

Monticello Nuclear Generating Plant  
2807 West County Road 75  
Monticello, MN 55362-9637

Operated by Nuclear Management  
Company LLC

February 1, 2001

US Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

10 CFR Part 50  
Section 50.90 and 50.91

**MONTICELLO NUCLEAR GENERATING PLANT**  
Docket No. 50-263 License No. DPR-22  
License Amendment Request  
Relocation of ASME Inservice Inspection Requirements to a Licensee Program

Reference 1: NUREG-1433, "Standard Technical Specification, General Electric Plants, BWR/4," Revision 1, 04/07/95.

Reference 2: NMC letter to NRC, "Request for Notice of Enforcement Discretion Monticello Technical Specification 3.15.A.1," February 1, 2001

Attached is a request for change to the Technical Specifications (TS) of the Operating License for the Monticello Nuclear Generating Plant. This request is submitted on an exigent basis in accordance with the provisions of 10 CFR 50.90 and 50.91. The proposed amendment would relocate requirements of the ASME Boiler and Pressure Vessel Code, Section XI Inservice Inspection (ISI) Program currently contained in Technical Specification 3.15.A.1 to a licensee controlled ISI program.

Upon NRC approval of the requested changes, the ISI requirements contained in 10 CFR 50.55a will be defined in the Monticello ISI program. The ISI program requirements do not meet the criteria of 10 CFR 50.36 for inclusion in the Technical Specifications and the Monticello Operating License includes provisions to meet 10 CFR 50, which includes 10 CFR 50.55a for ISI. The proposed license change is also consistent with NUREG-1433 (Reference 1).

We respectfully request this change on an exigent basis as discussed below.

This amendment request is associated with enforcement discretion exercised by the NRC on January 29, 2001 regarding Monticello Technical Specification 3.15.A.1, "Inservice Inspection." As committed to during the conference call between NRC staff and Monticello plant representatives on January 30, 2001 concerning a verbal request for Notice of Enforcement Discretion (NOED), a follow-up written request for NOED was submitted on February 1, 2001 (Reference 2). This letter is submitted pursuant to Monticello's further commitment to submit a license amendment request on or before February 1, 2001 to permanently rectify the situation which required the NOED.

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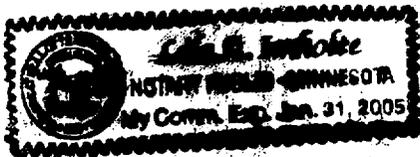
Compliance with the current wording of Technical Specification 3.15.A requires full compliance with Section XI of the ASME Boiler and Pressure Vessel Code (the Code) as a condition for considering Section XI-required equipment operable. Application of Technical Specification 3.15.A, requires declaring equipment inoperable and following the specified Limiting Condition for Operation requirements when a Code non-compliance is discovered. This may require an unnecessary plant shutdown when the equipment is fully operable in all other respects.

This exigent situation occurred because the potential for Technical Specification 3.15.A.1 to cause unnecessary operational evolutions was not previously recognized. Code nonconformances were recently identified during the course of inspections conducted by NRC staff. Technical Specification 3.15.A.1 directs that affected components be declared inoperable without regard for actual impact on operability. The need for a license amendment that would allow such nonconformances to be evaluated for their affect on equipment operability, thus preventing unnecessary operational evolutions, was subsequently identified. As a result, the need for a license amendment was determined to be unavoidable and not created by a failure to make a timely application for a license amendment.

Exhibit A contains a description of the proposed changes, the reasons for requesting the changes, and the supporting safety evaluation and significant hazards determination. Exhibit B contains current Monticello Technical Specification pages marked up to show the proposed changes. Exhibit C contains the revised Monticello Technical Specification pages.

We expect the amendment to be effective immediately upon issuance. Due to the number of procedures that require changes (e.g., cross-references) we request an implementation period of 30 days upon issuance of this proposed amendment.

If you have any questions related to this License Amendment Request please contact Doug Neve, Sr. Licensing Engineer, at 763-295-1353.



by James R. Morris  
James R. Morris  
Site General Manager  
Monticello Nuclear Generating Plant

Subscribed to and sworn before me this 1 day of February, 2001.

Sila A. Amhotté  
Notary



## Exhibit A

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### License Amendment Request Relocation of ASME Inservice Inspection Requirements to a Licensee Program

#### Evaluation of Proposed Change to the Monticello Technical Specifications

Pursuant to 10 CFR Part 50, Section 50.90 and 50.91, Nuclear Management Company, (NMC), LLC hereby proposes the following change to Appendix A to Facility Operating License DPR-22, "Technical Specifications" for the Monticello Nuclear Generating Plant.

#### Background

Monticello Technical Specification 3.15.A.1 includes the following requirement:

*To be considered operable, Quality Group A, B, and C components shall satisfy the requirements contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for continued service of ASME Code Class 1, 2, and 3 components, respectively, except where relief has been granted by the Commission pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).*

Revision 3 of the Monticello ISI plan for the third ten year interval was forwarded to NRC via Reference 1. The plan conforms to Section XI of the 1986 edition of the ASME Code, with approved requests for relief.

Details of the Inservice Inspection Program (ISI) in the Technical Specifications are proposed to be relocated to the licensee controlled ISI program. The ISI program is required by 10 CFR 50.55a to be performed in accordance with ASME Code Section XI. Compliance with 10 CFR 50.55a is required by the Monticello Operating License. The Monticello ISI program implements the applicable provisions of ASME Code Section XI. Changes to the license controlled ISI program are controlled by the provisions of 10 CFR 50.55a.

#### Proposed Changes and Reasons for Changes

The proposed changes to Monticello Operating License, Appendix A, "Technical Specifications," are described below. Specific wording changes are shown in Exhibits B and C.

The purpose of this amendment request is to remove the Inservice Inspection requirements from the Monticello TS and relocate them to a licensee controlled program. Upon NRC approval of the requested changes, the ISI requirements currently defined in the TS would be embodied within the Monticello ISI program. The ISI program requirements do not meet the criteria of 10 CFR 50.36 for inclusion in the Technical Specifications. The Monticello Operating License includes provisions to meet 10 CFR 50, including 10 CFR 50.55a for ISI. The proposed license change is also consistent with NUREG-1433 (Reference 2).

Details of the ISI program in the Technical Specifications are proposed to be relocated to the licensee controlled ISI program. The ISI program is required by 10 CFR 50.55a to be performed in accordance with ASME Code Section XI. Compliance with 10 CFR 50.55a is required by the Monticello Operating License. The Monticello ISI program

## Exhibit A

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implements the applicable provisions of ASME Code Section XI. Changes to the licensee controlled ISI program are controlled in accordance with the provisions of 10 CFR 50.55a.

Relocation of the ISI program will allow evaluation of Code non-conformances to be evaluated under the guidance of NRC Generic Letter 91-18 (Reference 3) as part of our corrective action program. As such, non-conformances would be evaluated for operability, and Code conformance would be regained or relief requested.

In addition, since the Inservice Testing Program is the only requirement remaining in the Technical Specifications, the reference to ASME Code Class 1,2, and 3 "components" will be changed to "pumps and valves" for clarity. Pumps and valves are the only components related to the Inservice Testing Program.

The following changes are proposed:

1. Table of Contents: The Table of Contents is revised to reflect the deletion of Inservice Inspection requirements from Technical Specifications 3.15 and 4.15.

Justification: The Table of Contents is revised to reflect the requested relocation of the ASME Inservice Inspection requirements to the licensee controlled program.

2. Specification 3.15 and 4.15, Inservice Inspection and Testing: Specification titles, Applicability, and Objectives are reworded to be specific to Inservice Testing requirements. Specifications 3.15.A and 4.15.A are deleted, since they are specific to Inservice Inspection.

Justification: The requirements are to be removed from the TS as ISI does not meet the criteria of 10 CFR 50.36 for inclusion in Technical Specifications. They will remain in the Monticello ASME Code Section XI ISI program. This change does not reduce any commitment to meeting ASME Code Section XI requirements.

Additionally, NUREG 1433, "Standard Technical Specifications, General Electric Plant, BWR/4, " Revision 1 (Reference 2) does not include ISI.

Similar changes have been implemented in plants converting to the Standard Technical Specifications (STS). One example is the conversion of WNP-2, which was submitted December 8, 1995, and approved by the NRC March 4, 1997.

Appropriate changes to the Technical Specification Bases are also included.

### Safety Evaluation

The changes proposed above relocate requirements for ISI to a licensee controlled program because the ISI program requirements do not meet the criteria of 10 CFR 50.36 for inclusion in the Technical Specifications. A comparison to the deterministic screening criteria of 10 CFR 50.36 is as follows:

1. The ISI program requirements do not constitute installed instrumentation used for, or capable of detecting a significant abnormal degradation of the reactor coolant pressure boundary prior to a design basis accident (DBA).

## Exhibit A

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2. The ISI program requirements do not constitute a process variable, design feature, or operating restriction assumed as initial conditions of a DBA or transient analyses that assumes the failure of, or presents a challenge to the integrity of a fission product barrier.
3. The ISI program does not constitute a primary success path in the mitigation of a DBA or transient that assumes the failure of, or presents a challenge to the integrity of a fission product barrier.
4. The ISI program does not constitute a system, structure or component which is a significant risk contributor to public health and safety. Assurance of operability of entire systems as verified in the system Specifications dominates risk contribution of the systems, compared to the long term assurance of system integrity provided by the ISI program requirements.

The individual system Technical Specification surveillances provide assurance that the systems are operable and in a ready state for accident mitigation. The ISI program requirements monitor long term component degradation and continued long term maintenance of structural conditions.

Thus, the proposed change eliminates requirements not required to be included in Technical Specifications. No reduction in any previous commitments to 10 CFR 50.55a or the ASME Code is proposed as a result of the relocation. This change is also consistent with NUREG-1433 (Reference 2).

### Determination of Significant Hazards Considerations

Nuclear Management Company, LLC (NMC) has proposed changes to the Monticello Technical Specifications (TS) for ASME Code Section XI required Inservice Inspection (ISI). The proposed changes relocate the ISI requirements to a licensee program document. Requirements for ISI programs do not meet the criteria of 10 CFR 50.36 for inclusion in Technical Specifications. The proposed changes have been evaluated to determine whether they constitute a significant hazards consideration as required by 10 CFR Part 50, Section 50.91 using the standards provided in Section 50.92. This analysis is provided below:

1. The proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated.

The requested changes are administrative in nature in that they relocate ISI requirements from the TS to the Monticello ISI program. The requested changes will not revise previous commitments to 10 CFR 50.55a or ASME Code Section XI ISI requirements.

The proposed changes do not involve a change to the configuration or method of operation of any plant equipment that is used to mitigate the consequences of an accident, nor do they affect any assumptions or conditions in any of the accident analyses. Since the accident analyses remain bounding, their radiological consequences are not adversely affected.

## Exhibit A

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Therefore, the probability or consequences of an accident previously evaluated are not affected.

2. The proposed amendment will not create the possibility of a new or different kind of accident from any accident previously analyzed.

The requested changes are administrative in nature in that they relocate ISI requirements from the TS to the Monticello ISI program. The requested changes will not revise previous commitments to 10 CFR 50.55a or ASME Code Section XI ISI requirements.

The proposed changes do not involve a change to the configuration or method of operation of any plant equipment that is used to mitigate the consequences of an accident, nor do they affect any assumptions or conditions in any of the accident analyses. Accordingly, no new failure modes have been defined for any plant system or component important to safety nor has any new limiting single failure been identified as a result of the proposed changes.

Therefore the possibility of a new or different kind of accident from any accident previously evaluated is not created.

3. The proposed amendment will not involve a significant reduction in the margin of safety.

The requested changes are administrative in nature in that they relocate ISI requirements from the TS to the Monticello ISI program. The requested changes will not revise previous commitments to 10 CFR 50.55a or ASME Code Section XI ISI requirements. Program requirements will ensure that Code requirements are met.

Therefore, a significant reduction in the margin of safety is not involved.

Based on the above evaluation, and pursuant to 10 CFR 50.91, the operation of Monticello in accordance with the proposed license amendment request does not involve any significant hazards considerations as defined by NRC regulations in 10 CFR 50.92.

### Environmental Assessment

NMC has evaluated the proposed changes and determined that:

1. The changes do not involve a significant hazards consideration, or
2. The changes do not involve a significant change in the types or significant increase in the amounts of any effluents that may be released offsite, or
3. The changes do not involve a significant increase in individual or cumulative occupational radiation exposure.

Accordingly, the proposed changes meet the eligibility criterion for categorical exclusion set forth in 10 CFR Part 51 Section 51.22(c)(9). Therefore, pursuant to 10 CFR Part 51 Section 51.22(b), an environmental assessment of the proposed changes is not required.

## Exhibit A

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### REFERENCES

1. NSP letter to NRC, "Revision No. 3 to the Monticello Inservice Inspection Plan," dated April 14, 2000.
2. NUREG-1433, "Standard Technical Specification, General Electric Plants, BWR/4," Revision 1, dated April 7, 1995
3. NRC Generic Letter 91-18, "INFORMATION TO LICENSEES REGARDING TWO NRC INSPECTION MANUAL SECTIONS ON RESOLUTION OF DEGRADED AND NONCONFORMING CONDITIONS AND ON OPERABILITY," Revision 1, dated October 8, 1997

## Exhibit B

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### License Amendment Request

#### Relocation of ASME Inservice Inspection Requirements to a Licensee Program

#### Current Monticello Operating License and Monticello Technical Specification Pages Marked Up With Proposed Change

This exhibit consists of current Monticello Operating License and Technical Specification pages marked up with the proposed change. The pages included in this exhibit are listed below:

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Monticello Technical Specifications pages

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### 3.0 LIMITING CONDITIONS FOR OPERATION

#### 3.15 INSERVICE INSPECTION AND TESTING

Applicability:

~~Applies to components which are part of the reactor coolant pressure boundary and their supports and other safety-related pressure vessels, piping, pumps, and valves.~~

Objective:

~~To assure the integrity of the reactor coolant pressure boundary and the operational readiness of safety-related pressure vessels, piping, pumps, and valves.~~

Specification:

~~A. Inservice Inspection(Deleted)~~

- ~~1. To be considered operable, Quality Group A, B, and C components shall satisfy the requirements contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for continued service of ASME Code Class 1, 2, and 3 components, respectively, except where relief has been granted by the Commission pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).~~

3.15/4.15

### 4.0 SURVEILLANCE REQUIREMENTS

#### 4.15 INSERVICE INSPECTION AND TESTING

Applicability:

~~Applies to the periodic inspection and testing of components which are part of the reactor coolant pressure boundary and their supports and other safety-related pressure vessels, piping, pumps, and valves.~~

Objective:

~~To verify the integrity of the reactor coolant pressure boundary and the operational readiness of safety-related pressure vessels, piping, pumps, and valves.~~

Specification:

~~A. Inservice Inspection(Deleted)~~

- ~~1. Inservice Inspection of Quality Group A, B, and C components shall be performed in accordance with the requirements for ASME Code Class 1, 2, and 3 components, respectively, contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where relief has been granted by the Commission pursuant to 10 CFR 50, Section 50.55a(g)(6)(i)~~

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Bases 3.15/4.15:

~~The inservice inspection program for the Monticello plant conforms to the requirements of 10 CFR 50, Section 50.55a(g). Where practical, the inspection of components classified into NRC Quality Groups A, B, and C conforms to the requirements of ASME Code Class 1, 2, and 3 components, respectively, contained in Section XI of the ASME Boiler and Pressure Vessel Code. A program of inservice testing of Quality Group A, B, and C pumps and valves is also in effect at the Monticello plant, that conforms to the requirements contained in Section XI of the ASME Boiler and Pressure Vessel Code or where alternate testing is justified in accordance with Generic Letter 89-04. If a Code required inspection is impractical for the Monticello facility, a request for a deviation from that requirement is submitted to the Commission in accordance with 10 CFR 50, Section 50.55a(g)(6)(i).~~

~~Deviations which are needed from the procedures prescribed in Section XI of the ASME Code and applicable Addenda will be reported to the Commission prior to the beginning of each 10-year inspection period if they are known to be required at that time. Deviations which are identified during the course of inspection will be reported quarterly throughout the inspection period.~~

## Exhibit C

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### License Amendment Request

#### Relocation of ASME Inservice Inspection Requirements to a Licensee Program

#### Revised Monticello Operating License and Technical Specification Pages

This exhibit consists of revised Monticello Operating License Technical Specification pages that incorporate the proposed change. The pages included in this exhibit are as listed below:

#### Pages

Monticello Technical Specifications pages

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**3.0 LIMITING CONDITIONS FOR OPERATION**

**4.0 SURVEILLANCE REQUIREMENTS**

**3.15 INSERVICE TESTING**

Applicability:

Applies to safety-related pumps and valves.

Objective:

To assure the integrity and operational readiness of safety-related pumps and valves.

Specification:

A. (Deleted)

**4.15 INSERVICE TESTING**

Applicability:

Applies to the periodic testing of safety-related pumps and valves.

Objective:

To verify the integrity and operational readiness of safety-related pumps and valves.

Specification:

A. (Deleted)

Bases 3.15/4.15:

A program of inservice testing of Quality Group A, B, and C pumps and valves is in effect at the Monticello plant that conforms to the requirements contained in Section XI of the ASME Boiler and Pressure Vessel Code or where alternate testing is justified in accordance with Generic Letter 89-04. If a Code required inspection is impractical for the Monticello facility, a request for a deviation from that requirement is submitted to the Commission in accordance with 10 CFR 50, Section 50.55a(g)(6)(i).