

DEC 04 1975

Docket No. 50-317

Baltimore Gas and Electric Company  
ATTN: Mr. A. E. Lundvall, Jr.  
Vice President - Supply  
Gas & Electric Building  
Charles Center  
Baltimore, Maryland 21203

Gentlemen:

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*B. Rusche*  
*E. Case*

This will confirm my telephone conversation on November 28, 1975, with Mr. Lee Russel of Baltimore Gas and Electric Company wherein I granted a limited exemption from the provisions of Appendix J of 10 CFR 50 and the requirements of 4.5.C.5 of the Technical Specifications for Calvert Cliffs Unit 1. The exemption allows plant operation to be continued until the next shutdown when the primary system is cooled down but no later than December 30, 1975, without completing the Type C tests on certain containment isolation valves.

During this period we will continue to assess your request dated November 18, 1975, for a six month extension of the interval for testing Type C containment valves. You should provide additional information to justify any exemption beyond that granted by this letter.

The appropriate modification of your Technical Specifications to reflect this exemption is enclosed as Amendment No. 11 to License No. DPR-53 with Change No. 10 to the Technical Specifications. A copy of the Notice of this amendment which has been submitted to the Federal Register for publication and a copy of our Safety Evaluation are enclosed.

Sincerely,

Original Signed by  
*Ben C. Rusche*

Ben C. Rusche, Director  
Office of Nuclear Reactor Regulation

*0900 12-5-75  
Notified B63 E (P. Olson)  
by telecom of the sign-off -  
He will have courier pick  
up copy today.  
JLB leaves*

Enclosures:

- 1. Amendment No. 11
- 2. Safety Evaluation
- 3. Federal Register Notice

cc w/enclosures:

See next page

*with subst filings as follows: NRC staff in lieu of Commission  
1st para of SER amendment in lieu of description  
Done Reland  
12/2*

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DEC 04 1975

Baltimore Gas and Electric Company

- 2 -

cc w/enclosures:

Mr. James A. Biddison, Jr.  
General Counsel  
Gas and Electric Building  
Charles Center  
Baltimore, Maryland 21203

James C. Cawood, Jr., Esquire  
Vice President  
Chesapeake Environmental  
Protection Association  
4700 Auth Place  
Camp Springs, Maryland 20023

George F. Trowbridge, Esquire  
Shaw, Pittman, Potts and  
Trowbridge  
910 17th Street, N. W.  
Washington, D. C. 20006

Bechtel Power Corporation  
ATTN: Mr. R. L. Ashley  
Chief Nuclear Engineer  
P. O. Box 607  
Gaithersburg, Maryland 20760

Combustion Engineering, Inc.  
ATTN: Mr. J. A. Honey  
Project Manager  
P. O. Box 500  
Windsor, Connecticut 06095

Calvert County Library  
Prince Frederick, Maryland 20678

Mr. Bernard Fowler  
President, Board of County  
Commissioners  
Prince Frederick, Maryland 20678

(w/4 cys of encls. to this letter  
and 1 cy of BG&E's filing dtd.  
11/18/75)

Mr. Warren D. Hodges, Director  
Department of State Planning  
301 West Preston Street  
Baltimore, Maryland 21201

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFFS UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

(EXEMPTION FROM APPENDIX J)

Amendment No. 11  
License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Baltimore Gas and Electric Company (the licensee) dated November 18, 1975, is acceptable as a request for exemption from Section III.D.3 of Appendix J of 10 CFR Part 50 of the Commission's rules and regulations set forth in 10 CFR Chapter I, and complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act);
  - B. Granting the temporary exemption from Appendix J of 10 CFR Part 50 is authorized by law and will not endanger life or property or the common defense and security and is otherwise in the public interest;
  - C. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - D. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
  - E. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Facility License No. DPR-53 is hereby amended to read as follows:

"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 10."

3. This license amendment is effective as of November 30, 1975.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by  
Ben C. Rusche

Ben C. Rusche, Director  
Office of Nuclear Reactor Regulation

Attachment:  
Change No. 10 to the  
Technical Specifications

Date of Issuance:

DEC 04 1975

ATTACHMENT TO LICENSE AMENDMENT NO. 11

CHANGE NO. 10 TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-53

DOCKET NO. 50-317

Replace pages 4.5-9, 4.5-25, 4.5-26 and 4.5-27 of the Appendix A portion of the Technical Specifications with the attached revised pages bearing the same numbers. The changed areas on the pages are shown by a marginal line.

NOTE: The revised pages are printed on one side only. Therefore, the existing page in the Technical Specifications should not be destroyed if the reverse side contains an unrevised page.

## 5. Periodic Retest Schedule

Type C tests shall be performed during each reactor shutdown for refueling, but in no case at intervals greater than 2 years.\*

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### D. Special Testing Requirements

Any major modification, replacement of a component which is part of the containment boundary, or resealing of a seal-welded door, performed after the preoperational leakage rate test, shall be followed by either a Type A, Type B, or Type C test, as applicable, for the area affected by the modification. The measured leakage rate from this test shall be included in the report to the Commission required by E.2, below. The acceptance criterion of A.5.b, B.4, and C.4, as appropriate, shall be met. Minor modifications, replacements, or resealing of seal-welded doors, performed directly prior to the conduct of a scheduled Type A test, do not require a separate test.

### E. Inspection and Reporting of Results

#### 1. Containment Inspection

A general inspection of the accessible interior and exterior surfaces of the containment structures and components shall be performed prior to any Type A test to uncover any evidence of structural deterioration which may affect either the containment structural integrity or leaktightness. If there is evidence of structural deterioration, Type A tests shall not be performed until corrective action is taken in accordance with repair procedures, non-destructive examinations, and tests as specified in the applicable code specified in 10 CFR 50, 55a. Such structural deterioration and corrective actions taken shall be reported as part of the test report submitted in accordance with E.2, below.

\* An exemption granted on November 28, 1975, allows the remaining Type C tests (asterisks on Table 4.5.2) to be performed during the next plant shutdown and cooldown following issuance of this change but no later than December 30, 1975.

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LOCAL LEAK TESTING OF  
CONTAINMENT ISOLATION VALVES

<u>Pene. No.</u>	<u>P&amp;ID (FSAR Figure)</u>	<u>Isolation Valve Identification No.</u>	<u>Location WRT Containment</u>	<u>Type of Valve</u>	<u>Test Type (a)</u>
1A*	M-66 (9-10)	CV-5465	Inside	Globe	3
		CV-5466	Inside	Globe	3
		CV-5467	Inside	Globe	3
		CV-5464	Outside	Globe	1
1B*	M-78 (11-2)	CV-2180	Outside	Globe	3
		CV-2181	Outside	Globe	1
1C*	M-73 (9-3)	CV-506	Inside	Globe	3
		CV-505	Outside	Globe	1
1D	M-463 (9-11A)	Sv-6529	Outside	Globe	1
2A*	M-73 (9-3)	CV-515	Inside	Globe	3
		CV-516	Inside	Globe	1
		7M3-1	Outside	Gate	1
		7M3-1	Outside	Gate	1
2B*	M-73 (6-1)	CV-517	Inside	Globe	5 (b)
		CV-518	Inside	Globe	5 (b)
		CV-519	Inside	Globe	5 (b)
		SP-210M3-2	Inside	Check	1
		210M3-2	Outside	Check	1
7A	M-65 (9-20A)	Blind Flange	Inside	---	2
		19-1	Outside	Gate	1
7B	M-65 (9-20A)	Blind Flange	Inside	---	2
		19-1	Outside	Gate	1
8 *	M-76 (5-10)	MOV-5462	Outside	Gate	1
		MOV-5463	Outside	Gate	1
9 *	M-74 (6-1)	238M3-1	Inside	Check	1
		238M3-2	Outside	Check	1
10 *	M-74 (6-1)	238M3-1	Inside	Check	1
		238M3-2	Outside	Check	1
13	M-65 (9-20)	CV-1412	Inside	Butterfly	4 (c)
		CV-1413	Outside	Butterfly	1
14	M-65 (9-20)	CV-1410	Inside	Butterfly	4 (c)
		CV-1411	Outside	Butterfly	1

## NOTES:

- (a) See Legend at end of table.  
 (b) Penetration in use during accident conditions  
 (c) Accident pressure seats and test pressure unseats these valves  
 \* See asterisk note in Specification 4.5.C.5.

<u>Pene. No.</u>	<u>P&amp;ID (FSAR Figure)</u>	<u>Isolation Valve Identification No.</u>	<u>Location WRT Containment</u>	<u>Type of Valve</u>	<u>Test Type (a)</u>
15	M-98 (11-4)	CV-5291 CV-5292	Inside Outside	Globe Globe	3 1
16*	M-51 (9-6)	CV-3832	Outside	Butterfly	4 (d)
18*	M-51 (9-6)	CV-3833	Outside	Butterfly	4 (d)
19A*	M-53 (9-23)	223-1 MOV-2080	Inside Outside	Check Gate	1 1
19B	M-479 (9-23A)	*19-2 130-1	Inside Outside	Gate Globe	1 1
20A	M-68 (5-10)	223-1 *CV-612 *CV-622 *CV-632 *CV-642	Outside Inside Inside Inside Inside	Check Gate Gate Gate Gate	1 2 2 2 2
20B*	M-68 (5-10)	223-1 223-2	Outside Inside	Check Check	1 1
20C	M-68 (5-10)	223-1 *223-2	Outside Inside	Check Check	1 1
23	M-77 (11-1)	CV-4260	Outside	Globe	1
24*	M-463 (9-11A)	SV-6531	Outside	Globe	1
37	M-479 (9-23A)	*29-1 142-1	Inside Outside	Gate Globe	1 1
38	M-72 (4-1)	CV-5460	Outside	Globe	1
39	M-74 (6-1)	130M3-1 130M3-2	Outside Outside	Globe Globe	5 (d) 1
41*	M-74 (6-1)	MOV-652 MOV-651	Inside Outside	Gate Gate	6 (e) 1
44	M-56 (9-22)	*238-1 238-1 MOV-6200	Inside Outside Outside	Check Check Gate	1 1 1  32
47A	M-463 (9-11A)	SV-6540A SV-6507A	Inside Outside	Globe Globe	1 1
47B	M-463 (9-11A)	SV-6540E SV-6507F	Inside Outside	Globe Globe	1 1
47C	M-463	SV-6540F SV-6507F	Inside Outside	Globe Globe	1 1
47D	M-463 (9-11A)	*SV-6540F SV-6507G	Inside Outside	Globe Globe	1 1

NOTES:

(d) Closed system

(e) Two valves, normally closed, connected to reactor coolant loop

\* See asterisk note in Specification 4.5.C.5.



TABLE 4.5.2 (Cont.)

<u>Pene. No.</u>	<u>P&amp;ID (FSAR Figure)</u>	<u>Isolation Valve Identification No.</u>	<u>Location WRT Containment</u>	<u>Type of Valve</u>	<u>Test Type (a)</u>
48A	M-65 (9-20A)	MOV-6900 MOV-6901	Inside Outside	Gate Gate	6 (f) 1
48B	M-65 (9-20A)	*238-1 MOV-6903	Inside Outside	Check Gate	1 1
49A	M-463 (9-11A)	SV-6540B SV-6507B	Inside Outside	Globe Globe	3 1
49B	M-463 (9-11A)	SV-6504C SV-6507D	Inside Outside	Globe Globe	3 1
49C	M-463 (9-11A)	SV-6540D SV-6507D	Inside Outside	Globe Globe	3 1
50	M-65 (9-20A)	Blind Flange Blind Flange	Inside Outside	--- ---	2 1
59	M-58 (9-7)	29M3-1 29M3-1	Inside Outside	Gate Gate	6 (f) 1
60	M-77 (11-1)	130-1 19-2	Inside Outside	Globe Globe	3 1
61	M-58 (9-7)	76Y-1 293M-1 293M-1 293M-1	Inside Inside Inside Outside	Gate Gate Gate Gate	6 (f) 6 (f) 6 (f) 1
62*	M-71 (5-10)	MOV-6579	Outside	Gate	1
64	M-71 (5-10)	238-1	Outside	Check	1

## NOTES:

- (f) Two valves normally closed.  
\* See asterisk note in Specification 4.5.C.5.

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 11 TO FACILITY LICENSE NO. DPR-53

CHANGE NO. 10 TO THE TECHNICAL SPECIFICATIONS

BALTIMORE GAS AND ELECTRIC COMPANY

CALVERT CLIFFS NUCLEAR POWER PLANT UNIT 1

DOCKET NO. 50-317

INTRODUCTION

By letter dated November 18, 1975, the Baltimore Gas and Electric Company (BG&E) requested an amendment to Facility License No. DPR-53 for the Calvert Cliffs Nuclear Plant Unit 1. The request involves a change to the Technical Specifications which would extend the interval for testing some Type C containment penetration valves from two years to two years and six months. However, we consider a 30-day extension adequate for BG&E to present more information in support of their six-month request or to plan for shutdown, cooldown and testing in an efficient manner. We consider the request acceptable for a limited exemption from the requirements of Section III.D.3 of Appendix J of 10 CFR Part 50. Consequently, it is being considered pursuant to Section 50.12 of 10 CFR Part 50.

DISCUSSION

The Type C tests were initially performed in November 1973 for Calvert Cliffs Unit 1. A portion of the Type C retesting was recently completed, but about 47% of the containment isolation valves are still to be tested. The plant must be shutdown, cooled down and depressurized to perform these tests. The licensee advised during our discussions that the remaining tests would be completed the next time the reactor was shutdown and cooled down.

EVALUATION

The Type C tests of containment isolation valves are performed periodically to assure that a combined leakage rate of all penetrations and valves (Type B and Type C tests) are less than 60% of the total containment allowable leakage. The required interval was selected to coincide with the refueling intervals which are normally not more than two years after the first refueling. Calvert Cliffs' first refueling interval will be about 36 months from the time the initial tests were performed.

BG&E has satisfactorily tested approximately 53% of the containment isolation valves. The remaining valves require plant shutdown, cool-down and depressurization prior to testing. We have reviewed the test results and concluded that the actual measured leakage (10 percent of allowable, including Type B tests) demonstrates only minor valve degradation. Thus, the 90% leakage margin for the remaining valves provides confidence that the technical specification leakage limits would not be exceeded due to valve degradation by extending the two year test interval by 30 days. The extension gives the licensee an opportunity to provide further information supporting the requested six-month extension. The discussion should include such matters as improved reliability, consequences of not testing, fossil fuel conservation, seasonal power demands, and other considerations of public interest. On this basis, the limited exemption allowing deferral of the remaining Type C valve leakage tests is considered acceptable.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) granting the exemption is authorized by law and will not endanger life or property or the common defense and security and is otherwise in the public interest, (2) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, and (3) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner.

Date:

DEC 04 1975

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-317

BALTIMORE GAS AND ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 11 to Facility Operating License No. DPR-53, issued to Baltimore Gas and Electric Company (the licensee), which revised Technical Specifications for operation of the Calvert Cliffs Nuclear Power Plant Unit 1 (the facility) located in Calvert County, Maryland. The amendment was effective as of November 30, 1975.

The amendment incorporates an exemption from the requirements of Section III.D.3 of Appendix J of 10 CFR Part 50. It changes the Technical Specifications for the facility to extend the first retest interval of approximately 47% of the containment isolation valves until the next plant shutdown and cooldown following issuance of this amendment, but no later than December 30, 1975.

The application complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. The Commission also concluded that the granting of the exemption from the requirements of Section III.D.3 of Appendix J for the above-referenced test is authorized by law and will not endanger life or property or the common defense and security and is

otherwise in the public interest. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

For further details with respect to this action, see (1) the application for amendment dated November 18, 1975, (2) the letter from the Director of the Nuclear Reactor Regulation to Baltimore Gas and Electric Company (issued concurrently with this Notice), (3) Amendment No. 11 to License No. DPR-53, with Change No. 10, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Calvert County Library, Prince Frederick, Maryland 20678. A single copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this DEC 04 1975

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by:  
Dennis L. Ziemann

Dennis L. Ziemann, Chief  
Operating Reactors Branch #2  
Division of Reactor Licensing

RL:ORB #2  
RM [handwritten initials]  
[handwritten initials]

OFFICE >	DLZiemann	OEL [handwritten initials]	TR [handwritten initials]	RL:ADDC [handwritten initials]	RL:A/DIR	NRR:D/DIR	NRR:DIR
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Docket No. 50-317

Baltimore Gas and Electric Company  
 ATTN: Mr. A. E. Lundvall, Jr.  
 Vice President - Supply  
 Gas & Electric Building  
 Charles Center  
 Baltimore, Maryland 21203

Gentlemen:

The Commission has issued the enclosed Amendment No. 11 to Facility Operating License No. DPR-55 for the Calvert Cliffs Nuclear Plant, Unit 1. The amendment consists of Change No. 10 in the Technical Specifications in accordance with your application dated November 18, 1975.

This amendment incorporates a temporary change to the Calvert Cliffs Unit 1 Technical Specifications. The change extends the interval for testing of certain containment isolation valves for up to six months.

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

Dennis L. Ziemann, Chief  
 Operating Reactors Branch #2  
 Division of Reactor Licensing

Enclosures:

1. Amendment No. 11
2. Safety Evaluation
3. Federal Register Notice

cc w/enclosures see next page

*No - see note to REK  
 enclosed  
 R 26 Nov 75*

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SURNAME →	<i>RMD</i> RMDiggs	<i>EAR</i> EAREeves:ah		DLZiemann	RTedesco	
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