



Entergy

Entergy Operations, Inc.
1448 S.R. 333
Russellville, AR 72802
Tel 501 858 5000

January 31, 2001

1CAN010103

U. S. Nuclear Regulatory Commission
Document Control Desk
Mail Station OP1-17
Washington, DC 20555

Subject: Arkansas Nuclear One - Unit 1
Docket No. 50-313
License No. DPR-51
Annual Update to the License Renewal Application (TAC No. MA8054)

Gentlemen:

By letter dated January 31, 2000 (1CAN000103), Entergy Operations submitted a License Renewal Application (LRA) for Arkansas Nuclear One, Unit 1 (ANO-1) which provided information required by 10CFR Part 54. During the NRC review of the ANO-1 LRA, Entergy Operations is required by 10CFR54.21(b) to report changes to the current licensing basis that materially affect the content of the ANO-1 LRA, including the Safety Analysis Report (SAR) Supplement.

In accordance with the requirements of 10CFR54.21(b), Entergy Operations has completed a review of facility changes since the submittal of the ANO-1 LRA and has determined that there have been no significant plant modifications that materially affect the contents of the ANO-1 LRA. Examples of the facility changes made since the submittal of the ANO-1 LRA include:

- Addition of relief valves to certain reactor building penetrations to ensure over-pressurization protection of the penetrations
- Addition of valves to the normal makeup pump minimum flow path to address valve reliability problems with the previous valve arrangement
- Addition of valves to the service water return piping from the decay heat pump coolers to allow for maintenance activities

A082

- Replacement of a valve from a main steam line vent with a cap to address leakage and accessibility concerns
- Addition of strainers/structural components to the service water bays

These changes have been incorporated into the site license renewal documentation and drawings. However, the addition or removal of these components does not require a revision to the ANO-1 LRA since these component changes do not change the aging effects identified nor the aging management programs credited in the ANO-1 LRA, including the SAR supplement.

Revisions to the SAR Supplement (Appendix A of the ANO-1 LRA) will be provided in response to open item 3.3-1 of the NRC's safety evaluation report (SER) issued on January 10, 2001 (1CNA010106).

During the license renewal regional inspections held the weeks of December 11, 2000, and January 22, 2001, the following clarifications to the ANO-1 LRA were identified:

- In Table 2.2-1 (page 2-17), Section 2.3.2.8 should not have been referenced for the intermediate cooling water system, however the other section references are correct.
- In Table 3.6-8 (page 3-132), Flamastic should have been listed as one of the penetration sealant materials.
- In Table 3.3-2 (page 3-45), the program/activity credited for addressing the aging effect of cracking and loss of material for the heat exchangers (DH coolers), which are stainless steel in borated water, should have been "primary chemistry monitoring" only. Two other programs were inadvertently listed in this row due to an administrative error.
- In Appendix B, Section 4.10 (page B-69), the frequency of the polar crane inspection is incorrectly stated as "annually." The correct interval is "at refueling intervals" or approximately every 18 months.

Also, 10CFR54.4(a)(1)(iii) was revised to support a revision to 10CFR50.67 allowing the use of an alternate accident source term. Entergy Operations is not currently using an alternate accident source term for ANO-1 as described in 10CFR50.67; therefore, the ANO-1 LRA remains unaffected by this revision to 10CFR Part 54.

In addition, Entergy Operations is in the process of obtaining NRC approval to implement improved technical specifications for ANO-1 consistent with NUREG-1430. Several references to the current ANO-1 Technical Specifications were made in the ANO-1 LRA (Appendix B). For information only, the references to current technical specification sections will correspond to the following when the improved technical specifications are approved.

Current ANO-1 Technical Specifications:

3.1.2
3.1.5
3.1.6
3.3.4.B
4.0.5
4.1.a should be 3.3.3.A (see below)
Table 4.1-2
4.6.1
4.8
4.10
4.13
4.15
4.18
6.8.4

Improved Technical Specifications:

3.4.3
Technical Requirements Manual
3.4.13 and 3.4.15
3.6.6
5.58
3.5.1
3.4.13
3.8.1, 3.8.2, and 5.5.13
3.7.5
3.7.10
3.7.8
Inservice Inspection Program
5.5.9
5.5.16

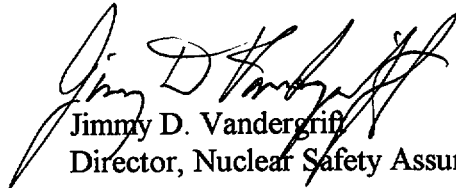
An incorrect reference was provided in Appendix B of the ANO-1 LRA. In Section 4.21.4 of Appendix B (page B-98), the reference to ANO-1 Technical Specification 4.1.a should have been to 3.3.4.A.

Should you have any further questions, please contact me.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on January 31, 2001.

Very truly yours,



Jimmy D. Vandergrieff
Director, Nuclear Safety Assurance

JDV/nbm

cc: Mr. Ellis W. Merschoff
Regional Administrator
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-8064

NRC Senior Resident Inspector
Arkansas Nuclear One
P.O. Box 310
London, AR 72847

Mr. William Reckley
NRR Project Manager Region IV/ANO-1
U. S. Nuclear Regulatory Commission
NRR Mail Stop O-7 D1
One White Flint North
11555 Rockville Pike
Rockville, MD 20852

Mr. Bob Prato
U. S. Nuclear Regulatory Commission
NRR Mail Stop O-12G15
One White Flint North
11555 Rockville Pike
Rockville, MD 20852