specifications listed above for temporary deletion are reinstated as specific requirements.

We are hereby amending License No. DPR-53 by including Section 7 to Appendix A which will, for a specified period of time, delete certain Technical Specifications in Appendix A and supplement the remainder with Specifications needed to assure safe conditions during the present shutdown operations. The amended Technical Specifications are presented as Change No. 5 to the Technical Specifications and is attached to the enclosed Amendment No. 6 to the Calvert Cliffs Unit 1 Operating License.

We have also considered your request to change the Technical Specifications so as to define a refueling shutdown as a plant outage during which fuel assemblies or CEAs within the core are replaced or rearranged. You state that such a change will ensure that a situation of the type you are presently involved in will not reoccur. We have concluded that the proposed change is unacceptable. The basic cause for the present situation should not be placed with the Technical Specifications but with the failure of the operations and maintenance personnel to place the plant in proper operational status following your prior shutdown. In addition, we find some of the requirements that you proposed be waived, following review by your Plant Operations and

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Safety Review Committee and the Off-Site Safety and Review Committee, not to be acceptable for such waiver. We conclude that sufficient justification for the change of definition that you propose has not been provided and that the current definitions provide for operating restraints that are consistent with safety and do not unnecessarily interfere with your operational capability. We recognize that current Technical Specifications are not perfect and we have a continuing effort underway to revise and standardize Technical Specifications in order to make them as meaningful and effective as practical with as little impact on operating flexibility as is consistent with safety requirements. We welcome your comments in this respect and will give them due consideration in our future deliberations.

Sincerely,

Original Signed by  
R. C. DeYoung

R. C. DeYoung, Assistant Director  
for Light Water Reactors Group 1  
Directorate of Licensing

Enclosures:

1. Amendment 6 to DPR-53
2. Federal Register Notice

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bcc:  
JBuchanan, ORNL  
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Mr. Warren D. Hodges, Director  
Department of State Planning  
301 West Preston Street  
Baltimore, Maryland 21201

Mr. Bernard Fowler, President  
Board of County Commissioners  
Prince Frederick, Maryland 20678

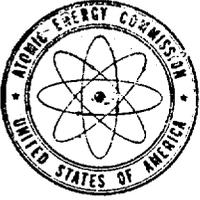
Mr. Robert J. Blanco, Region III  
Environmental Protection Agency  
Sixth and Walnut Streets  
Philadelphia, Pennsylvania 19106

Mr. Bruce Blanchard  
Environmental Projects Review  
Department of the Interior  
Room 5321  
18th and C Streets, N. W.  
Washington, D. C. 20240

Mr. Sheldon Myers  
ATTN: Mr. Jack Anderson  
Office of Federal Activities  
EPA, Room W-541, Waterside Mall  
401 M Street, S. W.  
Washington, D. C. 20460

Colonel Howard Sargent  
Executive Director of Civil Works  
Office of the Chief of Engineers  
Corps of Engineers  
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Washington, D. C. 20314

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UNITED STATES  
ATOMIC ENERGY COMMISSION  
WASHINGTON, D.C. 20545

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 6  
License No. DPR-53

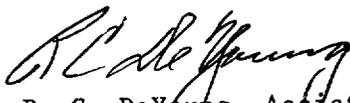
1. The Atomic Energy Commission (the Commission) having found that:
  - A. The application for amendment by Baltimore Gas and Electric Company (the licensee) dated December 9, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended, and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of the Facility License No. DPR-53 is hereby amended to read as follows:

"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 5."

3. This license amendment is effective as of the date of its issuance.

FOR THE ATOMIC ENERGY COMMISSION



R. C. DeYoung, Assistant Director  
for Light Water Reactors Group 1  
Directorate of Licensing

Attachment:  
Change No. 5 to Appendix A  
Technical Specifications

Date of Issuance:

ATTACHMENT TO LICENSE AMENDMENT NO. 6  
CHANGE NO. 5 TO THE TECHNICAL SPECIFICATIONS  
FACILITY OPERATING LICENSE NO. DPR-53  
BALTIMORE GAS AND ELECTRIC COMPANY  
CALVERT CLIFFS UNIT 1  
DOCKET NO. 50-317

SECTION 7

7.0 INTERIM SPECIAL TECHNICAL SPECIFICATIONS

Technical Specifications 4.1, 4.2, 4.3.C, 4.5.B.5, 4.5.C.5, 4.5.F.4, 4.6.C, 4.6.D, 4.7-1.A.2, 4.7-1.A.3, 4.7-1.B.1.c, 4.8, 4.9.B, and 4.13.A are temporarily waived in their entirety until the reactor is next taken out of the refueling shutdown condition. At such time Section 7 of Appendix A shall be deleted from License No. DPR-53 and the Technical Specifications listed above as temporarily waived shall be in effect. During the period until the reactor is next taken out of the refueling shutdown condition, the Technical Specifications in Appendix A, except for those as indicated above to be temporarily waived, shall be supplemented by the temporary Technical Specifications included in this Section 7.

7.1 Calibration, testing, and checking of instrument channels, reactor protective system and engineered safety features system logic channels, and miscellaneous instrument systems and controls shall be performed as specified in Tables 4.1.1 through 4.1.3 except that, with the exception of items 1, 2, and 6 of Table 4.1.3, those tests and checks that are listed as required to be performed each refueling are waived.

7.2 Equipment and sampling tests shall be conducted as specified in Tables 4.2.1 and 4.2.2 except that tests and checks noted in items 3, 4, 10, 16, 20, 21, 24, and 25 are not required.

- 7.3 Airlocks shall be tested at 6-month intervals. However, when containment integrity is required, airlocks which are opened during such intervals shall be tested after each opening or every three days, whichever interval is longer. In any event, the airlocks shall be tested prior to exceeding a cold shut-down condition.
- 7.4 All reactor vessel closure studs and nuts shall be visually inspected for thread damage.

UNITED STATES ATOMIC ENERGY COMMISSION

DOCKET NO. 50-317

BALTIMORE GAS AND ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO

FACILITY OPERATING LICENSE

Notice is hereby given that the U. S. Atomic Energy Commission (the Commission) has issued Amendment No. 6 to Facility Operating License No. DPR-53 issued to Baltimore Gas and Electric Company which revised Technical Specifications for operation of the Calvert Cliffs Nuclear Power Plant, Unit 1, located in Calvert County, Maryland. The amendment is effective as of its date of issuance.

The amendment modifies the Specifications to delete some of the Appendix A Technical Specifications Requirements and to supplement the remainder with Specification needed to assure safe conditions during reactor shutdown operations to inspect and recouple several control element assemblies that were not recoupled to their respective extension shafts following a recent core inspection.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

For further details with respect to this action, see (1) the application for amendment, dated December 19, 1974, (2) Amendment No. 6 to License No. DPR-53, with any attachments, and (3) the related safety evaluation contained in the Commission's letter to Baltimore Gas and Electric Company. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Calvert County Library, Prince Frederick, Maryland 20678.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Atomic Energy Commission, Washington, D. C. 20545, Attention: Deputy Director for Reactor Projects, Directorate of Licensing - Regulation.

Dated at Bethesda, Maryland, this 20 day of December 1974.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed By

O. D. Parr

Olan D. Parr, Chief  
Light Water Reactors  
Project Branch 1-3  
Directorate of Licensing