DISTRIBUTION

Docket No. 50-317

NOV 04 1975

Baltimore Gas and Electric Company
ATTN: Mr. A. E. Lundvall, Jr.
Vice President - Supply
Gas & Electric Building
Charles Center
Baltimore, Maryland 21203

Gentlemen:

	ORB #2 Reading	PCollins
	OELD	SVarga
	OI&E (3)	CHebron
	NDube	AESteen
	BJones (4)	DEisenhut
	JMcGough	ACRS (16)
	JSaltzmann, OAI	TBAbernathy, DTIE
	RMDiggs	HRood
	EAReeves	
	DLZiemann	
	KRGoller	
	SKari	
	BScharf (15)	

The Commission has issued the enclosed Amendment No. 10 to Facility Operating License No. DPR-53 for the Calvert Cliffs Nuclear Power Plant Unit 1. This amendment includes Change No. 9 to the Technical Specifications, Appendix A, and is in response to your request dated November 27, 1974.

The amendment incorporates into the Calvert Cliffs Nuclear Power Plant Unit 1 Technical Specifications changes to the reporting requirements. Changes to your proposal were necessary to meet our requirements. These have been discussed with your staff. The Technical Specifications are based on Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4.

We request that you use the formats presented in the Appendices to Regulatory Guide 1.16, Revision 4, for reporting operating information and that you report events of the type described under the section "Events of Potential Public Interest". Instructions for using these reporting formats are contained in Regulatory Guide 1.16 (a copy is enclosed for your use), and AEC report COO-SS-001 titled "Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File" of which you were previously provided a copy. This report is modified by updated instructions dated August 21, 1975 which are enclosed. Copy requirements are summarized in Regulatory Guide 10.1, "Compilation of Reporting Requirements for Persons Subject to AEC Regulations", a copy of which is also enclosed. This Guide will assist you in identifying reports that are required by the Commission's regulations set forth in Title 10 Code of Federal Regulations but are not contained in your Technical Specifications. Reports that are required by the regulations have not been repeated in your Technical Specifications.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

Original Signed by:
Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Reactor Licensing

Not needed — See attached 11/3/75 by S. Galler

OFFICE ▶	RL:ORB #2	RL:ORB #2			OELD	RL:ORB #2
SURNAME ▶	RMDiggs	EAReeves:ah				DLZiemann
DATE ▶	11/3/75	11/3/75			10/ /75	11/4/75

Enclosures: See next page

Form AEC-318 (Rev. 9-53) AECM 0240 ☆ U.S. GOVERNMENT PRINTING OFFICE: 1974 — 555-338

Enclosures:
1. Amendment No. 10
2. Regulatory Guide 1.16
3. Updated Instructions
4. Regulatory Guide 10.1
5. Safety Evaluation
6. Federal Register Notice

cc w/enclosures:
Mr. James A. Biddison, Jr.
General Counsel
Gas and Electric Building
Charles Center
Baltimore, Maryland 21203

James C. Cawood, Jr., Esquire
Vice President
Chesapeake Environmental
Protection Association
4700 Auth Place
Camp Springs, Maryland 20023

George F. Trowbridge, Esquire
Shaw, Pittman, Potts and
Trowbridge
910 17th Street, N. W.
Washington, D. C. 20006

Bechtel Power Corporation
ATTN: Mr. R. L. Ashley
Chief Nuclear Engineer
P. O. Box 607
Gaithersburg, Maryland 20760

Combustion Engineering, Inc.
ATTN: Mr. J. A. Honey
Project Manager
P. O. Box 500
Windsor, Connecticut 06095

Calvert County Library
Prince Frederick, Maryland 20678½

Mr. Bernard Fowler
President, Board of County
Commissioners
Prince Frederick, Maryland 20678

Mr. Warren D. Hodges, Director
Department of State Planning
301 West Preston Street
Baltimore, Maryland 21201
(w/4 cys of encls. to this letter
and 1 cy of BG&E's filing dtd.
11/27/74)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 10
License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Baltimore Gas and Electric Company (the licensee) dated November 27, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Facility License No. DPR-53 is hereby amended to read as follows:

"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 9."

3. This license amendment is effective 30 days after its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by:
Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Reactor Licensing

Attachment:
Change No. 9 to the
Technical Specifications

Date of Issuance: NOV 04 1975

OFFICE ➤						
SURNAME ➤						
DATE ➤						

ATTACHMENT TO LICENSE AMENDMENT NO. 10

CHANGE NO. 9 TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-53

DOCKET NO. 50-317

The Appendix A portion of the Technical Specifications are hereby revised as indicated below:

1. Delete pages iii, 1.4-1, 1.4-2, 6.5-2, 6.5-6, 6.5-8 and 6.6-1 and substitute revised pages of the same numbers, except that there is no revised pages 1.4-2. Changed areas on the revised pages are reflected by marginal lines and are identified with Change No. 9.

2. Delete pages 6.12-1 through 6.12-8a and substitute new pages 6.12-1 through 6.12-7.

NOTE: The revised pages are printed on one side only. Therefore, the existing page in the Technical Specifications should not be destroyed if the reverse side contains an unrevised page.

ATTACHMENT TO LICENSE AMENDMENT NO. 10

CHANGE NO. 9 TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-53

DOCKET NO. 50-317

The Appendix A portion of the Technical Specifications are hereby revised as indicated below:

1. Delete pages iii, 1.4-1, 1.4-2, 6.5-2, 6.5-6, 6.5-8 and 6.6-1 and substitute revised pages of the same numbers, except that there is no revised pages 1.4-2. Changed areas on the revised pages are reflected by marginal lines and are identified with Change No. 9.

2. Delete pages 6.12-1 through 6.12-8a and substitute new pages 6.12-1 through 6.12-7.

NOTE: The revised pages are printed on one side only. Therefore, the existing page in the Technical Specifications should not be destroyed if the reverse side contains an unrevised page.

TABLE OF CONTENTS (Cont'd)

1.4 MISCELLANEOUS DEFINITIONS

Operable

A system or component is operable if it is capable of fulfilling its design functions.

Operating

A system or component is operating if it is performing its functions.

Control Element Assemblies (CEAs)

All full-length shutdown and regulating control rods.

Containment Integrity

Containment integrity is defined to exist when all of the following are true:

a. All nonautomatic containment isolation valves and blind flanges which are not required to be open during accident conditions are closed.
b. The equipment hatch is properly closed and sealed.
c. At least one door in each personnel air lock is properly closed and sealed.
d. All automatic containment isolation valves connected to closed systems are operable or are locked closed. All automatic isolation valves that join the containment atmosphere with the atmosphere outside the containment are closed.
e. The uncontrolled containment leakage satisfies Specification 4.5.

9

other proposed procedures and changes thereto as determined by the Chief Engineer to affect nuclear safety.

2. Review proposed changes to equipment or systems that affect nuclear safety.

3. Investigate all reported or suspected instances of violations of Technical Specifications. Prepare and foreward a report covering evaluation and recommendations to prevent recurrence to the Chief Engineer and to the chairman of the OSSRC.

4. Review proposed changes to the operating license, including Technical Specifications.

5. Review events pursuant to 6.6.3 of these Technical Specifications. 9

6. Review all proposed tests and experiments.

7. Perform special reviews and investigations and render reports thereon as requested by the Chairman of the Off-Site Safety and Review Committee.

8. Review the procedures for the handling and shipment of nuclear fuel.

9. Review significant operating abnormalities or deviations from normal and expected performance of plant equipment.

10. Review any reported indication that there may be an unanticipated deficiency in some aspect of design or operation of nuclear safety-related structures, systems, or components.

11. Review the site emergency plan and implementing procedures.

12. Review the site security plan and implementing procedures.

shall be members of the plant staff.

e. Responsibilities

1. Review proposed changes to the operating license, including Technical Specifications.

2. Review minutes of meetings of the Plant Operations and Safety Review Committee to determine if matters considered by that Committee involve unreviewed or unresolved safety questions.

3. Review Safety Evaluations for changes to equipment, systems, and tests and expirements which have been implemented under the provisions of Section 50.59, 10CFR to verify that such actions did not involve "an unreviewed safety question" or which are referred by the POSRC.

4. Review the Environmental Monitoring Program and its results.

5. Investigate all reported instances of violations of Technical Specifications, applicable statutes, codes, regulations, orders, or license requirements; or internal procedures or instructions having nuclear safety significance.

6. Review proposed tests and experiments, and results thereof, and propsed changes to equipment, systems or procedures when such items may constitute an unreviewed safety question as defined in 10 CFR 50.59 or which are referred by the POSRC.

7. Review all events which are required by regulations or Technical Specifications to be reported to the NRC in writing within 24 hours. | 9

8. Review significant operating abnormalities or deviations from normal and expected performance of plant equipment.

9. Review any reported indication that there may be an unanticipated deficiency in some aspect of design or operation of nuclear safety-related structures, systems, or components.

10. Review all reports made to the NRC and NRC responses.

11. Review the site emergency plan and implementing procedures.

12. Review the site security plan and implementing procedures.

Audits shall be performed in accordance with appropriate written instructions or procedures and should include verification of compliance with internal rules, procedures (for example, operating, abnormal, emergency, maintenance, surveillance, test, and radiation control procedures and the emergency plan), regulations, and license provisions; training, qualification, and performance of operating staff; and corrective actions following events which are required by regulations or Technical Specifications to be reported to the NRC in writing within 24 hours. A representative portion of procedures and records of the activities performed during the audit period shall be audited, and in addition, observations of performance of operating and maintenance activities shall be included. Those performing the audits may be members of the audited organization; however, while performing the audit, they shall be responsible to the OSSRC. Written reports of such audits shall be reviewed at a scheduled meeting of the OSSRC and by appropriate members of management, including those having responsibility in the area audited. Follow-up action, including reaudit of deficient areas, shall be taken when indicated. 9

Periodic review of the audit program shall be performed by the OSSRC at least semiannually to assure that such audits are being accomplished in accordance with requirements of technical specifications.

h. Records

Minutes of all meetings of this Committee shall be recorded. Copies of the minutes shall be forwarded to the Vice President - Supply, Committee members, Calvert Cliffs Chief Engineer and such others as the Chairman may designate.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 All events which are required by regulations or Technical Specifications to be reported to the NRC in writing within 24 hours shall be reported immediately to the Chief Engineer who in turn shall report to the Chairman of the Off-Site Safety and Review Committee and the Manager - Electric Production Department.

6.6.2 Initial notification to the NRC of such events shall normally be made by the Chief Engineer in accordance with 6.7 or 6.12.2. Followup written reports shall be made by the Vice President - Supply.

6.6.3 The Plant Operations and Safety Review Committee shall promptly review all events which are required by regulations or Technical Specifications to be reported to the NRC in writing within 24 hours.

6.6.4 Copies of all such reports shall be submitted to the Manager - Electric Production Department and the Chairman of the Off-Site Safety and Review Committee for review. 9

6.12 Reporting Requirements

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the Director of the appropriate Regional Office of Inspection and Enforcement unless otherwise noted.

6.12.1 Routine Reports

a. Startup Report. A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report. 9

Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

b. Annual Operating Report.[1/] Routine operating reports covering the operation of the unit during the previous calendar year should be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

The annual operating reports made by licensees shall provide a comprehensive summary of the operating experience gained during the year, even though some repetition of

1/ A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

previously reported information may be involved. References in the annual operating report to previously submitted reports shall be clear.

Each annual operating report shall include:

(1) A narrative summary of operating experience during the report period relating to safe operation of the facility, including safety-related maintenance not covered in 6.12.1.b.(2)(e) below.

(2) For each outage or forced reduction in power[2/] of over twenty percent of design power level where the reduction extends for greater than four hours:

(a) the proximate cause and the system and major component involved (if the outage or forced reduction in power involved equipment malfunction);

(b) a brief discussion of (or reference to reports of) any reportable occurrences pertaining to the outage or power reduction;

(c) corrective action taken to reduce the probability of recurrence, if appropriate;

(d) operating time lost as a result of the outage or power reduction (for scheduled or forced outages,[3/] use the generator off-line hours; for forced reductions in power, use the approximate duration of operation at reduced power);

(e) a description of major safety-related corrective maintenance performed during the outage or power reduction, including the system and component involved and identification of the critical path activity dictating the length of the outage or power reduction; and

9

2/ The term "forced reduction in power" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the load on the unit be reduced for corrective action immediately or up to and including the very next weekend. Note that routine preventive maintenance, surveillance and calibration activities requiring power reductions are not covered by this section.

3/ The term "forced outage" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the unit be removed from service for corrective action immediately or up to and including the very next weekend.

(f) a report of any single release of radioactivity or radiation exposure specifically associated with the outage which accounts for more than 10% of the allowable annual values.

(3) A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,[4/] e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

(4) Indications of failed fuel resulting from irradiated fuel examinations, including eddy current tests, ultrasonic tests, or visual examinations completed during the report period. 9

c. Monthly Operating Report. Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Office of Inspection and Enforcement, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the appropriate Regional Office, to arrive no later than the tenth of each month following the calendar month covered by the report.

6.12.2 Reportable Occurrences

Reportable occurrences, including corrective actions and measures to prevent reoccurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

4/ This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

a. Prompt Notification With Written Followup. The types of events listed below shall be reported as expeditiously as possible, but within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the appropriate Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

(1) Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.

Note: Instrument drift discovered as a result of testing need not be reported under this item but may be reportable under 6.12.2.a(5), (6) or 6.12.2.b(1) below. 9

(2) Operation of the unit or affected systems when any parameter or operation subject to a limiting condition is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.

Note: If specified action is taken when a system is found to be operating between the most conservative and the least conservative aspects of a limiting condition for operation listed in the technical specifications, the limiting condition for operation is not considered to have been violated and need not be reported under this item, but it may be reportable under 6.12.2.b(2) below.

(3) Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

Note: Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

(4) Reactivity anomalies, involving disagreement with the predicted value of reactivity balance under steady state conditions during power operation, greater than or equal to 1% Δk/k; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if sub-critical, an unplanned reactivity insertion of more than 0.5% Δk/k or occurrence of any unplanned criticality.

(5) Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.

(6) Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.

Note: For 6.12.2.a(5) and (6) reduced redundancy that does not result in a loss of system function need not be reported under this section but may be reportable under 6.12.2.b(2) and (3) below. 9

(7) Conditions arising from natural or man-made events that, as a direct result of the event require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.

(8) Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

(9) Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

Note: This item is intended to provide for reporting of potentially generic problems.

b. Thirty Day Written Reports. The reportable occurrences discussed below shall be the subject of written reports to the Director of the appropriate Regional Office within thirty days of occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

(1) Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

(2) Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

Note: Routine surveillance testing, instrument calibration, or preventative maintenance which require system configurations as described in 6.12.2.b(1) and (2) need not be reported except where test results themselves reveal a degraded mode as described above. 9

(3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.

(4) Abnormal degradation of systems other than those specified in 6.12.2.a(3) above designed to contain radioactive material resulting from the fission process.

Note: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

6.12.3 Unique Reporting Requirements

a. Radioactive Effluent Releases. A report shall be submitted to NRC within 60 days after January 1 and July 1 of each year specifying the quantity of each of the principal radionuclides released to unrestricted areas in liquid and gaseous effluents during the previous 6 months. The format and content of the report shall be in accordance with Regulatory Guide 1.21 Revision 1 dated June 1974. 9

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 10 TO FACILITY LICENSE NO. DPR-53

CHANGE NO. 9 TO THE TECHNICAL SPECIFICATIONS

BALTIMORE GAS AND ELECTRIC COMPANY

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT 1

DOCKET NO. 50-317

Introduction

By letter dated November 27, 1974, Baltimore Gas and Electric Company proposed changes to the Technical Specifications appended to Facility Operating License No. DPR-53, for the Calvert Cliffs Nuclear Power Plant Unit 1. The proposed changes involve changes to the reporting requirements.

Discussion

The proposed changes would be administrative in nature and would affect the conduct of operation. The proposed changes are intended to provide uniform license requirements. Areas covered by the proposed uniform specifications include reporting requirements and an abnormal occurrence definition change.

In Section 208 of the Energy Reorganization Act of 1974 "abnormal occurrence" is defined as an unscheduled incident or event which the Commission determines is significant from the standpoint of public health or safety. The term "abnormal occurrence" is reserved for usage by NRC. Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4, enumerates required reports consistent with Section 208. The proposed change to required reports identifies the reports required of all licensees not already identified by the regulations and those unique to this facility. The proposal would formalize present reporting and would delete any reports no longer needed for assessment of safety related activities.

Evaluation

The new guidance for reporting operating information does not identify any event as an "abnormal occurrence". The proposed reporting requirements also delete reporting of information no longer required and

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DATE ▶						

duplication of reported information. The standardization of required reports and desired format for the information will permit more rapid recognition of potential problems. Similar changes are being approved for all power reactor licensees, so all licensees will have the same requirements presented in a uniform manner.

During our review of the proposed changes, we found that certain modifications to the proposal were necessary to have conformance with the desired regulatory position. These changes were discussed with the licensee's staff and have been incorporated into the proposal.

We have concluded that the proposal as modified improves the licensee's program for evaluating plant performance and the reporting of the operating information needed by the Commission to assess safety related activities and is acceptable. The modified reporting program is consistent with the guidance provided by Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications, Revision 4.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date:

NOV 04 1975

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-317

BALTIMORE GAS AND ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 10 to Facility Operating License No. DPR-53, issued to Baltimore Gas and Electric Company (the licensee), which revised Technical Specifications for operation of the Calvert Cliffs Nuclear Power Plant Unit 1 (the facility) located in Calvert County, Maryland. The amendment becomes effective 30 days after its date of issuance.

The amendment revises the reporting requirements of the Technical Specifications for the facility.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CRR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

For further details with respect to this action, see (1) the application for amendment dated November 27, 1974, (2) Amendment No. 10 to License No. DPR-53, with Change No. 9, and (3) the Commission's

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DATE➤						

related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Calvert County Library, Prince Frederick, Maryland 20678. A single copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 4th day of November, 1975.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by:
Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Reactor Licensing

See note attached.
DLZ

OFFICE	RL:ORB #2	RL:ORB #2	OELD	RL:ORB #2		
SURNAME	RMDiggs	EAReeves		DLZiemann		
DATE	11/3/75	11/3/75	10/ /75	10/4/75		

Form AEC-318 (Rev. 9-53) AECM 0240

☆ U. S. GOVERNMENT PRINTING OFFICE: 1974-526-166

ROUTING AND TRANSMITTAL SLIP		ACTION
1 TO (Name, office symbol or location)	INITIALS	CIRCULATE
OELD - f/concurrences	DATE	COORDINATION
2	INITIALS	FILE
DLZiemann - f/signatures	DATE	INFORMATION
3	INITIALS	NOTE AND RETURN
Reba - for final checks	DATE	PER CONVERSATION
4	INITIALS	SEE ME
	DATE	SIGNATURE

REMARKS

Attached for your concurrence are five packages (Dresden Station, Quad Cities Station, Cooper, Pilgrim and Calvert Cliffs) of nine from ORB 2 which incorporate standard reporting requirement sections into the Appendix A Technical Specifications. One package, Pilgrim also revises the entire administrative controls section.

It is requested that, in the interest of review consistency, these packages (and the 4 future reporting requirements packages) be assigned to one OELD reviewer.

Questions may be directed to the PM for the particular case or to Mike Fletcher, coordinator for reporting (Exts. 7403, 7450)

11/3/75 No need for OELD concurrence this time. Thus subject folder

Do NOT use this form as a RECORD of approvals, concurrences, disapprovals, clearances, and similar actions

FROM (Name, office symbol or location)	DATE
DLZiemann DLZ	11-3-75
	PHONE 7380

OPTIONAL FORM 41
AUGUST 1967
GSA FPMR (41CFR) 100-11.206

648-16-81594-1 552-103 GPO 5041-101

Save w/50-317 yellow.