

9/30/77

Docket Nos. 50-317  
50-318

Baltimore Gas & Electric Company  
ATTN: Mr. A. D. Lundvall, Jr.  
Vice President - Supply  
Post Office Box 1475  
Baltimore, Maryland 21203

Gentlemen:

The Commission has issued the enclosed Amendment Nos. 25 and 8 to Facility Operating License Nos. DPR-53 and DPR-69 for the Calvert Cliffs Nuclear Power Plant Unit Nos. 1 and 2, respectively. The amendments are in response to that portion of your April 26, 1977 application relating to removal of snubbers associated with the containment spray rings, and your supplements dated July 29, 1977, and September 8 and 14, 1977.

The amendments change the Technical Specifications to delete snubbers associated with the containment spray rings.

That portion of your April 26, 1977 application dealing with a change to the footnote following Table 3.7-4 will be considered by us at a later date.

Copies of the related Safety Evaluation and the Notice of Issuance also are enclosed.

Sincerely,

*Don K. Davis*  
Don K. Davis, Acting Chief  
Operating Reactors Branch #2  
Division of Operating Reactors

Enclosures:

1. Amendment No. 25 to License No. DPR-53
2. Amendment No. 8 to License No. DPR-69
3. Safety Evaluation
4. Notice

010400

CB 9/22  
LB

9/30/77  
Called L&E (PKello) 1520, tried 1364E numbers for 40 minutes - no contact. 82E

0755 10-3-77  
Notified C/6 (Olson) that amendment was signed on 9-30-77. Remove second sentence of Discussion & Evaluation on Safety Evaluation and add first sentence.

DOR:ORB #2  
DK Davis  
9/30/77  
Const. 1

OFFICE	DOR:ORB #2	DOR:ORB #2	DOR:PSB/OT	DOR:EB/OT	STS	OELD
SURNAME	RMDiggs	MConner	WButler	LShao	JMcGough	Obustead
DATE	9/23/77	9/23/77	9/26/77	12/1	9/23/77	9/29/77

September 30, 1977

cc w/enclosures:

James A. Biddison, Jr.  
General Counsel  
Gas and Electric Building  
Charles Center  
Baltimore, Maryland 21203

Dr. Steven Long  
Power Plant Siting Program  
Department of Natural Resources  
B-3, Tawes  
State Office Building  
Annapolis, Maryland 21401

George F. Trowbridge, Esquire  
Shaw, Pittman, Potts and  
Trowbridge  
1800 M Street, N. W.  
Washington, D. C. 20036

Bechtel Power Corporation  
ATTN: Mr. R. L. Ashley  
Chief Nuclear Engineer  
P. O. Box 607  
Gaithersburg, Maryland 20760

Combustion Engineering, Inc.  
ATTN: Mr. J. A. Honey  
Project Manager  
P. O. Box 500  
Windsor, Connecticut 06095

Mr. R. C. L. Olson  
Baltimore Gas and Electric Company  
Room 922 Gas and Electric Building  
Post Office Box 1475  
Baltimore, Maryland 21203

Mr. R. M. Douglass, Chief Engineer  
Calvert Cliffs Nuclear Power Plant  
Baltimore Gas and Electric Company  
Lusby, Maryland 20657

Calvert County Library  
Prince Frederick, Maryland 20678

Chief, Energy Systems  
Analyses Branch  
Office of Radiation Programs  
U. S. Environmental Protection Agency  
Room 645, East Tower  
401 M Street, N. W.  
Washington, D. C. 20460

U. S. Environmental Protection Agency  
Region III Office  
ATTN: EIS COORDINATOR  
Curtis Building (Sixth Floor)  
Philadelphia, Pennsylvania 19106

Mr. Bernard Fowler  
President, Board of County  
Commissioners  
Prince Frederick, Maryland 20678

cc w/enclosures and cy. of BG&E  
filings dtd. 4/26/77, 7/29/77,  
9/8 & 14/77:

Dr. Paul Massicot, Director  
Department of Natural Resources  
Power Plant Siting Program  
Energy and Coastal Zone Admin.  
Tawes State Office Building  
Annapolis, Maryland 21401



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

BALTIMORE GAS & ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 25  
License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated April 26, 1977, as it relates to removal of snubbers associated with the containment spray rings, as supplemented by letters dated July 29, 1977, and September 8 and 14, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

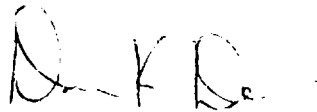
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility License No. DPR-53 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.25, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Don K. Davis, Acting Chief  
Operating Reactors Branch #2  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 30, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 25

FACILITY OPERATING LICENSE NO. DPR-53

DOCKET NO. 50-317

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 7-49

3/4 7-50

3/4 7-51

TABLE 3.7-4  
SAFETY RELATED HYDRAULIC SNUBBERS\*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-60-22	SPRAY TO CONTAINMENT CHARCOAL FILTER #13 66'	I	Yes	No
1-60-23	SPRAY TO CONTAINMENT CHARCOAL FILTER #13 67'	I	Yes	No
1-60-24	SPRAY TO CONTAINMENT CHARCOAL FILTER #13 66'	I	Yes	No
1-60-25	SPRAY TO CONTAINMENT CHARCOAL FILTER #11 63'	I	Yes	No
1-60-26	SPRAY TO CONTAINMENT CHARCOAL FILTER #11 66'	I	Yes	No
1-60-27	SPRAY TO CONTAINMENT CHARCOAL FILTER #11 66'	I	Yes	No
1-60-28	SPRAY TO CONTAINMENT CHARCOAL FILTER #11 66'	I	Yes	No
1-61-1	CONTAINMENT SPRAY PUMP #12 DISCHARGE -15'	A	No	No

TABLE 3.7-4  
SAFETY RELATED HYDRAULIC SNUBBERS\*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-61-14	CONTAINMENT SPRAY D/STRM S/D H/X 5'	A	No	No

TABLE 3.7-4  
SAFETY RELATED HYDRAULIC SNUBBERS\*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-61-17	CONTAINMENT SPRAY D/STRM S/D H/X -15'	A	No	No
1-63-9	S.G. #11 BLOWDOWN ORIFICE LINE 70'	I	Yes	No
1-63-10	S.G. #11 BLOWDOWN ORIFICE BYPASS 78'	I	Yes	No
1-63-11	NITROGEN to S.G. #12 74'	I	Yes	No
1-63-12	NITROGEN to S.G. #12 69'	I	Yes	No
1-63-13	STEAM GENERATORS 75'	I	Yes	No
1-63-14	STEAM GENERATORS 75'	I	Yes	No
1-63-15	STEAM GENERATORS 75'	I	Yes	No
1-63-16	STEAM GENERATORS 75'	I	Yes	No
1-63-17	STEAM GENERATORS 75'	I	Yes	No



TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUEBERS\*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
1-63-18	STEAM GENERATORS 75'	I	Yes	No
1-63-19	STEAM GENERATORS 75'	I	Yes	No
1-63-20	STEAM GENERATORS 75'	I	Yes	No
1-63-21	STEAM GENERATORS 75'	I	Yes	No
1-63-22	STEAM GENERATORS 75'	I	Yes	No
1-63-23	STEAM GENERATORS 75'	I	Yes	No
1-63-24	STEAM GENERATORS 75'	I	Yes	No
1-63-25	STEAM GENERATORS 75'	I	Yes	No
1-63-26	STEAM GENERATORS 75'	I	Yes	No
1-63-27	STEAM GENERATORS 75'	I	Yes	No
1-63-28	STEAM GENERATORS 75'	I	Yes	No
1-64-1	LINE TO PRESS. RELIEF MOV-403 81'	I	Yes	No



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

BALTIMORE GAS & ELECTRIC COMPANY

DOCKET NO. 50-318

CALVERT CLIFFS UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 8  
License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated April 26, 1977, as it relates to removal of snubbers associated with the containment spray rings, as supplemented by letters dated July 29, 1977, and September 8 and 14, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility License No. DPR-69 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.8, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Don K. Davis, Acting Chief  
Operating Reactors Branch #2  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 30, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 8

FACILITY OPERATING LICENSE NO. DPR-69

DOCKET NO. 50-318

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 7-43

3/4 7-44

3/4 7-45

CALVERT CLIFFS-UNIT 2

3/4 7-43

Amendment No. 8

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS\*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
2-60-28	SPRAY TO CONT. CHARCOAL FILTER #22 77'	I	Yes	No
2-60-29	SPRAY TO CONT. CHARCOAL FILTER #22 63'	I	Yes	No
2-60-30	SPRAY TO CONT. CHARCOAL FILTER #21 60'	I	Yes	No
2-61-1	HPSI PUMP #23 SUCT. FROM S/D COOLING H.X. #22 9'9"	A	No	No
2-61-2	HPSI PUMP #23 SUCT. FROM S/D COOLING H.X. #22 9'9"	A	No	No
2-61-3	CONT. SPRAY PUMP #22 DISCH 3'6"	A	No	No
2-61-4	CONT. SPRAY HDR PENETRATION PIPING 10'0"	A	No	No
2-61-5	CONT. SPRAY HDR DOWNSTREAM S/D COOL H.X. #22 17'5"	A	No	No
2-61-5A	CONT. SPRAY HDR DOWNSTREAM S/D COOL H.X. #22 17'5"	A	No	No

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS\*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
2-61-16	CONT. SPRAY HDR FOR SPRAY RING #22 70'	I	Yes	No
2-61-17	CONT. SPRAY HDR FOR SPRAY RING #22 46'	I	Yes	Yes
2-61-18	CONT. SPRAY HDR FOR SPRAY RING #22 39'	I	Yes	Yes

CALVERT CLIFFS-UNIT 2

3/4 7-45

Amendment No. 8

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS\*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
2-61-19	CONT. SPRAY HDR FOR SPRAY RING #22 39'	I	Yes	Yes
2-63-3	NITROGEN LINE TO S/G #22 77'6"	I	Yes	No
2-63-4	NITROGEN LINE TO S/G #22 77'6"	I	Yes	No
2-63-5	S/G #21 SURFACE BLOWDOWN LINE 76'9"	I	Yes	No
2-63-6	S/G #21 SURFACE BLOWDOWN LINE 76'9"	I	Yes	No
2-63-11	STEAM GENERATOR #21 75'	I	Yes	Yes
2-63-12	STEAM GENERATOR #21 75'	I	Yes	Yes
2-63-13	STEAM GENERATOR #21 75'	I	Yes	Yes
2-63-14	STEAM GENERATOR #21 75'	I	Yes	Yes
2-63-15	STEAM GENERATOR #21 75'	I	Yes	Yes
2-63-16	STEAM GENERATOR #21 75'	I	Yes	Yes
2-63-17	STEAM GENERATOR #21 75'	I	Yes	Yes

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS\*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
2-63-18	STEAM GENERATOR #21 75'	I	Yes	Yes
2-63-19	STEAM GENERATOR #22 75'	I	Yes	Yes
2-63-20	STEAM GENERATOR #22 75'	I	Yes	Yes
2-63-21	STEAM GDNERATOR #22 75'	I	Yes	Yes
2-63-22	STEAM GENERATOR #22 75'	I	Yes	Yes
2-63-23	STEAM GENERATOR #22 75'	I	Yes	Yes
2-63-24	STEAM GENERATOR #22 75'	I	Yes	Yes
2-63-25	STEAM GENERATOR #22 75'	I	Yes	Yes
2-63-26	STEAM GENERATOR #22 75'	I	Yes	Yes
2-64-1	PRESSURIZER REL PIPING UPSTREAM MOV 403 81'6"	I	Yes	No
2-64-2	PRESSURIZER REL PIPING TO RV 200 79'11"	I	Yes	No
2-64-3	PRESSURIZER REL PIPING DOWNSTREAM MOV 405 84'3"	I	Yes	No

CALVERT CLIFFS-UNIT 2

3/4 7-46





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 25 TO LICENSE NO. DPR-53 AND

AMENDMENT NO. 8 TO LICENSE NO. DPR-69

BALTIMORE GAS AND ELECTRIC COMPANY

CALVERT CLIFFS NUCLEAR POWER PLANT UNITS NO. 1 AND NO. 2

DOCKET NOS. 50-317 AND 50-318

INTRODUCTION

By application for license amendment dated April 26, 1977, and supplements thereto dated July 29, and September 8 and 14, 1977, Baltimore Gas & Electric Company (BG&E) requested changes to the Technical Specifications for Calvert Cliffs Unit Nos. 1 and 2. The proposed changes to the Technical Specifications consist of revising Table 3.7-4 to permit deletion of certain hydraulic shock suppressors (snubbers) listed in that Table as part of a complete redesign of the support systems of the containment spray rings on which the snubbers are installed.

BG&E additionally proposed to amend the footnote following Table 3.7-4 to allow deletion of snubbers from safety related systems without license amendment to Table 3.7-4 provided that a stress analysis is performed in accordance with the design code as stated in the FSAR or a later revision of the code to justify the removal of any snubber. This proposed change is not acceptable at this time. However, the snubber issue is under generic review and a final judgment on this proposal will be made at a later time.

DISCUSSION AND EVALUATION

BG&E is in the process of systematically reviewing all snubber installations at Calvert Cliffs. The intention of this review is to eliminate as many snubbers as possible. The reliability of the piping system is increased by reducing the number of hydraulic snubbers necessary to satisfy the design criteria. Hydraulic snubbers have a potential for leakage and miscalibration. This potential makes them inherently less reliable than other types of rigid and mechanical supports. Also a snubber without hydraulic fluid provides essentially no piping support. After reviewing the original design considerations of the Containment Spray System, BG&E determined that it was possible to remove the snubbers on the containment spray header adding fixed restraints where appropriate.

The containment spray header and riser was reanalyzed in accordance with the design code specified in Table 6-6 of the FSAR. The reanalysis was performed by Bechtel Power Corporation and utilized their ME 632 computer code. ME 632 utilizes a linear elastic modeling technique and is capable

of handling 350 degrees of freedom and 999 data points. The code performs thermal, dead weight, and seismic analysis. This reanalysis is very similar to that originally performed at the time of initial installation and ensures that all design criteria are satisfied.

In the reanalysis, the original temperature of 300°F was assumed. This is the maximum anticipated temperature that the containment spray lines will experience during a LOCA. Response spectra from the original reactor building analyses were used as seismic input. In total, all snubbers were removed from the inner and outer spray rings. To accomplish this, rigid restraints were both removed and added as appropriate to remain within the specified stress allowables. The overall combined thermal, dead weight, and seismic stress was computed to be less than that of the original design. The seismic stress component is somewhat higher because of the fewer number of restraints. However, the fewer restraints allow for more thermal growth and the thermal stress component is lower. The containment spray risers and rings were evaluated for the effects of fluid-induced reaction loads on the piping system caused by the accelerating flow. It was found that the modified design provided sufficient restraint to mitigate the effects of such a transient.

The redesign reduces the surveillance burden, improves the overall system reliability and thus reduces the potential of a safety system piping failure due to a malfunction of the support system. Thus the reliability of the containment spray piping system will be increased by reducing the number of hydraulic snubbers necessary to satisfy the design criteria.

#### ENVIRONMENTAL CONSIDERATION

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: September 30, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-317 AND 50-318

BALITMORE GAS AND ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 25 and 8 to Facility Operating License No. DPR-53, and DPR-69 (respectively), issued to Baltimore Gas & Electric Company (the licensee), which revised the licenses and their appended Technical Specifications for operation of the Calvert Cliffs Nuclear Power Plant Unit Nos. 1 and 2 (the facilities) located in Calvert County, Maryland. The amendments are effective as of their date of issuance.

The amendments permitted deletion of certain hydraulic shock suppressors as part of a complete redesign of the support systems of the containment spray rings.

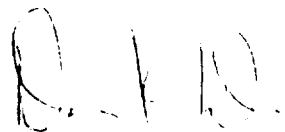
The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendments.

For further details with respect to this action, see (1) the application for amendment dated April 26, 1977, and supplements dated July 29, 1977, and September 8 and 14, 1977, (2) Amendment No. 25 to License No. DPR-53, and Amendment No. 8 to License No. DPR-69, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Calvert County Library, Prince Frederick, Maryland 20678. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 30th day of September, 1977.

FOR THE NUCLEAR REGULATORY COMMISSION



Don K. Davis, Acting Chief  
Operating Reactors Branch #2  
Division of Operating Reactors