TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
CHAPTER 1.0 - USE AND APPLICATION

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
1.0-LA.1	N/A	Not used.	N/A	N/A	N/A
1.0-LA.2	1.32.a, b, c, f and 1.39.b, e	Moves items a, b, c, and f from the PRIMARY CONTAINMENT INTEGRITY definition to the ITS 3.6.1.1 Bases and items b and e from the CTS SECONDARY CONTAINMENT INTEGRITY definition to the ITS 3.6.4.1 Bases.	Bases	Bases Control Process in ITS Chapter 5.0	1

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
CHAPTER 2.0 - SAFETY LIMITS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.0 - LCO AND SR APPLICABILITY

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.1 - REACTIVITY CONTROL SYSTEMS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.1.1, SHUTDOWN MARGIN					
NONE	NONE	NONE	NONE	NONE	NONE
3.1.2, Reactivity Anomalies					
3.1.2-LA.1	3.1.2 Action a	The requirement to perform an analysis to determine and explain the cause of the reactivity difference.	Bases	Bases Control Program in ITS Chapter 5	3
3.1.3, Control Rod OPERABILITY					
3.1.3-LA.1	3.1.3.1 Actions a.1.b), b.1.b), and b.2.a), 3.1.3.6 Action a.1.b), 3.1.3.7 Action a.3.b)	The details of the recommended procedures for disarming control rod drives.	Bases	Bases Control Program in ITS Chapter 5	3
3.1.3-LA.2	3.1.3.7 Actions a.1 and a.2	Details of methods for determining the position of a control rod.	Bases	Bases Control Program in ITS Chapter 5	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.1 - REACTIVITY CONTROL SYSTEMS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.1.4, Control Rod Scram Times					
3.1.4-LA.1	4.1.3.2.c	ITS SR 3.1.4.2 will test a "representative sample" of control rods each 120 days of power operation instead of the CTS requirement of "10% of the control rods on a rotating basis." The details of what constitutes a representative sample are relocated.	Bases	Bases Control Program in ITS Chapter 5	3
3.1.5, Control Rod Scram Accumulators					
NONE	NONE	NONE	NONE	NONE	NONE
3.1.6, Rod Pattern Control					
NONE	NONE	NONE	NONE	NONE	
3.1.7, Standby Liquid Control System					
3.1.7-LA.1	4.1.5.a.2	The detail of the method for performance of the temperature verification surveillance (verifying temperature "on the local indicator").	Bases	Bases Control Program in ITS Chapter 5	3
3.1.7-LA.2	4.1.5.c.1	The details of the method for performing the Surveillance to verify flow through the SLC subsystem into the reactor pressure vessel (initiating an explosive valve and the requirements on the replacement charges for explosive valve).	Bases	Bases Control Program in ITS Chapter 5	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.1 - REACTIVITY CONTROL SYSTEMS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.1.7-LA.3	4.1.5.c.1, 4.1.5.c.4	The details of the method for performing: 1) The Surveillance to verify flow through the SLC subsystem into the reactor pressure vessel, and 2)The Surveillance to demonstrate all piping between the SLC storage tank and the pump suction is unblocked.	Bases	Bases Control Program in ITS Chapter 5	3
3.1.7-LA4	4.1.5.c.3	The testing requirements for SLC System relief valve setting verification.	IST Program	IST Program in ITS Chapter 5	5
3.1.7-LA.5	Figure 3.1.5-2	The contained tank volumes associated with the SLC storage tank low level and high level alarms.	UFSAR	10 CFR 50.59	1
3.1.8, SDV Vent and Drain Valves					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.1.3.8, Control Rod Drive Housing Support					
None-L.1	3/4.1.3.8	Requirement for Control Rod Drive Housing Support to be in place.	NONE	NONE	NONE
Current Specification 3/4.1.6, Economic Generation Control System					
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.2 - POWER DISTRIBUTION LIMITS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.2.1, AVERAGE PLANAR LINEAR HEAT GENERATION RATE					
3.2.1-LA.1	3.2.1 Action	The requirement in the CTS 3.2.1 ACTION to "initiate corrective action within 15 minutes" to restore the limit is relocated in the form of a discussion that "prompt action" should be taken to restore the parameter to within the limits.	Bases	Bases Control Program in ITS Chapter 5	3
3.2.2, MINIMUM CRITICAL POWER RATIO					
3.2.2-LA.1	3.2.3 Action	The requirement in the CTS 3.2.3 ACTION to "initiate corrective action within 15 minutes" to restore the limit is relocated in the form of a discussion that "prompt action" should be taken to restore the parameter to within the limits.	Bases	Bases Control Program in ITS Chapter 5	3
3.2.2-LA.2	4.2.3.2	The detail of the method for comparing scram testing results and determining the MCPR limit is relocated in the form of a discussion.	Bases	Bases Control Program in ITS Chapter 5	3
3.2.2-LA.3	4.2.3.2	The requirement that, prior to initial scram time testing for an operating cycle, the MCPR operating limit be based on the Technical Specification Scram Speed (TSSS).	COLR	COLR change control process described in Chapter 5 of the ITS	3
3.2.3, LINEAR HEAT GENERATION RATE					

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.2 - POWER DISTRIBUTION LIMITS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.2.3-LA.1	3.2.4 Action	The requirement in the CTS 3.2.4 ACTION to "initiate corrective action within 15 minutes" to restore the limit is relocated in the form of a discussion that "prompt action" should be taken to restore the parameter to within the limits.	Bases	Bases Control Program in ITS Chapter 5	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.3 - INSTRUMENTATION

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.3.1.1, RPS Instrumentation					
3.3.1.1-LA.1	Table 3.3.1-2, LCO 3.3.1, 4.3.1.3	The RPS response times and associated information (description of how the APRM response time is measured).	TRM	10 CFR 50.59	1
3.3.1.1-LA.2	3.3.1 Action footnotes * and **	The details relating to placing channels in trip (e.g, if tripping causes Trip Function to occur, tripping trip system with the most inoperable channels).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.1.1-LA.3	Table 4.3.1.1-1 Note (b), 4.3.1.2	Details of the methods for performing CTS 4.3.1.1, the IRM and APRM CHANNEL CHECK (½ decade overlap), and CTS 4.3.1.2, the LOGIC SYSTEM FUNCTIONAL TEST (simulated automatic operation).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.1.1-LA.4		Not used.			
3.3.1.1-LA.5	3.3.1-1 Notes (c) and (i)	The LPRM inputs for OPERABILITY of the APRM (2 per level, 14 total); and the system design details describing that the Turbine Stop Valve — Closure and the Turbine Control Valve Fast Closure, Valve Trip System Oil Pressure — Low Functions are automatically bypassed when THERMAL POWER is less than 25% RTP.	Bases	Bases Control Program in ITS Chapter 5	1

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3.3.1.1-LA.6	Table 3.3.1-1 Notes (e), (g), and (j), Table 2.2.1-1 Function 4	Relocates the following design details: 1) The Main Steam Isolation Valve — Closure Function shall be automatically bypassed when the reactor mode switch is not in the Run position; 2) The Primary Containment Pressure—High Function also actuates the Standby Gas Treatment System; 3) The Turbine Stop Valve—Closure and the Turbine Stop Valve Fast Closure, Valve Trip System Oil Pressure—Low Functions also actuate the EOC-RPT System; and 4) The description that the Reactor Vessel Water Level - Low, Level 3 Allowable Value is in terms of inches "above instrument zero."	UFSAR	10 CFR 50.59	1
3.3.1.1-LA.7	Table 2.2.1-1	Trip Setpoint values.	TRM	10 CFR 50.59	2
3.3.1.1-LA.8	Table 4.3.1.1-1 footnote (d)	The definition of the gain adjustment factor.	Bases	Bases Control Program in ITS Chapter 5	1
3.3.1.1-LA.9	Table 3.3.1-2 footnote ##	The detail concerning the methods of response time testing (that the response time testing and conformance to the administrative limits for the remaining channel including trip unit and relay logic are required).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.1.2, SRM Instrumentation					
3.3.1.2-LA.1	4.3.7.6.c	The detail of the method for performing the Surveillance ("with the detector fully inserted").	Bases	Bases Control Program in ITS Chapter 5	3
3.3.1.2-LA.2	3.9.2, 4.9.2.a.2	The details relating to SRM OPERABILITY; i.e., that the SRMs shall be inserted to the normal operating level with continuous indication in the control room.	Bases	Bases Control Program in ITS Chapter 5	1, 2

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3.3.1.2-LA.3		Not used.			
3.3.1.2-LA.4	3.9.2 Applicability	The additional spatial limitations when movable detectors are being used.	Bases	Bases Control Program in ITS Chapter 5	3
3.3.2.1, Control Rod Block Instrumentation					
3.3.2.1-LA.1	Table 3.3.6-1 Note (a)	The statement that the RBM shall be automatically bypassed when a peripheral control rod is selected.	UFSAR	10 CFR 50.59	1
3.3.2.1-LA.2	Table 4.3.6-1 Function 1, footnote (c), 4.1.4.1.a, 4.1.4.1.b, 4.1.4.1.c	Details of the methods for performing Surveillances (i.e., the RBM CFT includes the reactor manual control multiplexing system input and the RWM CFT includes verifying correct annunciation of the selection error of at least one out-of-sequence control rod and verifying inability to withdraw an out-of-sequence control rod).	Bases	Bases Control Program in Its Chapter 5	3
3.3.2.1-LA.3	Table 3.3.6-2	Trip Setpoint values.	TRM	10 CFR 50.59	2
3.3.2.1-R.1	Tables 3.3.6-1, 3.3.6-2, and 4.3.6-1 Trip Functions 2, 3, 4, 5, and 6	The APRM, SRM, IRM, Scram Discharge Volume, and Recirculation Flow Unit control rod blocks.	TRM	10 CFR 50.59	5
3.3.2.2, Feedwater System and Main Turbine High Water Level Trip Instrumentation					
3.3.2.2-LA.1	Table 3.3.8-2	Trip Setpoint values.	TRM	10 CFR 50.59	2

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3.3.2.2-LA.2	3.3.8 Action b footnote *	The details relating to placing channels in trip (i.e., if tripping causes Trip Function to occur).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.3.1, Post Accident Monitoring Instrumentation					
3.3.3.1-LA.1	Table 3.3.7.5-1 footnote *	The details relating to operation of the drywell hydrogen and oxygen analyzers (actuated after LOCA).	Bases	Bases Control Program in ITS Chapter 5	2
3.3.3.1-LA.2	Table 3.3.7.5-1 ACTION 81	The use of alternate methods of monitoring (initiate the preplanned alternate method of monitoring the parameters).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.3.2, Remote Shutdown Monitoring System					
3.3.3.2-LA.1	3.3.7.4, 3.3.7.4 Action a, 4.3.7.4, Table 3.3.7.4-1, Table 4.3.7.4-1	The CTS details relating to system design and operation (i.e., the specific instrument listings and that remote shutdown panel is external to the control room).	TRM	10 CFR 50.59	1, 2
3.3.4.1, EOC-RPT Instrumentation					
3.3.4.1-LA.1	Table 3.3.4.2-1 footnote (b)	The statement that Turbine Stop Valve—Closure and the Turbine Control Valve—Fast Closure shall not be automatically bypassed when THERMAL POWER is greater than or equal to 25% of RATED THERMAL POWER.	Bases	Bases Control Program in ITS Chapter 5	1

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3.3.4.1-LA.2	Table 3.3.4.2-3	The EOC-RPT System Response Time Limits Table.	TRM	10 CFR 50.59	1
3.3.4.1-LA.3	Table 3.3.4.2-2	Trip Setpoint values.	TRM	10 CFR 50.59	2
3.3.4.1-LA.4	4.3.4.2.3	The description of the Frequency application that is consistent with the ITS definition of STAGGERED TEST BASIS when applied to the two input Functions.	Bases	Bases Control Program in ITS Chapter 5	3
3.3.4.2, ATWS-RPT Instrumentation					
3.3.4.2-LA.1	Table 3.3.4.1-2	Trip Setpoint values.	TRM	10 CFR 50.59	2
3.3.5.1, ECCS Instrumentation					
3.3.5.1-LA.1	Table 3.3.3-2	Trip Setpoint values.	TRM	10 CFR 50.59	2
3.3.5.1-LA.2	4.3.3.2	The detail relating to methods (simulated automatic operation) for performing the LOGIC SYSTEM FUNCTIONAL TESTS.	Bases	Bases Control Program in ITS Chapter 5	3
3.3.5.1-LA.3	Table 3.3.3-1, including footnote (c)	System design and operation details (e.g., Trip System Nomenclature, specific equipment affected, logic description).	Bases	Bases Control Program in Its Chapter 5	1, 2
3.3.5.1-LA.4	Table 3.3.3-3 footnote *	The ECCS response times and associated information.	TRM	10 CFR 50.59	1
3.3.5.2, RCIC System Instrumentation					

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3.3.5.2-LA.1	4.3.5.2	The detail relating to methods (simulated automatic operation) for performing the LOGIC SYSTEM FUNCTIONAL TESTS.	Bases	Bases Control Program in ITS Chapter 5	3
3.3.5.2-LA.2	Table 3.3.5-1, Note (b) and (c)	System design and operation details (i.e., the number of trip systems, and the logic design for the Manual Initiation and Reactor Vessel Water Level—High, Level 8 Functional Units).	Bases	Bases Control Program in ITS Chapter 5	1, 2
3.3.5.2-LA.3	Table 3.3.5-2	Trip Setpoint values.	TRM	10 CFR 50.59	2
3.3.6.1, Primary Containment Isolation Instrumentation					
3.3.6.1-LA.1	Table 3.3.2-2	Trip Setpoint values.	TRM	10 CFR 50.59	2
3.3.6.1-LA.2	Table 3.3.2-3 including footnotes * and #	The Isolation System Instrumentation Response Time Table and associated footnotes.	TRM	10 CFR 50.59	1
3.3.6.1-LA.3	3.3.2 Actions footnotes *, **, and ***	The details relating to placing channels in trip (e.g, if tripping causes Trip Function to occur, tripping trip system with the most inoperable channels).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.6.1-LA.4	4.3.2.2	The detail relating to methods (simulated automatic operation) for performing the LOGIC SYSTEM FUNCTIONAL TESTS.	Bases	Bases Control Program in ITS Chapter 5	3
3.3.6.1-LA.5	Table 3.3.2-1, including footnotes (f), (g), and (h)	System design and operational details (i.e., the Valve Groups operated by signal column, the logic description for certain Functions, and that the Manual Initiation Functions isolate the inboard and outboard valves).	Bases	Bases Control Program in ITS Chapter 5	1, 2

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3.3.6.1-LA.6	Table 3.3.2-2 footnote **	The detail that the Allowable Value for the Reactor Vessel Pressure—High is corrected for cold water head with reactor vessel flooded.	Bases	Bases Control Program in ITS Chapter 5	3
3.3.6.1-LA.7	3.3.2-3 footnote ##	The detail concerning the methods of response time testing (that the response time testing and conformance to the administrative limits for the remaining channel including trip unit and relay logic are required).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.6.1-LA.8	Tables 3.3.2-1, 3.3.2-2, and 4.3.3.1-1 Trip Function 6.c	RHR Pump Suction Flow - High instrumentation requirements.	TRM	10 CFR 50.59	3
3.3.6.1-LA.9	Table 3.3.2-1, 3.3.2-2, and 4.3.2.1-1 Trip Function 3.l	RWCU Pump Suction Flow - High instrumentation requirements.	TRM	10 CFR 50.59	3
3.3.6.2, Secondary Containment Isolation Instrumentation					
3.3.6.2-LA.1	Table 3.3.2-2	Trip Setpoint values.	TRM	10 CFR 50.59	2
3.3.6.2-LA.2	3.3.2 Actions footnotes *, **, and ***	The details relating to placing channels in trip (e.g, if tripping causes Trip Function to occur, tripping trip system with the most inoperable channels).	Bases	Bases Control Program in ITS Chapter 5	3

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3.3.6.2-LA.3	4.3.2.2	The detail relating to methods (simulated automatic operation) for performing the LOGIC SYSTEM FUNCTIONAL TESTS.	Bases	Bases Control Program in ITS Chapter 5	3
3.3.6.2-LA.4	Table 3.3.2-1, including footnotes (c) and (e)	System design and operational details (i.e., the Manual Initiation Functions isolate the inboard and outboard valves and specific valves and systems affected).	Bases	Bases Control Program in ITS Chapter 5	1
3.3.7.1, CRAF System Instrumentation					
3.3.7.1-LA.1	Table 3.3.7.1-1	The measurement range of the Main Control Room Atmospheric Control System Radiation Monitoring System channels.	UFSAR	10 CFR 50.59	1
3.3.8.1, Loss of Power Instrumentation					
3.3.8.1-LA.1	Table 3.3.3-2	Trip Setpoint values.	TRM	10 CFR 50.59	2
3.3.8.1-LA.2	4.3.3.2	The detail relating to methods (simulated automatic operation) for performing the LOGIC SYSTEM FUNCTIONAL TESTS.	Bases	Bases Control Program in ITS Chapter 5	3
3.3.8.1-LA.3	Tables 3.3.3-1 and 3.3.3-2	Details relating to system design (the total number of channels provided in the design, the number of channels required to generate a trip and the types of relays).	Bases	Bases Control Program in ITS Chapter 5	1
3.3.8.2, RPS Electric Power Monitoring					

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3.3.8.2-LA.1	4.8.3.4.b	The details relating to methods (simulated automatic operation of the RPS overvoltage, undervoltage, and underfrequency protective relays, tripping logic, and output circuit breakers) for performing the system functional test.	Bases	Bases Control Program in ITS Chapter 5	3
Current Specification 3/4.3.7.3, Meteorological Monitoring Instrumentation					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.3.7.11, Explosive Gas Monitoring Instrumentation					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.3.7.12, Loose Part Detection System					
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.4 - REACTOR COOLANT SYSTEM

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.4.1, Recirculation Loops Operating					
3.4.1-LA.1	3.4.1.1 Action a.1.a)	The details relating to operational controls of the flow control system during single recirculation loop operation.	UFSAR	10 CFR 50.59	2
3.4.1-LA.2	3.4.1.5 Actions a.1 and a.2.a)	Requirement to "immediately initiate action to" reduce THERMAL POWER to exit Region I when one or both recirculation loops are in operation and to “immediately” reduce CORE THERMAL POWER to reduce power below 36% RTP when no recirculation loops are in operation is relocated in the form of a discussion that "prompt action should be taken" to exit Region I of CTS Figure 3.4.1.5-1 (ITS Figure 3.4.1-1) or reduce THERMAL POWER to below 36% RTP, as applicable.	Bases	Bases Control Program in ITS Chapter 5	3
3.4.1-LA.3	3.4.1.5 Actions b.1 and b.2	The details on how to restore APRM and LPRM noise levels within limits. Requirement to “immediately initiate corrective action” to restore APRM and LPRM flux noise levels to within limits is relocated in the form of the discussion that “prompt action should be taken” to satisfy the requirements of the LCO.	Bases	Bases Control Program in ITS Chapter 5	3
3.4.1-LA.4	4.4.1.5.1.b	Requirement to “initiate the surveillance within 15 minutes” after entering Region II of CTS Figure 3.4.1.5-1 or completing an increase of at least 5% of RATED THERMAL POWER is relocated in the form of the discussion that “prompt action should be taken” to verify APRM and LPRM flux noise levels.	Bases	Bases Control Program in ITS Chapter 5	3

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3.4.1-LA.5	4.4.1.5 footnote #	The details concerning which LPRM detectors to monitor (i.e., detector levels A and C of one LPRM string per core octant plus detectors A and C of one LPRM string in the center region of the core should be monitored).	Bases	Bases Control Program in ITS Chapter 5	3
3.4.1-LA.6	3.4.1.1 Action a.1.c)	The detail of the actual MCPR correction factor for the MCPR operating limit for single loop operation ("0.01").	COLR	COLR change control process described in Chapter 5 of the ITS	3
3.4.2, Flow Control Valves					
NONE	NONE	NONE	NONE	NONE	NONE
3.4.3, Jet Pumps					
3.4.3-LA.1	4.4.1.2.1, 4.4.1.2.1.a, 4.4.1.2.2.a	Details of the methods for determining established flow control valve position (determining the loop flow characteristics of the flow control valve for two recirculation loop operation versus single recirculation loop operation and ensuring balanced recirculation loop flows when both recirculation loops are operating).	Bases	Bases Control Program in ITS Chapter 5	3
3.4.3-LA.2	4.4.1.2.1.b, 4.4.1.2.2.b	Details of the methods of deriving total core flow values during single loop operation (i.e., the established THERMAL POWER-core flow relationship or established core plate differential pressure-core flow relationship for two recirculation loop operation).	Bases	Bases Control Program in ITS Chapter 5	3
3.4.4, Safety/Relief Valves					

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3.4.4-LA.1	3.4.2 footnote *	The details relating to lift setting pressure of the safety/relief valves (the lift setting pressure shall correspond to ambient conditions of the valves at nominal operating temperatures and pressures).	Bases	Bases Control Program in ITS Chapter 5	1
3.4.5, RCS Operational Leakage					
NONE	NONE	NONE	NONE	NONE	NONE
3.4.6, RCS Pressure Isolation Valve Leakage					
3.4.6-LA.1	Table 3.4.3.2-1	The list of pressure isolation valves (PIVs).	TRM	10 CFR 50.59	1
3.4.6-LA.2	4.4.3.2.2.a.1	Requirement to leak test each PIV “at least once per 18 months.”	IST Program	IST Program in ITS Chapter 5	3
3.4.6-LC.1	3.4.3.2 Action d, 4.4.3.2.2.b	High/low pressure interface valve leakage pressure monitors, along with supporting Actions and Surveillances.	TRM	10 CFR 50.59	3
3.4.7, RCS Leakage Detection Instrumentation					
NONE	NONE	NONE	NONE	NONE	NONE
3.4.8, RCS Specific Activity					
3.4.8-LA.1	Table 4.4.5-1 Item 5	Offgas isotopic analysis for xenon and krypton.	TRM	10 CFR 50.59	3

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3.4.9, RHR Shutdown Cooling System - Hot Shutdown					
3.4.9-LA.1	LCO 3.4.9.1	The details of what constitutes an OPERABLE RHR shutdown cooling subsystem (i.e., each subsystem consists of one OPERABLE RHR pump and heat exchanger).	Bases	Bases Control Program in ITS Chapter 5	1
3.4.9-LA.2	4.4.9.1	The detail of the method of verifying operation of the RHR shutdown cooling subsystem (circulating reactor coolant).	Bases	Bases Control Program in ITS Chapter 5	3
3.4.10, RHR Shutdown Cooling System - Cold Shutdown					
3.4.10-LA.1	LCO 3.4.9.2	The details of what constitutes an OPERABLE RHR shutdown cooling subsystem (i.e., each subsystem consists of one OPERABLE RHR pump and heat exchanger).	Bases	Bases Control Program in ITS Chapter 5	1
3.4.10-LA.2	4.4.9.2	The detail of the method of verifying operation of the RHR shutdown cooling subsystem (circulating reactor coolant).	Bases	Bases Control Program in ITS Chapter 5	3
3.4.11, RCS Pressure and Temperature Limits					
3.4.11-LA.1	4.4.6.1.1, 4.4.6.1.2	The specific requirements that operation be to the right of the limits lines of Figures 3.4.6.1-1 and 3.4.6.1-1a.	Bases	Bases Control Program in ITS Chapter 5	3
3.4.11-LA.2	3.4.6.1 Action	The detail to perform an engineering evaluation to determine the effects of the out-of-limit condition on the structural integrity of the reactor coolant system.	Bases	Bases Control Program in ITS Chapter 5	3

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3.4.11-LA.3	3.4.1.4	The detail on where the temperature is monitored (e.g., space or line) to determine the temperature differential between the reactor pressure vessel steam space coolant and the bottom head drain.	Bases	Bases Control Program in ITS Chapter 5	3
3.4.11-LA.4	LCO 3.4.1.4.b, 3.4.1.4 Action, 4.4.1.4	The details and its associated Action relating to operational limits (maximum operating loop flow rate) during a return to two recirculation pump operation from single recirculation loop operation.	UFSAR	10 CFR 50.59	1
3.4.12, Reactor Steam Dome Pressure					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.4.8, Structural Integrity					
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.5 - ECCS AND RCIC SYSTEM

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.5.1, ECCS-Operating					
3.5.1-LA.1	LCO 3.5.1	The details relating to ECCS OPERABILITY (i.e., that the ECCS subsystems shall have flow paths capable of taking suction from the suppression chamber and transferring water to the reactor vessel).	Bases	Bases Control Program in ITS Chapter 5	1
3.5.1-LA.2	4.5.1.a.1, 4.5.1.c.1, 4.5.1.d.2.b	The details relating to methods for performing Surveillances (i.e., venting at the high point vent, verifying actuation of the system throughout its emergency operating sequence, including each automatic valve actuating to the correct position, and verifying proper operation of the ADS valves).	Bases	Bases Control Program in ITS Chapter 5	2
3.5.1-LA.3	3.5.1 Action j, 4.5.1.a.4, 4.5.1.c.4	The requirements for corner room watertight doors to be Operable (i.e., closed and capable of repelling water).	TRM	10 CFR 50.59	3
3.5.2, ECCS-Shutdown					
3.5.2-LA.1	LCO 3.5.2	The details of what constitutes an OPERABLE ECCS subsystem.	Bases	Bases Control Program in ITS Chapter 5	1
3.5.2-LA.2	LCO 3.5.3.b	The suppression pool volume which corresponds to the level limit.	Bases	Bases Control Program in ITS Chapter 5	1

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3.5.2-LA.3	LCO 3.5.3.b footnote **, 4.5.3.1.a.2 footnote **	The plant elevation level which corresponds to the suppression pool level limit.	Bases	Bases Control Program in ITS Chapter 5	1
3.5.3, RCIC System					
3.5.3-LA.1	LCO 3.7.3	The details of CTS 3.7.3 relating to system OPERABILITY (in this case that the RCIC System shall have a flow path capable of taking suction from the suppression pool and transferring water to the reactor pressure vessel).	Bases	Bases Control Program in ITS Chapter 5	1
3.5.3-LA.2	4.7.3.a.1, 4.7.3.a.4, 4.7.3.c.1	The details relating to methods for performing Surveillances (i.e., by venting from the high point vent, verifying that the RCIC pump controller is in the correct position, and verifying that each automatic valve in the flow path actuates to the proper position during the actuation test).	Bases	Bases Control Program in ITS Chapter 5	3
3.5.3-LA.3	4.7.3.c.2	The 135 psig minimum pressure for performing the RCIC System flow test is relocated in the form of a discussion describing when adequate pressure is available to perform the test.	Bases	Bases Control Program in ITS Chapter 5	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.6 - CONTAINMENT SYSTEMS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.6.1.1, Primary Containment					
3.6.1.1 - LA.1	4.6.2.1.d	The initial differential pressure (1.5 psi) of the drywell-to-suppression chamber bypass leak test.	Bases	Bases Control Program in ITS Chapter 5	3
3.6.1.2, Primary Containment Air Lock					
NONE	NONE	NONE	NONE	NONE	NONE
3.6.1.3, Primary Containment Isolation Valves					
3.6.1.3 - LA.1	4.6.3.3	The requirement to stroke time test the power operated, non-automatic, PCIVs.	IST Program	10 CFR 50.59 and 10 CFR 50.55a	3
3.6.1.3 - LA.2	4.6.3.5.b	Requirements in CTS 4.6.3.5.b concerning the replacement charges for the traversing in-core probe (TIP) explosive valves (i.e., replacement charge shall be from the same batch or from another batch that has had one charge fired, and no charge shall remain in use past its shelf-life and operating-life).	Bases	Bases Control Program in ITS Chapter 5	3

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3.6.1.3-L.7	4.6.3.2	The requirement to perform CTS 4.6.3.2 during COLD SHUTDOWN or REFUELING has been deleted. The proposed Surveillance (for a functional test of each primary containment isolation valve) does not include the restriction on plant conditions. Some isolation valves could be adequately tested in other than Cold Shutdown or Refueling, without jeopardizing safe plant operations. This information is encompassed by the Bases discussion justifying the frequency for ITS 3.6.1.3.7.	Bases	Bases Control Program in ITS Chapter 5	3
3.6.1.4, Drywell and Suppression Chamber Pressure					
NONE	NONE	NONE	NONE	NONE	NONE
3.6.1.5, Drywell Air Temperature					
3.6.1.5-LA.1	4.6.1.7	The method for performing the drywell average air temperature Surveillance (i.e., average of the temperatures of the operating return air plenum upstream of the primary containment ventilation heat exchanger coil and cabinet at various elevations and azimuths).	Bases	Bases Control Program in ITS Chapter 5	3
3.6.1.6, Suppression Chamber-to-Drywell Vacuum Breakers					
3.6.1.6-LA.1	4.6.4.1.b.2	The detail that the opening setpoint is verified from the closed position.	Bases	Bases Control Program in ITS Chapter 5	3

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3.6.2.1, Suppression Pool Average Temperature					
NONE	NONE	NONE	NONE	NONE	NONE
3.6.2.2, Suppression Pool Water Level					
3.6.2.2-LA.1	3.6.2.1.a.1 including footnote **, 3.5.3.a including footnote **, 4.5.3.1.a.1 footnote **	The suppression pool volumes which correspond to the level limits and the reference as to how the level limits correspond to plant elevation.	Bases	Bases Control Program in ITS Chapter 5	1
3.6.2.3, RHR Suppression Pool Cooling					
3.6.2.3-LA.1	3.6.2.3	The details relating to system OPERABILITY (in this case the suppression pool cooling function is designated as two "independent" loops, each with a pump and flow path).	Bases	Bases Control Program in ITS Chapter 5	1
3.6.2.4, RHR Suppression Pool Spray					
3.6.2.4-LA.1	3.6.2.2	The details relating to system OPERABILITY (in this case the suppression pool spray function is designated as two "independent" loops, each with a pump and flow path).	Bases	Bases Control Program in ITS Chapter 5	1

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3.6.3.1, Primary Containment Hydrogen Recombiners					
3.6.3.1-LA.1	3.6.6.1	The detail relating to system design (i.e., that the recombiners are "independent").	Bases	Bases Control Program in ITS Chapter 5	1
3.6.3.1-LA.2	4.6.6.1.b, 4.6.6.1.c.2	Details of the methods for performing a system functional test (i.e., the current in each heater phase is within limits, and the increase in reaction chamber gas temperature within 2 hours) and the heater resistance to ground test (perform test within 30 minutes after the system functional test and the heater resistance to ground limit).	Bases	Bases Control Program in ITS Chapter 5	3
3.6.3.1-LA.3	4.6.6.1.a	CTS 4.6.6.1.a performed every 92 days is a functional test of the hydrogen recombiner which checks the recombiner flow control and recirculation valves.	ITS Program	10 CFR 50.59 and 10 CFR 50.55	3
3.6.3.2, Primary Containment Oxygen Concentration					
NONE	NONE	NONE	NONE	NONE	NONE
3.6.4.1, Secondary Containment					
NONE	NONE	NONE	NONE	NONE	NONE
3.6.4.2, Secondary Containment Isolation Valves					

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3.6.4.2-LA.1	3/4.6.5.2, Table 3.6.5.2-1	The list of secondary containment isolation dampers referenced in CTS 3/4.6.5.2 and appearing in CTS Table 3.6.5.2-1 with their isolation times, are proposed to be relocated to the Technical Requirements Manual consistent with Generic Letter 91-08. In addition, due to the relocation, the name of the isolation dampers has been generically changed to Secondary Containment Isolation Valves (SCIVs).	TRM	10 CFR 50.59	1
3.6.4.3, Standby Gas Treatment System					
3.6.4.3-LA.1	3.6.5.3	The detail relating to system design (i.e., that the SGT subsystems are "independent")	Bases	Bases Control Program in ITS Chapter 5	1
3.6.4.3-LA.2	4.6.5.3.a	Details of the methods for performing the standby gas treatment subsystem 31 day operating Surveillance (by initiating, from the control room, flow through the HEPA filters and charcoal adsorbers).	Bases	Bases Control Program in ITS Chapter 5	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.7 - PLANT SYSTEMS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.7.1, Residual Heat Removal Service Water System					
3.7.1-LA.1	LCO 3.7.1.1	The details relating to system OPERABILITY (that the RHRSW subsystems shall be independent and that each subsystem shall have two RHRSW pumps capable of taking suction from the CSCS water tunnel and transferring the water to the associated RHR heat exchanger).	Bases	Bases Control Program in ITS Chapter 5	1
3.7.1-LA.2	3/4.7.1.1	LCO requirements, Actions, and Surveillance Requirements for the RHRSW System when in MODES 4 and 5.	TRM	10 CFR 50.59	3
3.7.2, Diesel Generator Cooling Water System					
3.7.2-LA.1	LCO 3.7.1.2	The details relating to system OPERABILITY (that the DGCW subsystems will be independent and each subsystem shall have one OPERABLE DGCW pump, and an OPERABLE flow path capable of taking suction from the CSCS water tunnel and transferring water to the associated diesel generator).	Bases	Bases Control Program in ITS Chapter 5	1
3.7.2-LA.2	3/4.7.1.2	LCO requirements, Actions, and Surveillance Requirements for the DGCW System when in MODES or conditions other than MODE 1, 2, or 3.	TRM	10 CFR 50.59	3
3.7.2-LA.3	4.7.1.2.b	The details regarding the specific start signals (from the associated DG or from the LPCS pump) to be used during the DGCW automatic start Surveillance.	Bases	Bases Control Program in ITS Chapter 5	3
3.7.3, Ultimate Heat Sink					

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3.7.3-LA.1	3/4.7.1.3	LCO requirements, Actions, and Surveillance Requirements for the Ultimate Heat Sink when in MODES or conditions other than MODE 1, 2, or 3.	TRM	10 CFR 50.59	3
3.7.3-LA.2	4.7.1.3.a	Details of the methods for determining the level of sediment deposition in the CSCS pond (by a series of sounding cross-sections compared to as-built soundings).	Bases	Bases Control Program in ITS Chapter 5	3
3.7.4, Control Room Area Filtration System					
3.7.4-LA.1	LCO 3.7.2	The detail relating to system design (that the CRAF subsystems are "independent").	Bases	Bases Control Program in ITS Chapter 5	1
3.7.4-LA.2	4.7.2.d.2.a)	Requirement to verify that the CRAF System automatically switches to the pressurization mode of operation on detection of smoke in an outside air intake.	TRM	10 CFR 50.59	3
3.7.4-LA.3	4.7.2.d.2	Details regarding the methodology for performing a surveillance to verify the ability of each CRAF System to maintain a positive pressure in the control room and auxiliary electric equipment rooms relative to the adjacent areas during emergency train operation at a specified flow rate (by manually initiating flow through the control room and auxiliary equipment room recirculation filters).	Bases	Bases Control Program in ITS Chapter 5	3
3.7.4-LA.4	4.7.2.d.2	Details of the methods for performing the CRAF System actuation test (the source of the signal used for automatic actuation and that the CRAF subsystems automatically switch to the pressurization mode of operation)	Bases	Bases Control Program in ITS Chapter 5	3

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3.7.5, Control Room Area Ventilation Air Conditioning System					
NONE	NONE	NONE	NONE	NONE	NONE
3.7.6, Main Condenser Offgas					
3.7.6-LA.1	4.11.2.2.1	The requirement to continuously monitor radioactivity rate of noble gases prior to the holdup line.	ODCM	ODCM Control Process in ITS Chapter 5	3
3.7.6-LA.2	4.11.2.2.2	Details defining the methods for performing this Surveillance, the location of the sample, and method for determining when an increase has occurred.	Bases	Bases Control Program in ITS Chapter 5	3
3.7.7, Main Turbine Bypass System					
3.7.7-LA.1	4.7.10.b.1	The details relating to methods of performing the main turbine bypass system functional test (simulated automatic actuation and verifying each automatic valve actuates to its correct position).	Bases	Bases Control Program in ITS Chapter 5	3
3.7.7-LA.2	4.7.10.b.2	Details of the actual TURBINE BYPASS SYSTEM RESPONSE TIME.	TRM	10 CFR 50.59	1
3.7.8, Spent Fuel Storage Pool Water Level					
3.7.8-LA.1	3.9.9 Action	The requirement to suspend crane operations with loads in the spent fuel storage pool area when the spent fuel storage pool water level is not within the limit.	UFSAR	10 CFR 50.59	3
3.7.8-LA.2	3.9.9 Action	Details of the methods for suspending movement of fuel assemblies (after placing the fuel assemblies in a safe condition).	Bases	Bases Control Program in ITS Chapter 5	3

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Current Specification 3/4.7.4, Sealed Source Contamination					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.7.7, Area Temperature Monitoring					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.7.8, Structural Integrity of Class 1 Structures					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.7.9, Snubbers					
None- LA.1ONE	3/4.7.9	Snubber inspection and testing requirements.	TRM	10 CFR 50.59	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.8 - ELECTRICAL POWER SYSTEMS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.8.1, AC Sources - Operating					
3.8.1-LA.1	LCO 3.8.1.1.a, LCO 3.8.1.1.b, LCO 3.8.1.1.b.3, LCO 3.8.2.1.d	The details relating to system design and OPERABILITY (i.e., that the offsite circuits are "physically independent," the DGs are "separate and independent," the nomenclature of the DGs, that each DG has "a separate fuel transfer pump," and some components of the opposite unit's offsite circuit).	Bases	Bases Control Program in ITS Chapter 5	1
3.8.1-LA.2	N/A	Not used.	N/A	N/A	N/A
3.8.1-LA.3	3.8.1.1 Action g	Details regarding the subsystems required to be declared inoperable.	Bases	Bases Control Program in ITS Chapter 5	3

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3.8.1-LA.4	* footnote to CTS 4.8.1.1.2, 4.8.1.1.2. d.4, 4.8.1.1.2. d.5, 4.8.1.1.2. d.6, 4.8.1.1.2. d.8, and 4.8.1.1.2. e, 4.8.1.1.2. a.4 footnote **	The detail that the engine prelube requirements are "as recommended by the manufacturer," and the reason for performing the diesel engine warmup and loading procedures (i.e., in order to minimize mechanical stress and wear on the DGs caused by fast starting of the DGs).	Bases	Bases Control Program in ITS Chapter 5	3
3.8.1-LA.5	4.8.1.1.2. d.2	The specific kilowatt value of the single largest post-accident load for the single load rejection Surveillance Requirement.	Bases	Bases Control Program in ITS Chapter 5	1
3.8.1-LA.6		Not used.			
3.8.1-LA.7	4.8.1.1.2. d.9, LCO 3.8.2.1.d	The specific load value for the auto-connected loads and the specific bus designation (141Y and 241Y) associated with the offsite circuit path.	UFSAR	10 CFR 50.59	1
3.8.1-LA.8	LCO 3.8.1.1.b footnote *	The details that the control circuit for the unit cross-tie breakers between buses 142Y and 242Y are temporarily modified to allow the breakers to be closed.	Bases	Bases Control Program in ITS Chapter 5	3
3.8.2, AC Sources - Shutdown					

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3.8.2-LA.1	LCO 3.8.1.2.b.3	The detail relating to system design and OPERABILITY (i.e., that each DG has a fuel oil transfer pump).	Bases	Bases Control Program in ITS Chapter 5	1
3.8.3, Diesel Fuel Oil and Starting Air					
3.8.3-LA.1	4.8.1.1.2.f	The 10 year surveillance to drain, remove sediment and clean each fuel oil tank and to perform a pressure test on the DG fuel oil piping system.	TRM	10 CFR 50.59	3
3.8.4, DC Sources - Operating					
3.8.4-LA.1	LCO 3.8.2.3	Details relating to system OPERABILITY (what constitutes a DC Source subsystem) relocated to Bases and the actual battery identification numbers are relocated to the UFSAR.	Bases UFSAR	Bases Control Program in ITS Chapter 5 10 CFR 50.59	1 1
3.8.4-LA.2	4.8.2.3.2.d	The details of the DC loads and the licensed service duration for the battery service test.	UFSAR	10 CFR 50.59	1
3.8.4-LA.3	4.8.2.3.2.f	Specific limits on battery degradation and guidance regarding the intent of the term "degradation."	Bases	Bases Control Program in ITS Chapter 5	3
3.8.5, DC Sources - Shutdown					
3.8.5-LA.1	LCO 3.8.2.4	Details relating to system OPERABILITY (what constitutes a DC Source subsystem) relocated to Bases and the actual battery identification numbers are relocated to the UFSAR.	Bases UFSAR	Bases Control Program in ITS Chapter 5 10 CFR 50.59	1 1

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3.8.6, Battery Cell Parameters					
3.8.6-LA.1	4.8.2.3.2. b.3	Details relating to the plant specific determination of "representative" are relocated, since IEEE-450 requires only "representative" cells be measured for temperature and every sixth cell (which is approximately 10 cells for the LaSalle 1 and 2 125V batteries) is provided as a suggestion only in IEEE-450.	Bases	Bases Control Program in ITS Chapter 5	3
3.8.7, Distribution Systems - Operating					
3.8.7-LA.1	3.8.2.1 including Actions a, b, and c, 3.8.2.3 including Actions a, b, c, 4.8.2.1, 4.8.2.3.1, 4.7.3.d	The details relating to system design (the list of buses) and OPERABILITY (the buses are required to be energized).	Bases	Bases Control Program in ITS Chapter 5	1
3.8.7-LA.2	4.8.2.3.1, 4.7.3.d.1. a)	The details for verifying the required Distribution Systems are OPERABLE (the voltage limit and that the charger and battery provide the power).	UFSAR	10 CFR 50.59	1
3.8.8, Distribution Systems - Shutdown					

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3.8.8-LA.1	3.8.2.2 including Actions a, b, and c, 3.8.4.2 including Actions a, b, and d, 4.8.2.2, 4.8.2.4.1	The details relating to system design (the list of buses) and OPERABILITY (the buses are required to be energized).	Bases	Bases Control Program in ITS Chapter 5	1
3.8.8-LA.1	4.8.2.4.1	The detail (voltage limit) for verifying the required DC Distribution subsystems are OPERABLE.	UFSAR	10 CFR 50.59	1
Current Specification 3/4.8.3.1, AC Circuits Inside Primary Containment					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.8.3.2, Primary Containment Penetration Conductor Overcurrent Protective Devices					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.8.3.3, Motor Operated Valves Thermal Overload Protection					
None - LA.1	3/4.8.3.3	The requirements of CTS 3/4.8.3.3 for bypass devices that are associated with thermal overload protection of the valves listed in CTS Table 3.8.3.3-1.	TRM	10 CFR 50.59	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.9 - REFUELING OPERATIONS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.9.1, Refueling Equipment Interlocks					
NONE	NONE	NONE	NONE	NONE	NONE
3.9.2, Refuel Position One-Rod-Out Interlock					
NONE	NONE	NONE	NONE	NONE	NONE
3.9.3, Control Rod Position					
NONE	NONE	NONE	NONE	NONE	NONE
3.9.4, Control Rod Position Indication					
NONE	NONE	NONE	NONE	NONE	NONE
3.9.5, Control Rod OPERABILITY - Refueling					
NONE	NONE	NONE	NONE	NONE	NONE
3.9.6, RPV Water Level - Irradiated Fuel					
3.9.6-LA.1	3.9.8 Action	Details of the methods for suspending movement of fuel assemblies (after placing the fuel assemblies in a safe condition).	Bases	Bases Control Program in ITS Chapter 5	3
3.9.7, RPV Water Level - New Fuel or Control Rods					

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.9 - REFUELING OPERATIONS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.9.7-LA.1	3.9.8 Action	Details of the methods for suspending movement of fuel assemblies and control rods (after placing the fuel assemblies and control rods in a safe condition).	Bases	Bases Control Program in ITS Chapter 5	3
3.9.8, Residual Heat Removal - High Water Level					
3.9.8-LA.1	LCO 3.9.11.1	The details in CTS 3.9.11.1.a and 3.9.11.1.b of what constitutes an OPERABLE RHR shutdown cooling subsystem (i.e., each subsystem consists of one OPERABLE RHR pump and heat exchanger).	Bases	Bases Control Program in ITS Chapter 5	3
3.9.8-LA.2	4.9.11.1	The detail of the method of verifying operation of the RHR shutdown cooling subsystem (circulating reactor coolant).	Bases	Bases Control Program in ITS Chapter 5	3
3.9.9, Residual Heat Removal - Low Water Level					
3.9.9-LA.1	LCO 3.9.11.2	The details in CTS 3.9.11.2.a and 3.9.11.2.b of what constitutes an OPERABLE RHR shutdown cooling subsystem (i.e., each subsystem consists of one OPERABLE RHR pump and heat exchanger).	Bases	Bases Control Program in ITS Chapter 5	3
3.9.9-LA.2	4.9.11.2	The detail of the method of verifying operation of the RHR shutdown cooling subsystem (circulating reactor coolant).	Bases	Bases Control Program in ITS Chapter 5	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.9 - REFUELING OPERATIONS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
Current Specification 3/4.9.4, Decay Time					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.9.5, Communications					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.9.6, Crane and Hoist					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.9.7, Crane Travel					
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.10 - SPECIAL OPERATIONS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.10.1, Reactor Mode Switch Interlock Testing					
3.10.1-LA.1	Table 1.2 footnote #, 3.9.1 footnote ##	The method used to verify control rods remain fully inserted (by verification using a second licensed operator or other technically qualified member of the unit technical staff).	Bases	Bases Control Program in ITS Chapter 5	3
3.10.2, Single Control Rod Withdrawal - Hot Shutdown					
NONE	NONE	NONE	NONE	NONE	NONE
3.10.3, Single Control Rod Withdrawal - Cold Shutdown					
3.10.3-LA.1	LCO 3.9.10.1.d 4.9.10.1.d	The details of the recommended procedures for disarming control rods (i.e., electrically or hydraulically).	Bases	Bases Control Program in ITS Chapter 5	3
3.10.4, Single Control Rod Drive Removal - Refueling					
3.10.4-LA.1	LCO 3.9.10.1.d 4.9.10.1.d	The details of the recommended procedures for disarming control rods (i.e., electrically or hydraulically).	Bases	Bases Control Program in ITS Chapter 5	3
3.10.5, Multiple Control Rod Withdrawal - Refueling					
NONE	NONE	NONE	NONE	NONE	NONE
3.10.6, Control Rod Testing - Operating					

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.10 - SPECIAL OPERATIONS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
NONE	NONE	NONE	NONE	NONE	NONE
3.10.7, SDM Test - Refueling					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.10.1, Primary Containment Integrity					
None-M.1	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.10.5, Oxygen Concentration					
None-M.1	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.10.6, Training Startups					
None-M.1	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.10.8, Suppression Chamber Water Temperature (Unit 1 only)					
None-A.1	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
CHAPTER 4.0 - DESIGN FEATURES

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
4.0-LA.1	5.2.1, 5.2.2, 5.2.3, 5.4, 5.4.1, 5.4.2	Primary containment configuration and design details, primary containment design temperatures and pressures, secondary containment design details, and the Reactor Coolant System design pressure and temperature and volume.	UFSAR	10 CFR 50.59	1
4.0-LA.2	5.3.2	The nominal active control rod assembly absorber length.	UFSAR	10 CFR 50.59	1

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
CHAPTER 5.0 - ADMINISTRATIVE CONTROLS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
5.1, Responsibility					
5.1-LA.1	6.1.A.2	Replaces the specific title "Plant Manager" with the generic title "station manager" and relocates the specific title.	QA Manual	10 CFR 50.54	3
5.1-LA.2	6.1.B	The requirement that delineates the responsibility of the Shift Manager for directing and commanding the overall operation of the facility on his shift.	UFSAR	10 CFR 50.59	3
5.2, Organization					
5.2-LA.1	6.1.A.1, 6.1.A.2, 6.1.A.3, 6.1.C.6, 6.1.C.8	Replaces the specific title "Chief Nuclear Officer" with the generic term "a corporate officer." Replaces the specific title "Plant Manager" with the generic title "station manager." Replaces the specific title “Shift Manager” with the generic term “shift manager.” Replaces the specific titles "Operations Manager” and "Shift Operations Supervisor" with the generic titles "operations manager" and "shift operations supervisor." The specific titles are relocated.	QA Manual	10 CFR 50.54	3
5.2-LA.2	6.1.C.1, 6.1.C.3, Figure 6.1-3 footnote (b)	Details that require all CORE ALTERATIONS to be observed and directly supervised by either a licensed SRO or SRO limited to fuel handling, who has no other concurrent responsibilities during this operation and the details concerning location of RO’s and SRO’s.	UFSAR	10 CFR 50.59	3
5.2-LA.3	6.1.C.5, including footnote #	The Independent Safety Engineering Group requirements.	QA Manual	10 CFR 50.54	3

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5.2-LA.4	Figure 6.1-3, including footnotes (a) and (b)	Details of the minimum shift crew requirements.	UFSAR	10 CFR 50.59	3
5.3, Unit Staff Qualifications					
5.3-LA.1	6.1.D, 6.1.D.1, 6.1.D.2	Replaces the specific titles "Health Physics Supervisor" and "Radiation Protection Technician" with the generic titles "radiation protection manager" and "radiation protection technician." The specific titles are relocated.	QA Manual	10 CFR 50.54	3
5.3-LA.2	6.1.D	The requirement that the qualifications of the station management shall meet those described in ANSI N18.1-1971.	QA Manual	10 CFR 50.54	3
5.4, Procedures					
5.4-LA.1	6.2.A.e	The requirement that written procedures for the PROCESS CONTROL PROGRAM (PCP) be established, implemented, and maintained.	UFSAR	10 CFR 50.59	3
5.4-LA.2	6.2.C, 6.2.D	The details of procedure reviews and approvals including temporary changes.	QA Manual	10 CFR 50.54	3
5.5, Programs and Manuals					
5.5-LA.1	6.2.F.2	The details contained in CTS 6.2.F.2, "In-Plant Radiation Monitoring."	UFSAR	10 CFR 50.59	3

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5.5-LA.2	6.2.F.5	The details contained in CTS 6.2.F.5, "Radiological Environmental Monitoring Program."	ODCM	ODCM Control Process in ITS Chapter 5	3
5.5-LA.3	6.2.F.6, 6.8.2.b	Requirement that the changes to the Inservice Inspection Program for Post Tensioning Tendons must be reviewed and approved by the Onsite Review and Investigative Function and the requirement that the ODCM must be reviewed and accepted by the Onsite Review and Investigative Function, prior to implementation and to document this review and acceptance.	QA Manual	10 CFR 50.54	3
5.5-LA.4	4.0.5, 4.0.5.a, 4.0.5.b, 4.0.5.c, 4.0.5.d, 4.0.5.f	Details of the Inservice Inspection (ISI) Program are relocated; and since the Inservice Testing Program is the only requirement remaining, the reference to ASME Code Class 1, 2, and 3 “components” has been changed to “pumps and valves” for clarity.	ISI Program	10 CFR 50.55a	3
5.5-LA.5	4.0.5.a	Details of the Inservice Testing Program.	IST Program	10 CFR 50.55a	3
5.5-LA.6	3/4.11.1.1 , 3/4.11.2.1	The details for implementing the liquid holdup tank and explosive gas mixture requirements.	TRM	10 CFR 50.59	3
5.5-LA.7	5.7, Table 5.7.1-1	Details of the components governed by the Component Cyclic or Transient Limit Specification.	UFSAR	10 CFR 50.59	3
5.5-LA.8	6.8.2.b	Replaces the specific title "Plant Manager" with the generic title "station manager." The specific title is relocated.	QA Manual	10 CFR 50.54	3
5.6, Reporting Requirements					

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5.6-LA.1	6.6.A.1	The details associated with the Startup Report.	TRM	10 CFR 50.59	3
5.6-LA.2	6.6.A.6.a.(2)	The details associated with the MCPR Specification (i.e.,scram time dependent MCPR limits, power and flow dependent MCPR limits, and effects of analyzed equipment out of service).	Bases	Bases Control Program in ITS Chapter 5	1
5.6-LA.3	6.6.A.6.b	The details of the actual topical reports document date, revision number, volume, supplement and company.	COLR	COLR Control Process In ITS Chapter 5	3
5.7, High Radiation Area					
5.7-LA.1	6.1.1.1, 6.1.1.3	Replaces the specific title “Health Physicist” with the generic term “radiation protection manager.” Replaces the specific title “Shift Manager” with the generic term “shift manager.” The specific titles are relocated.	QA Manual	10 CFR 50.54	3
Current Specification 6.1.E/F, Training					
None-LA.1	6.1.E, 6.1.F	The details on training and replacement training of station personnel.	UFSAR	10 CFR 50.59	3
Current Specification 6.2.B, Radiation Protection Program					
None-LA.1	6.2.B	The details on Radiation Protection Program procedures.	UFSAR	10 CFR 50.59	3
Current Specification 6.3, Reportable Event Action					

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None-LA.1	6.3.b	The details related to Reportable Event reviews (by the Onsite Review and Investigative Function).	QA Manual	10 CFR 50.54	3
Current Specification 6.4, Safety Limit Violation					
None-LA.1	6.4	The requirements for: 1) notification of the Site Vice President and Director of Safety Review in the event of a Safety Limit violation; and 2) the Onsite and Offsite Review Investigative Functions to review the Safety Limit Violation Report.	QA Manual	10 CFR 50.54	3
Current Specification 6.5, Plant Operating Records					
None-LA.1	6.5	The details contained in the Plant Operating Records Specification.	QA Manual	10 CFR 50.54	3
Current Specification 6.7, Process Control Program					
None-LA.1	6.7	The details contained in the Process Control Program Specification and the definition of PROCESS CONTROL PROGRAM.	UFSAR	10 CFR 50.59	3
Current Specification 6.9, Major Changes to Radioactive Waste Treatment System					

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None-LA.1	6.9	The requirements regarding major changes to the radioactive waste treatment systems (i.e., a description of the content of the report to be submitted to the NRC regarding the major changes and a requirement that the major changes become effective upon review and acceptance by the Onsite Review and Investigative Function).	ODCM	ODCM Control Process in ITS Chapter 5	3

CHANGE TYPE

1. Details of system design and system description including design limits
2. Description of system operation
3. Procedural details for meeting TS requirement, relocated reporting requirements and relocated specification requirements.

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