



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

January 31, 2001

Mr. T. F. Plunkett
President - Nuclear Division
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

SUBJECT: TURKEY POINT UNITS 3 AND 4 - ISSUANCE OF AMENDMENTS
REGARDING CHANGES TO CONTAINMENT STRUCTURAL INTEGRITY
TECHNICAL SPECIFICATIONS (TAC NOS. MA9047 AND MA9048)

Dear Mr. Plunkett:

The U.S. Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. 210 to Facility Operating License No. DPR-31 and Amendment No. 204 to Facility Operating License No. DPR-41 for the Turkey Point Plant, Units 3 and 4, respectively. The amendments revise Turkey Point Plant, Units 3 and 4, Technical Specification Section 3/4.6.1.6, to incorporate that portion of the August 8, 1996, Final Amended Rule (61 FR 41303) related to the revised requirements of inservice inspection of the containment post-tensioning system. The amendments are in response to your application dated May 22, 2000, as supplemented by letter dated October 4, 2000.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Kahtan N. Jabbour, Senior Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-250 and 50-251

Enclosures:

1. Amendment No. 210 to DPR-31
2. Amendment No. 204 to DPR-41
3. Safety Evaluation

cc w/enclosures: See next page

January 31, 2001

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*See previous concurrence

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT PLANT UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 210
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power and Light Company (the licensee) dated May 22, 2000, as supplemented by letter dated October 4, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-31 is hereby amended to read as follows:

B. Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 210, are hereby incorporated in the license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard P. Correia, Chief, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: **January 31, 2001**



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT PLANT UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 204
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power and Light Company (the licensee) dated May 22, 2000, as supplemented by letter dated October 4, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-41 is hereby amended to read as follows:

(B) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 204, are hereby incorporated in the license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard P. Correia, Chief, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: **January 31, 2001**

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 210 FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 204 FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NOS. 50-250 AND 50-251

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

Remove pages

3/4 6-8

3/4 6-9

3/4 6-10

B 3/4 6-2

Insert pages

3/4 6-8

3/4 6-9

3/4 6-10

3/4 6-10a

B 3/4 6-2

CONTAINMENT SYSTEMS

BASES

3/4.6.1.5 AIR TEMPERATURE

The limitations on containment average air temperature ensure that the design limits for a LOCA are not exceeded, and that the environmental qualification of equipment is not impacted. If temperatures exceed 120° F, but remain below 125° F for up to 336 hours during a calendar year, no action is required. If the 336-hour limit is approached, an evaluation may be performed to extend the limit if some of the hours have been spent at less than 125° F. Measurements shall be made at all listed locations, whether by fixed or portable instruments, prior to determining the average air temperature.

3/4.6.1.6 CONTAINMENT STRUCTURAL INTEGRITY

This limitation ensures that the structural integrity of the containment will be maintained comparable to the original design standards for the life of the facility. Structural integrity is required to ensure that the containment will withstand the maximum analyzed peak pressure of 49.9 psig in the event of a LOCA. The measurement of containment tendon lift-off force, the tensile tests of the tendon wires or strands, the visual examination of tendons, anchorages and exposed interior and exterior surfaces of the containment, and the Type A leakage test are sufficient to demonstrate this capability.

Some containment tendons are inaccessible at one end due to personnel safety considerations at potential steam exhaust locations. These tendons, if selected for examination, will be exempted from the full examination requirements, and the following alternative examinations shall be performed:

1. The accessible end of each exempt tendon shall be examined in accordance with IWL-2524 and IWL-2525.
2. For each exempt tendon, a substitute tendon shall be selected and examined in accordance with IWL requirements.
3. In addition, an accessible tendon located as close as possible to each exempt tendon shall be examined at both ends in accordance with IWL-2524 and IWL-2525.

The required Special Reports from any engineering evaluation of containment abnormalities shall include a description of the tendon condition, the condition of the concrete (specially at tendon anchorages), the inspection procedures, the tolerances on cracking, the results of the engineering evaluation, and the corrective actions taken.

The submittal of a Special Report for a failed tendon surveillance is considered an administrative requirement and it does not impact the plant operability. The administrative requirements for Special Reports are defined in Technical Specifications section 6.9.2.

3/4.6.1.7 CONTAINMENT VENTILATION SYSTEM

The containment purge supply and exhaust isolation valves are required to be closed during a LOCA. When not purging, power to the purge valve actuators will be removed (sealed closed) to prevent inadvertent opening of these valves. Maintaining these valves sealed closed during plant operation ensures that excessive quantities of radioactive materials will not be released via the Containment Purge System.

Leakage integrity tests with a maximum allowable leakage rate for containment purge supply and exhaust supply valves will provide early indication of

CONTAINMENT SYSTEMS

CONTAINMENT STRUCTURAL INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.6 The structural integrity of the containment shall be maintained at a level consistent with the acceptance criteria in Specification 4.6.1.6.

APPLICABILITY MODES 1, 2, 3, and 4.

ACTION:

- a. With more than one tendon with an observed lift-off force between 90% and 95% of the predicted force, or with one tendon below 90% of the predicted force, restore the tendon(s) to the required level of integrity within 15 days and perform an engineering evaluation of the containment and provide a Special Report to the Commission within 30 days in accordance with Specification 6.9.2 or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With the average of all measured tendon forces for each type of tendon (dome, vertical, and hoop), including those measured in ACTION a., less than the predicted force, restore the tendon(s) to the required level of integrity within 15 days and perform an engineering evaluation of the containment and provide a Special Report to the Commission within 30 days in accordance with Specification 6.9.2 or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With any abnormal degradation of the structural integrity other than ACTION a. and ACTION b., at a level below the acceptance criteria of Specifications 4.6.1.6.1, 4.6.1.6.2 and 4.6.1.6.3, restore the containment to the required level of integrity within 72 hours and perform an engineering evaluation of the containment and provide a Special Report to the Commission within 15 days in accordance with Specification 6.9.2 or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.6.1 Containment Tendons. The containment tendons and the containment exterior surfaces shall be examined in accordance with ASME Boiler and Pressure Vessel Code, Section XI, Subsection IWL, "Requirements for Class CC Concrete Components of Light-Water Cooled Plants," and the modifications presented in 10 CFR 50.55a(b)(2)(viii), "Examination of concrete containments," as modified by approved exemptions. The containment structural integrity shall be demonstrated during the inspection periods specified in IWL-2410 and IWL-2420. The tendons' structural integrity shall be demonstrated by:

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- a. Determining that tendons, selected in accordance with IWL-2521, have the average of all measured tendon forces for each type of tendon (dome, vertical and hoop) equal to or greater than the minimum required prestress specified at the anchorage for that type of tendon.
- b. Assuring that the measured force in each individual tendon is not less than 95% of the predicted force unless the following conditions are satisfied:
 - 1) The measured force in no tendon is below 90% of the predicted force and the measured force in no more than one tendon is between 90% and 95% of the predicted force;
 - 2) The measured force in two tendons located adjacent to the tendon in 1) are not less than 95% of the predicted forces; and
 - 3) The measured forces in all the remaining sample tendons are not less than 95% of the predicted force.

The predicted force for each tendon shall be calculated individually for each inspection prior to the beginning of each inspection, and should consider such factors as:

- Prestressing history;
- Friction losses; and
- Time-dependent losses (creep, shrinkage, relaxation), considering time elapsed from prestressing.

When evaluation of consecutive surveillances of prestressing forces for the same tendon or tendons in a group indicates a trend of prestress loss such that the tendon force(s) would be less than the minimum design prestress requirements before the next inspection interval, an evaluation shall be performed and reported in the Engineering Evaluation Report as prescribed in IWL-3300.

- c. Performing tendon detensioning, examinations, and testing on a sample tendon of each type (dome, vertical, and hoop). A single wire or strand shall be removed from each detensioned tendon. Each removed wire or strand shall be examined over its entire length for corrosion and mechanical damage. Tension tests shall be performed on each removed wire or strand: one at each end, one at mid-length, and one in the location of the most corroded area, if any. The following information shall be obtained from each test:
 - 1) Yield strength;
 - 2) Ultimate tensile strength;
 - 3) Elongation.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

The condition of wire or strand is acceptable if:

- 1) Samples are free of physical damage;
 - 2) Sample ultimate tensile strength and elongation are not less than minimum specified values.
- d. Performing tendon retensioning of those tendons that have been detensioned to at least the force predicted for the tendon at the time of the test. However, the retensioning force shall not exceed 70% of the specified minimum ultimate tensile strength of the tendon based on the number of effective wires or strands in the tendon at the time of retensioning. During retensioning of these tendons, if the elongation corresponding to a specific load (adjusted for effective wires or strands) differs by more than 10% from that recorded during the last measurement, an evaluation must be performed to determine whether the difference is related to wire failures or slip of wires in anchorage. A difference of more than 10% must be identified in the ISI Summary Report required by IWA-6000.
- e. Performing examination of corrosion protection medium and free water in accordance with IWL-2525, with acceptance standards prescribed in IWL-3221.4. The following conditions, if they occur, shall be reported in the ISI Summary Report required by IWA-6000:
- 1) The sheathing filler grease contains chemically combined water exceeding 10% by weight or the presence of free water;
 - 2) The absolute difference between the amount removed and the amount replaced exceeds 10% of the tendon net duct volume.
 - 3) Grease leakage is detected during general visual examination of the containment surface.

4.6.1.6.2 End Anchorages and Containment Concrete Surfaces. The structural integrity of the end anchorages of all tendons inspected pursuant to Specification 4.6.1.6.1 and the containment concrete surfaces shall be demonstrated by performing examination of tendon anchorage areas and containment concrete surfaces in accordance with IWL-2000, with acceptance standards prescribed in IWL-3000. Acceptability of inaccessible areas shall be evaluated when conditions exist in accessible areas that could indicate the presence of or result in degradation to such inaccessible areas. For each inaccessible area identified, the following shall be provided in the ISI Summary Report required by IWA-6000:

- 1) A description of the type and estimated extent of degradation, and the conditions that led to the degradation;
- 2) An evaluation of each area, and the result of the evaluation; and
- 3) A description of necessary corrective actions.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

4.6.1.6.3. Containment Surfaces Inspection for Containment Leakage Rate Testing Program. In accordance with the Containment Leakage Rate Testing Program, a visual inspection of the accessible interior and exterior surfaces of the containment, including the liner plate, shall be performed. The purpose of this inspection shall be to identify any evidence of structural deterioration which may affect containment structural integrity or leaktightness. The visual inspection shall be general in nature; its intent shall be to detect gross areas of widespread cracking, spalling, gouging, rust, weld degradation, or grease leakage. The visual examination may include the utilization of binoculars or other optical devices. Corrective actions taken, and recording of structural deterioration and corrective actions, shall be in accordance with the Containment Leakage Rate Testing Program. Records of previous inspections shall be reviewed to verify no apparent changes in appearance. The first inspection performed will form the baseline for future surveillances.



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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 210 TO FACILITY OPERATING LICENSE NO. DPR-31
AND AMENDMENT NO. 204 TO FACILITY OPERATING LICENSE NO. DPR-41
CONTAINMENT STRUCTURAL INTEGRITY TECHNICAL SPECIFICATION
FLORIDA POWER AND LIGHT COMPANY
TURKEY POINT UNIT NOS. 3 AND 4
DOCKET NOS. 50-250 AND 50-251

1.0 INTRODUCTION

By letter dated May 22, 2000, as supplemented October 4, 2000, Florida Power and Light Company (FPL or the licensee), proposed an amendment to the Technical Specifications (TS) for Turkey Point Plant, Units 3 and 4 (Ref. 1). The amendment essentially consists of incorporating into the TS that portion of the August 8, 1996, Final Rule on 10 CFR 50.55a [Title 10, Code of Federal Regulations 50.55a] (61 FR 41303), related to inservice inspection of the containment post-tensioning system. The supplemental letter did not expand the scope of the initial *Federal Register* Notice nor change the U.S. Nuclear Regulatory Commission (NRC) staff's initial proposed no significant hazards consideration determination.

The Final Rule states that the American Society of Mechanical Engineers (ASME), Section XI, Subsection IWL, as modified and supplemented by the requirements of 10 CFR 50.55a(b)(2)(viii), shall be used by the licensees of nuclear power plants (NPPs) when performing containment examinations. The Final Rule requires implementation of the revised requirements for containment examination by September 9, 2001. FPL is planning on performing the containment tendon surveillance for Turkey Point Units 3 and 4 in March 2001.

2.0 EVALUATION

The containments of Turkey Point Units 3 and 4 are constructed from prestressed concrete. The regulation, 10 CFR 50.55a, requires the licensees of NPPs with prestressed concrete containments to perform the inservice inspection (ISI) of concrete and post-tensioning tendons in accordance with the 1992 Edition and 1992 Addenda of Subsection IWL of Section XI of the ASME Boiler and Pressure Vessel Code (the Code) as modified by 10 CFR 50.55a(b)(2)(ix). The regulation also requires the licensees of NPPs to perform the ISI of their metallic components (in this case containment liners and penetrations) in accordance with the 1992 Edition and 1992 Addenda of Subsection IWE of the Code, as modified by 10 CFR 50.55a(b)(2)(x). The Turkey Point Units 3 and 4 TS related to containment integrity are contained in Sections 3.6.1.1 to 3.6.1.6, and the associated surveillance requirements are contained in Sections 4.6.1.1 to 4.6.1.6.

ENCLOSURE

In the above submittals, FPL proposed to revise the following Turkey Point Units 3 and 4 TS to incorporate the requirements specified in ASME Section XI, Subsection IWL, as modified and supplemented by the requirements of 10 CFR 50.55a(b)(2)(viii), Examination of concrete containments:

TS 3.6.1.6 a, b, and c, will be revised to conform to IWL tendon lift-off force requirements;

Surveillance Requirements 4.6.1.6.1, 4.6.1.6.2, 4.6.1.6.3 will be revised to conform to the containment tendon and containment surfaces inspection requirements specified in ASME Section XI, Subsection IWL, 1992 Edition with the 1992 Addenda, and 10 CFR 50.55a(b)(2)(viii).

The licensee proposed change to LCO [limiting condition for operation] 3.6.1.6 b, related to the exempt tendons, substitutes the present criterion with an acceptance criterion that considers the average prestressing forces in the tendons, as required by the ASME Code, Subsection IWL, Paragraph IWL-3221.1. The staff was concerned regarding this substitution in the LCO. However, the staff's concerns were resolved by the licensee's actions regarding the exempt tendons as stated in the relief granted to the licensee by letter dated October 20, 1999 (Ref. 2). The licensee also provided additional information in Ref. 3 to address the staff's concerns regarding the effects of high temperature on the exempt tendons due to the closeness of steam exhaust. In addition, the licensee stated that in order to consider the high- temperature effects on all tendons, in 1994 it performed a containment re-analysis assuming a 50 percent increase in the tendon relaxation loss. The staff evaluated and accepted the licensee's analysis of this issue as stated in a letter to the licensee dated November 29, 1995 (Ref. 4). Based on the above discussion, the staff finds the change to TS 3.6.1.6 b acceptable.

The proposed changes to TS 3.6.1.6 a, and c, and Surveillance Requirements 4.6.1.6.1, 4.6.1.6.2 and 4.6.1.6.3, are consistent with the tendon inspection requirements of ASME Subsection IWL as modified and supplemented by the requirements of 10 CFR 50.55a(b)(2)(ix). On the basis of its review, the staff finds that the proposed containment examination requirements are equivalent to, or more rigorous than the previous TS requirements. Therefore, the changes are acceptable.

3.0 STATE CONSULTATION

Based upon a letter dated March 8, 1991, from Mary E. Clark of the State of Florida, Department of Health and Rehabilitative Services, to Deborah A. Miller, Licensing Assistant, NRC, the State of Florida does not desire notification of issuance of license amendments.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20, and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding

that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (65 FR 48750, dated August 9, 2000). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

Based on the staff evaluation discussed above, the staff concludes that the proposed TS changes are acceptable.

The Commission has concluded, based on the consideration discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

6.0 REFERENCES

1. Letter from R. Hovey (FPL) to NRC, "Proposed License Amendments; Changes to Structural Integrity Technical Specifications," May 22, 2000.
2. Letter from R. Hernan (NRC) to T. Plunkett (FPL), "Safety Evaluation of the Containment Inservice Inspection Program, First Ten-Year Interval Request for Relief No. 20 - Turkey Point Plant, Units 3, 4," October 20, 1999.
3. Letter from R. Hovey (FPL) to NRC, "Response to Request for Additional Information - Proposed License Amendments," October 4, 2000.
4. Letter from R. Croteau (NRC) to J. Goldberg (FPL), "Containment Structural Re-Analysis," November 29, 1995.

Principal Contributor: H. Ashar, NRR

Date: January 31, 2001

Mr. T. F. Plunkett
Florida Power and Light Company

TURKEY POINT PLANT

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