TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
CHAPTER 1.0 - USE AND APPLICATION

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
1.0-LA.1	1.0	Moves the definition of FRACTION OF RATED POWER (FRTP) and TRANSIENT LINEAR HEAT GENERATION RATE (TLHGR) to ITS 3.2.4 Bases.	Bases	Bases Control Process in ITS Chapter 5.0	1
1.0-LA.2	1.0	Moves the definition of STEADY STATE LINEAR HEAT GENERATION RATE (SLHGR) to ITS 3.2.3 Bases.	Bases	Bases Control Process in ITS Chapter 5.0	1
1.0-LA.3	1.0	Moves items a, b, c, and f from the PRIMARY CONTAINMENT INTEGRITY definition to the ITS 3.6.1.1 Bases and items b and e from the CTS SECONDARY CONTAINMENT INTEGRITY definition to the ITS 3.6.4.1 Bases.	Bases	Bases Control Process in ITS Chapter 5.0	1

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
CHAPTER 2.0 - SAFETY LIMITS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.0 - LCO AND SR APPLICABILITY

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.1 - REACTIVITY CONTROL SYSTEMS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.1.1, SHUTDOWN MARGIN					
3.1.1-LA.1	4.3.A.1, 4.3.A.3	The details of the methods to determine SHUTDOWN MARGIN (SDM).	Bases	Bases Control Program in ITS Chapter 5	3
3.1.2, Reactivity Anomalies					
3.1.2-LA.1	3.3.B Action	The requirement to perform an analysis to determine and explain the cause of the reactivity difference.	Bases	Bases Control Program in ITS Chapter 5	3
3.1.3, Control Rod OPERABILITY					
3.1.3-LA.1	3.3.C Actions 1.a.2), 2.b, and 2.c, 3.3.H Action 1.b, 3.3.I Action 1.c	The details of the recommended procedures for disarming control rod drives.	Bases	Bases Control Program in ITS Chapter 5	3
3.1.3-LA.2	3.3.I Actions 1.a and 1.b	Details of methods for determining the position of a control rod.	Bases	Bases Control Program in ITS Chapter 5	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.1 - REACTIVITY CONTROL SYSTEMS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.1.4, Control Rod Scram Times					
3.1.4-LA.1	4.3.D.3	ITS SR 3.1.4.2 will test a "representative sample" of control rods each 120 days of power operation instead of the CTS requirement of "10% of the control rods on a rotating basis". The details of what constitutes a representative sample are relocated.	Bases	Bases Control Program in ITS Chapter 5	3
3.1.5, Control Rod Scram Accumulators					
NONE	NONE	NONE	NONE	NONE	NONE
3.1.6, Rod Pattern Control					
NONE	NONE	NONE	NONE	NONE	NONE
3.1.7, Standby Liquid Control System					
3.1.7-LA.1	4.4.A.2.b	The details of the method for performing the Surveillance to determine boron concentration is within limits (by a chemical analysis).	Bases	Bases Control Program in ITS Chapter 5	3
3.1.7-LA.2	4.4.A.4.a	The details of the method for performing the Surveillance to verify flow through the SLC subsystem into the reactor pressure vessel (initiating an explosive valve).	Bases	Bases Control Program in ITS Chapter 5	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.1 - REACTIVITY CONTROL SYSTEMS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.1.8, SDV Vent and Drain Valves					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.3.J, Control Rod Drive Housing Support					
None-L.1	3/4.3.J	Requirement for Control Rod Drive Housing Support to be in place.	NONE	NONE	NONE
Current Specification 3/4.3.N, Economic Generation Control System					
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA- REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.2 - POWER DISTRIBUTION LIMITS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.2.1, AVERAGE PLANAR LINEAR HEAT GENERATION RATE					
3.2.1-LA.1	3.11.A Action 1	The requirement in the CTS 3.11.A Action 1 to "initiate corrective action within 15 minutes" to restore the limit is relocated in the form of a discussion that "prompt action" should be taken to restore the parameter to within the limits.	Bases	Bases Control Program in ITS Chapter 5	3
3.2.2, MINIMUM CRITICAL POWER RATIO					
3.2.2-LA.1	3.11.C Action 1	The requirement in the CTS 3.11.C Action 1 to "initiate corrective action within 15 minutes" to restore the limit is relocated in the form of a discussion that "prompt action" should be taken to restore the parameter to within the limits.	Bases	Bases Control Program in ITS Chapter 5	3
3.2.3, LINEAR HEAT GENERATION RATE					
3.2.3-LA.1	3.11.D Action 1	The requirement in the CTS 3.11.D Action 1 to "initiate corrective action within 15 minutes" to restore the limit is relocated in the form of a discussion that "prompt action" should be taken to restore the parameter to within the limits.	Bases	Bases Control Program in ITS Chapter 5	3
3.2.4, APRM GAIN AND SETPOINT					

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TABLE LA- REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.2 - POWER DISTRIBUTION LIMITS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.2.4-LA.1	3.11.B Action	The requirement in the CTS 3.11.B Action to "initiate corrective action within 15 minutes" to restore the limit is relocated in the form of a discussion that "prompt action" should be taken to restore the parameter to within the limits.	Bases	Bases Control Program in ITS Chapter 5	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.3 - INSTRUMENTATION

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.3.1.1, RPS Instrumentation					
3.3.1.1-LA.1	3.1.A Action footnotes a and b	The details relating to placing channels in trip (e.g, if tripping causes trip function to occur, tripping trip system with the most inoperable channels).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.1.1-LA.2	Table 3.1.A-1 footnote (e)	The LPRM inputs for OPERABILITY of the APRM (2 per level; 50% of the LPRM inputs to each required APRM).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.1.1-LA.3	4.1.A-1 footnote (b)	Details of the methods for performing CTS 4.1.A.1, the IRM and APRM CHANNEL CHECK (½ decade overlap).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.1.1-LA.4	Table 2.2.A-1 footnote (a)	The detail of system description for the APRM Flow-Biased Neutron Flux—High scram Allowable Value (the definition of W, the recirculation loop drive flow).	Bases	Bases Control Program in ITS Chapter 5	1
3.3.1.1-LA.5	Table 2.2.A-1 footnote (b)	The details in concerning the Allowable Value of the Reactor Vessel Water Level—Low Function (i.e., the Allowable Value is referenced to a level above the top of active fuel, and that the top of active fuel is defined to be 360 inches above vessel zero).	UFSAR	10 CFR 50.59	1
3.3.1.2, SRM Instrumentation					
3.3.1.2-LA.1	4.2.G.1	The detail of the method for performing the Surveillance ("with the detector fully inserted").	Bases	Bases Control Program in ITS Chapter 5	3

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3.3.1.2-LA.2	3.10.B, 3.10.B.1, 4.10.B.1	The details relating to SRM OPERABILITY; i.e., that the SRMs shall be inserted to the normal operating level with continuous indication in the control room.	Bases	Bases Control Program in ITS Chapter 5	1, 2
3.3.1.2-LA.3	3.10.B Applicability	The additional spatial limitations when movable detectors are being used.	Bases	Bases Control Program in ITS Chapter 5	3
3.3.2.1, Control Rod Block Instrumentation					
3.3.2.1-LA.1	Table 3.2.E-1 footnote (a)	The statement that the RBM shall be automatically bypassed when a peripheral control rod is selected.	Bases	Bases Control Program in ITS Chapter 5	1
3.3.2.1-LA.2	Table 4.2.E-1 Functional Unit 1, footnote (c), 4.3.L.2.a, 4.3.L.2.b	Details in of the methods for performing Surveillances (i.e., the RBM CFT includes the reactor manual control “relay select matrix” system input and the RWM CFT includes verifying correct indication of the selection error of at least one out-of-sequence control rod and verifying the rod block function).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.2.2, Feedwater System and Main Turbine High Water Level Trip Instrumentation					
3.3.2.2-LA.1	Table 3.2.J-1 footnote (a)	The detail concerning the Allowable Value for the Reactor Vessel Water Level — High Function (i.e., the Allowable Value is referenced to a level above the top of active fuel, and that the top of active fuel is defined to be 360 inches above vessel zero).	UFSAR	10 CFR 50.59	1

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3.3.3.1, Post Accident Monitoring Instrumentation					
3.3.3.1-LA.1	Table 3.2.F-1 Action 61	The use of alternate methods of monitoring (initiate the preplanned alternate method of monitoring the parameters).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.3.1-LA.2	Table 4.2.F-1 footnote (a)	The detail of the method for performing the CHANNEL CALIBRATION of the Drywell Radiation Monitors.	Bases	Bases Control Program in ITS Chapter 5	3
3.3.3.1-LA.3	Table 3.2.F-1 (including footnote (a)) and 4.2.F-1 Instrumentation 13	The Torus Pressure Function because it is shared with drywell pressure functions which are retained in ITS.	Bases	Bases Control Program in ITS Chapter 5	1
3.3.4.1, ATWS-RPT Instrumentation					
3.3.4.1-LA.1	3.2.C Action 2 footnote (a)	The details relating to placing channels in trip (e.g, if tripping causes trip function to occur).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.4.1-LA.2	Table 3.2.C-1 footnote (c)	The detail concerning the Allowable Value for the Reactor Vessel Water Level — Low Low Function (i.e., the Allowable Value is referenced to a level above the top of active fuel, and that the top of active fuel is defined to be 360 inches above vessel zero).	UFSAR	10 CFR 50.59	1
3.3.5.1, ECCS Instrumentation					

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3.3.5.1-LA.1	Table 3.2.B-1 footnote (h), Table 3.2.B-1 Functional Unit 3.d	The detail concerning the Allowable Values for the Reactor Vessel Water Level Functions (i.e., the Allowable Value is referenced to a level above the top of active fuel, and that the top of active fuel is defined to be 360 inches above vessel zero) and the detail that the Allowable Value for the HPCI Suppression Chamber Water Level - High Function is referenced above the bottom of the chamber.	UFSAR	10 CFR 50.59	1
3.3.5.1-LA.2	Table 3.2.B-1 footnote (i)	The detail relating to system design (e.g., valves associated with isolation signals).	Bases	Bases Control Program in ITS Chapter 5	1
3.3.5.2, IC System Instrumentation					
NONE	NONE	NONE	NONE	NONE	NONE
3.3.6.1, Primary Containment Isolation Instrumentation					
3.3.6.1-LA.1	3.2.A Action 2 footnote (a)	The details relating to placing channels in trip (e.g, if tripping causes trip function to occur).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.6.1-LA.2	Table 3.2.A-1 footnote (i)	The detail concerning the Allowable Values for the Reactor Vessel Water Level Functions (i.e., the Allowable Value is referenced to a level above the top of active fuel, and that the top of active fuel is defined to be 360 inches above vessel zero).	UFSAR	10 CFR 50.59	1

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3.3.6.1-LA.3	Table 3.2.A-1 footnote (f)	The detail in CTS that the Standby Liquid Control System Initiation Function channel closes only reactor water cleanup system isolation valves	Bases	Bases Control Program in ITS Chapter 5	1
3.3.6.2, Secondary Containment Isolation Instrumentation					
3.3.6.2-LA.1	3.2.A Action 2 footnote (a)	The details relating to placing channels in trip (e.g, if tripping causes trip function to occur).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.6.2-LA.2	Table 3.2.A-1 footnote (i)	The detail concerning the Allowable Values for the Reactor Vessel Water Level Functions (i.e., the Allowable Value is referenced to a level above the top of active fuel, and that the top of active fuel is defined to be 360 inches above vessel zero).	UFSAR	10 CFR 50.59	1
3.3.6.2-LA.3	Tables 3.2.A-1 and 4.2.A-1 footnote (c)	Details relating to system design (i.e., specific valves and systems affected).	Bases	Bases Control Program in ITS Chapter 5	1
3.3.6.2-LA.4	4.7.P.4.b. 2)	The details in CTS 4.7.P.4.b.2 relating to methods for performing the LOGIC SYSTEM FUNCTIONAL TEST (use of simulated signals).	Bases	Bases Control Program in ITS Chapter 5	3
3.3.6.3, Relief Valve Instrumentation					
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.3 - INSTRUMENTATION

3.3.7.1, CREV System Instrumentation					
NONE	NONE	NONE	NONE	NONE	NONE
3.3.8.1, Loss of Power Instrumentation					
3.3.8.1-LA.1	Table 3.2.B-1 Functional Unit 6.a	The detail relating to the methods (on decreasing voltage) for determining the 4160 V ESS Bus Undervoltage (Loss of Voltage) Allowable Value.	Bases	Bases Control Program in ITS Chapter 5	1
3.3.8.2, RPS Electric Power Monitoring					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.2.H, Explosive Gas Monitoring					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.2.I, Suppression Chamber and Drywell Spray Actuation					
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.4 - REACTOR COOLANT SYSTEM

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.4.1, Recirculation Loops Operating					
3.4.1-LA.1	3.6.A Action 1.b	The details of the actual MCPR correction factor for the MCPR operating limit for single loop operation (“0.01").	COLR	COLR change control process described in Chapter 5 of the ITS	3
3.4.1-LA.2	4.6.A	The details relating to the recirculation pump MG set scoop tube stop settings.	TRM	10 CFR 50.59	1
3.4.1-LA.3	3.6.A Action 2	The requirement to "immediately initiate measures to place the unit in at least STARTUP" when no recirculation loops are in operation is relocated in the form of a discussion that "action must be taken as soon as practicable" to be in MODE 2.	Bases	Bases Control Program in ITS Chapter 5	3
3.4.2, Jet Pumps					
NONE	NONE	NONE	NONE	NONE	NONE
3.4.3, Safety and Relief Valves					
3.4.3-LA.1	3.6.E footnote (a)	The details relating to lift setting pressure of the safety valves (the lift setting pressure shall correspond to ambient conditions of the valves at nominal operating temperatures and pressures).	Bases	Bases Control Program in ITS Chapter 5	1
3.4.3-LA.2	3.6.F footnote (a)	The details indicating that one of the relief valves is a “Target Rock” combination safety valve.	Bases	Bases Control Program in ITS Chapter 5	3

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3.4.3-LA.3	4.6.E.2	The requirements for safety valve setting verification demonstrating the Reactor Coolant System (RCS) safety valves are OPERABLE.	IST Program	IST Program in ITS Chapter 5	3
3.4.4, RCS Operational Leakage					
3.4.4-LA.1	4.6.H.2	Details of the method for performing the reactor coolant system leakage Surveillance (by determining the primary containment sump flow rate).	Bases	Bases Control Program in ITS Chapter 5	3
3.4.5, RCS Leakage Detection Instrumentation					
3.4.5-LA.1	4.6.G.2	The detail of what Drywell Floor Drain Sump Monitoring System instrumentation (pump discharge flow integrator) is subject to a CHANNEL CALIBRATION.	Bases	Bases Control Program in ITS Chapter 5	3
3.4.6, RCS Specific Activity					
NONE	NONE	NONE	NONE	NONE	NONE
3.4.7, Shutdown Cooling System - Hot Shutdown					
3.4.7-LA.1	3.6.O.1, 3.6.O.2	The details of what constitutes an OPERABLE SDC subsystem (i.e., each subsystem consists of an OPERABLE pump and heat exchanger).	Bases	Bases Control Program in ITS Chapter 5	1
3.4.7-LA.2	4.6.O	The detail of the method of verifying operation of the SDC subsystem (circulating reactor coolant).	Bases	Bases Control Program in ITS Chapter 5	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.4 - REACTOR COOLANT SYSTEM

3.4.8, Shutdown Cooling System - Cold Shutdown					
3.4.8-LA.1	3.6.P.1, 3.6.P.2	The details of what constitutes an OPERABLE SDC subsystem (i.e., each subsystem consists of an OPERABLE pump and heat exchanger).	Bases	Bases Control Program in ITS Chapter 5	1
3.4.8-LA.2	4.6.P	The detail of the method of verifying operation of the SDC subsystem (circulating reactor coolant).	Bases	Bases Control Program in ITS Chapter 5	3
3.4.9, RCS Pressure and Temperature (P/T) Limits					
3.4.9-LA.1	3.6.K Action 2	The detail to perform an engineering evaluation to determine the effects of the out-of-limit condition on the structural integrity of the reactor coolant system.	Bases	Bases Control Program in ITS Chapter 5	3
3.4.9-LA.2	3.6.D Action, 4.6.D	The details relating to operational limits (loop flow) during a return to two recirculation pump operation from single recirculation loop operation.	UFSAR	10 CFR 50.59	1
3.4.10, Reactor Steam Dome Pressure					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.6.N, Structural Integrity					
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.5 - ECCS AND IC SYSTEM

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.5.1, ECCS-Operating					
3.5.1-LA.1	3.5.A	The details relating to ECCS subsystem OPERABILITY (i.e., number of pumps and flow paths capable of taking suction from the suppression chamber and transferring water to the reactor vessel).	Bases	Bases Control Program in ITS Chapter 5	1
3.5.1-LA.2	4.5.A.1.a. 2) footnote (a)	The details relating to what "correct position" means for an automatic valve.	Bases	Bases Control Program in ITS Chapter 5	3
3.5.1-LA.3	4.5.A.1.b, 4.5.A.3.b. 1), 4.5.A.3.b. 2), 4.5.A.4.b	The details relating to methods for performing Surveillances (i.e., the minimum pressure to perform the low pressure HPCI flow test, verifying the HPCI System pump flow controller is in the correct position, verifying the HPCI suction is automatically transferred from the contaminated condensate storage tank to the suppression pool on the proper signals, and verifying proper operation of the ADS valves).	Bases	Bases Control Program in ITS Chapter 5	2, 3
3.5.2, ECCS-Shutdown					
3.5.2-LA.1	3/4.5.B, 3.5.C.2	The details of CTS 3/4.5.B relating to system OPERABILITY (in this case what constitutes an OPERABLE ECCS subsystem) and CTS 3.5.C.2 (reference for suppression chamber level).	Bases	Bases Control Program in ITS Chapter 5	1
3.5.3, IC System					
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.6 - CONTAINMENT SYSTEMS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.6.1.1, Primary Containment					
NONE	NONE	NONE	NONE	NONE	NONE
3.6.1.2, Primary Containment Air Lock					
3.6.1.2-LA.1	3.7.C Action 2	The purpose as to why an individual is dedicated to ensure the necessary administrative controls during entry and exit of personnel through an air lock with an inoperable air lock interlock mechanism are followed (i.e., “to assure that both air lock doors are not opened simultaneously”).	Bases	Bases Control Program in ITS Chapter 5	3
3.6.1.3, Primary Containment Isolation Valves					
3.6.1.3-LA.1	4.7.D.3	The requirement to stroke time test the power operated, non-automatic, PCIVs.	IST Program	10 CFR 50.59 and 10 CFR 50.55a	3
3.6.1.3-LA.2	4.7.D.5.b	Requirements in CTS 4.7.D.5.b concerning the replacement charges for the traversing in-core probe (TIP) explosive valves (i.e., replacement charge shall be from the same batch or from another batch that has had one charge fired, and no charge shall remain in use past its shelf-life and operating-life).	Bases	Bases Control Program in ITS Chapter 5	3
3.6.1.3-LA.3	4.7.D.6	The details that the main steam isolation valve leakage is on a maximum pathway leakage basis and is tested “in accordance with the methods” of the Primary Containment Leakage Rate Testing Program.	Bases	Bases Control Program in ITS Chapter 5	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.6 - CONTAINMENT SYSTEMS

3.6.1.4, Drywell Pressure					
NONE	NONE	NONE	NONE	NONE	NONE
3.6.1.5, Drywell Air Temperature					
NONE	NONE	NONE	NONE	NONE	NONE
3.6.1.6, Low Set Relief Valves					
NONE	NONE	NONE	NONE	NONE	NONE
3.6.1.7, Reactor Building-to-Suppression Chamber Vacuum Breakers					
3.6.1.7-LA.1	3.7.F	The detail comprising what "OPERABLE" means (i.e, closed).	Bases	Bases Control Program in ITS Chapter 5	1
3.6.1.8, Suppression Chamber-to-Drywell Vacuum Breakers					
3.6.1.8-LA.1	4.7.E.2.c. 1)	The detail that the opening setpoint is verified from the closed position.	Bases	Bases Control Program in ITS Chapter 5	3
3.6.2.1, Suppression Pool Average Temperature					
NONE	NONE	NONE	NONE	NONE	NONE

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3.6.2.2, Suppression Pool Water Level					
3.6.2.2-LA.1	3.5.C.1	The detail that the suppression pool level limit is referenced from the bottom of the suppression chamber .	Bases	Bases Control Program in ITS Chapter 5	1
3.6.2.3, Suppression Pool Cooling					
3.6.2.3-LA.1	3.7.M	The details relating to system OPERABILITY (in this case the suppression pool cooling function is designated as two "independent" subsystems, each with a pump and flow path).	Bases	Bases Control Program in ITS Chapter 5	1
3.6.2.4, Suppression Pool Spray					
3.6.2.4-LA.1	3.7.L	The details relating to system OPERABILITY (in this case the suppression pool spray function is designated as two "independent" subsystems, each with a pump and flow path).	Bases	Bases Control Program in ITS Chapter 5	1
3.6.2.5, Drywell-to-Suppression Chamber Differential Pressure					
3.6.2.5-LA.1	3.7.H footnote (a)	The detail that defines the types of required Surveillances (“which reduces the differential pressure”) where the drywell-to-suppression chamber differential pressure can be outside of limits for 4 hours.	Bases	Bases Control Program in ITS Chapter 5	3
3.6.3.1, Primary Containment Oxygen Concentration					
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.6 - CONTAINMENT SYSTEMS

3.6.4.1, Secondary Containment					
NONE	NONE	NONE	NONE	NONE	NONE
3.6.4.2, Secondary Containment Isolation Valves					
NONE	NONE	NONE	NONE	NONE	NONE
3.6.4.3, Standby Gas Treatment System					
3.6.4.3-LA.1	3.7.P	The detail relating to system design (i.e., that the SGT subsystems are "independent")	Bases	Bases Control Program in ITS Chapter 5	1
3.6.4.3-LA.2	4.7.P.1, 4.7.P.4.b.1	Details of the methods for performing the standby gas treatment subsystem 31 day operating Surveillance (by initiating, from the control room, flow through the HEPA filters and charcoal adsorbers) and the current manual actuation test requirement (verifying "Manual initiation from the control room").	Bases	Bases Control Program in ITS Chapter 5	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.7 - PLANT SYSTEMS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.7.1, Containment Cooling Service Water System					
3.7.1-LA.1	3.8.A	The details relating to system OPERABILITY (that the CCSW subsystems shall be independent and that each subsystem shall have two CCSW pumps capable of taking suction from the ultimate heat sink and transferring the water to the associated LPCI heat exchanger and safety related equipment).	Bases	Bases Control Program in ITS Chapter 5	1
3.7.1-LA.2	3/4.8.A	LCO requirements, Actions, and Surveillance Requirements for the CCSW System when in MODES 4 and 5 and when handling irradiated fuel in the secondary containment, CORE ALTERATIONS, and OPDRVs.	TRM	10 CFR 50.59	3
3.7.2, Diesel Generator Cooling Water System					
3.7.2-LA.1	3.8.B	The details relating to system OPERABILITY (that each DGCW subsystem shall have one OPERABLE DGCW pump, and an OPERABLE flow path capable of taking suction from the ultimate heat sink and transferring water to the associated diesel generator).	Bases	Bases Control Program in ITS Chapter 5	1
3.7.2-LA.2	3/4.8.B	LCO requirements, Actions, and Surveillance Requirements for the DGCW System when in MODES or conditions other than MODE 1, 2, or 3. Due to this change, the Applicability has been modified to be “MODES 1, 2, and 3,” consistent with the DG Applicability requirements in ITS 3.8.1.	TRM	10 CFR 50.59	3
3.7.2-LA.3	4.8.B.2	The detail concerning the specific start signal (start signal for the associated DG) to be used during the surveillance.	Bases	Bases Control Program in ITS Chapter 5	3

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3.7.3, Ultimate Heat Sink					
3.7.3-LA.1	3.8.C	LCO requirements, Actions, and Surveillance Requirements for the Ultimate Heat Sink when in MODES or conditions other than MODE 1, 2, or 3.	TRM	10 CFR 50.59	3
3.7.4, Control Room Emergency Ventilation System					
3.7.4-LA.1	3.8.D	The details of what constitutes the Control Room Emergency Ventilation System (i.e, an Operable control room emergency filtration system).	Bases	Bases Control Program in ITS Chapter 5	1
3.7.4-LA.2	4.8.D.2, 4.8.D.5.b	Details of the methods for performing the CREV System 31 day operating Surveillance and system actuation test (by initiating, from the control room, flow through the HEPA filters and charcoal adsorbers and by verifying the filter train starts and isolation dampers close on manual initiation from the control room).	Bases	Bases Control Program in ITS Chapter 5	3
3.7.5, Control Room Emergency Ventilation Air Conditioning System					
3.7.5-LA.1	3.8.D	The details of what constitutes the Control Room Emergency Ventilation Air Conditioning System (i.e, an Operable refrigeration control unit).	Bases	Bases Control Program in ITS Chapter 5	1
3.7.6, Main Condenser Offgas					
3.7.6-LA.1	4.8.I.1	The requirement to continuously monitor radioactivity rate of noble gases from the main condenser air ejector.	ODCM	ODCM Control Process in ITS Chapter 5	3

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3.7.6-LA.2	4.8.I.2	Details defining the method for performing this Surveillance (i.e., performing an isotopic analysis of a representative sample of gases taken at the recombiner outlet, or the air ejector outlet, if the recombiner is bypassed).	Bases	Bases Control Program in ITS Chapter 5	3
3.7.7, Main Turbine Bypass System					
NONE	NONE	NONE	NONE	NONE	NONE
3.7.8, Spent Fuel Storage Pool Water Level					
3.7.8-LA.1	3.10.H Action	The requirement to suspend crane operations with loads in the spent fuel storage pool area when the spent fuel storage pool water level is not within the limit.	UFSAR	10 CFR 50.59	3
3.7.8-LA.2	3.10.H Action	Details of the methods for suspending movement of fuel assemblies (after placing the fuel assemblies in a safe condition).	Bases	Bases Control Program in ITS Chapter 5	3
Current Specification 3/4.8.E, Flood Protection					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.8.F, Snubbers					
None-LA.1	3/4.8.F	Snubber inspection and testing requirements.	TRM	10 CFR 50.59	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.7 - PLANT SYSTEMS

Current Specification 3/4.8.G, Sealed Source Contamination					
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.8 - ELECTRICAL POWER SYSTEMS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.8.1, AC Sources - Operating					
3.8.1-LA.1	LCO 3.9.A.1, LCO 3.9.A.2, LCO 3.9.A.2.c	The details relating to system design and OPERABILITY (i.e., that the offsite circuits are "physically independent," the DGs are "separate and independent," and that each DG has "a separate fuel transfer pump").	Bases	Bases Control Program in ITS Chapter 5	1
3.8.1-LA.2	4.9.A.8.b	The specific kilowatt value of the single largest post-accident load for the single load rejection Surveillance Requirement (increased to the proper value).	Bases	Bases Control Program in ITS Chapter 5	1
3.8.1-LA.3	N/A	Not used.	N/A	N/A	N/A
3.8.1-LA.4	4.9.A.8.i	The specific load value for the auto-connected loads.	UFSAR	10 CFR 50.59	1
3.8.2, AC Sources - Shutdown					
3.8.2-LA.1	LCO 3.9.B.2.c	The detail relating to system design and OPERABILITY (i.e., that each DG has a fuel oil transfer pump).	Bases	Bases Control Program in ITS Chapter 5	1
3.8.2-LA.2	3.9.B Action 1.d	Requirements to suspend crane operations over the spent fuel storage pool if fuel assemblies are stored therein when an AC Source is inoperable	UFSAR	10 CFR 50.59	3
3.8.3, Diesel Fuel Oil and Starting Air					
3.8.3-LA.1	4.9.A.10	The 10 year requirement to drain, remove sediment and and clean each fuel tank.	TRM	10 CFR 50.59	3

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3.8.4, DC Sources - Operating					
3.8.4-LA.1	LCO 3.9.C.1, LCO 3.9.C.2	Details relating to system OPERABILITY (what constitutes a DC Source division).	Bases	Bases Control Program in ITS Chapter 5	1
3.8.4-LA.2	4.9.C footnote (a)	The detail that an alternate 125 volt battery shall adhere to these same Surveillance Requirements to be considered OPERABLE	Bases	Bases Control Program in ITS Chapter 5	3
3.8.4-LA.3	4.9.C.4	The details of the method (actual or simulated) to perform the battery service test.	Bases	Bases Control Program in ITS Chapter 5	3
3.8.4-LA.4	4.9.C.6	Specific limits on battery degradation and guidance regarding the intent of the term "degradation."	Bases	Bases Control Program in ITS Chapter 5	3
3.8.5, DC Sources - Shutdown					
3.8.5-LA.1	LCO 3.9.D.1, LCO 3.9.D.2	Details relating to system OPERABILITY (what constitutes a required DC electrical power source.	Bases	Bases Control Program in ITS Chapter 5	1
3.8.5-LA.2	4.9.D footnote (a)	The detail that an alternate 125 volt battery shall adhere to these same Surveillance Requirements to be considered OPERABLE is relocated in the form of a discussion that states the alternate 125 VDC battery can be used to meet the requirements of the LCO.	Bases	Bases Control Program in ITS Chapter 5	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.8 - ELECTRICAL POWER SYSTEMS

3.8.6, Battery Cell Parameters					
3.8.6-LA.1	4.9.C footnote (a)	The detail that an alternate 125 volt battery shall adhere to these same Surveillance Requirements to be considered OPERABLE is relocated in the form of a discussion that states the alternate 125 VDC battery can be used to meet the requirements of the LCO.	Bases	Bases Control Program in ITS Chapter 5	3
3.8.7, Distribution Systems - Operating					
3.8.7-LA.1	LCO 3.9.E, LCO 3.9.E.1, LCO 3.9.E.2, LCO 3.9.E.3, LCO 3.9.E.4, 3.9.E Actions 1 and 2	The details relating to system design (the list of buses) and OPERABILITY (the buses are required to be energized).	Bases	Bases Control Program in ITS Chapter 5	1
3.8.8, Distribution Systems - Shutdown					

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.8 - ELECTRICAL POWER SYSTEMS

3.8.8-LA.1	LCO 3.9.F, LCO 3.9.F.1, LCO 3.9.F.2, LCO 3.9.F.3, 3.9.F Action	The details relating to system design (the list of buses) and OPERABILITY (the buses are required to be energized).	Bases	Bases Control Program in ITS Chapter 5	1

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.9 - REFUELING OPERATIONS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.9.1, Refueling Equipment Interlocks					
NONE	NONE	NONE	NONE	NONE	NONE
3.9.2, Refuel Position One-Rod-Out Interlock					
NONE	NONE	NONE	NONE	NONE	NONE
3.9.3, Control Rod Position					
NONE	NONE	NONE	NONE	NONE	NONE
3.9.4, Control Rod Position Indication					
NONE	NONE	NONE	NONE	NONE	NONE
3.9.5, Control Rod OPERABILITY - Refueling					
NONE	NONE	NONE	NONE	NONE	NONE
3.9.6, RPV Water Level - Irradiated Fuel					
3.9.6-LA.1	3.10.G Action	Details of the methods for suspending movement of fuel assemblies (after placing the fuel assemblies in a safe condition).	Bases	Bases Control Program in ITS Chapter 5	3
3.9.7, RPV Water Level - New Fuel or Control Rods					

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.9 - REFUELING OPERATIONS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.9.7-LA.1	3.10.G Action	Details of the methods for suspending movement of fuel assemblies and control rods (after placing the fuel assemblies and control rods in a safe condition).	Bases	Bases Control Program in ITS Chapter 5	3
3.9.8, Shutdown Cooling (SDC) - High Water Level					
3.9.8-LA.1	3.10.K.1, 3.10.K.2	The details of what constitutes an OPERABLE RHR shutdown cooling subsystem (i.e., each subsystem consists of one OPERABLE RHR pump and heat exchanger).	Bases	Bases Control Program in ITS Chapter 5	3
3.9.8-LA.2	4.10.K	The detail of the method of verifying operation of the shutdown cooling subsystem (circulating reactor coolant).	Bases	Bases Control Program in ITS Chapter 5	3
3.9.9, Shutdown Cooling (SDC) - Low Water Level					
3.9.9-LA.1	3.10.L.1, 3.10.L.2	The details of what constitutes an OPERABLE RHR shutdown cooling subsystem (i.e., each subsystem consists of one OPERABLE RHR pump and heat exchanger).	Bases	Bases Control Program in ITS Chapter 5	3
3.9.9-LA.2	4.10.L	The detail of the method of verifying operation of the shutdown cooling subsystem (circulating reactor coolant).	Bases	Bases Control Program in ITS Chapter 5	3
Current Specification 3/4.10.E, Communications					
NONE	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.10 - SPECIAL OPERATIONS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.10.1, Reactor Mode Switch Interlock Testing					
3.10.1-LA.1	Table 1-2 footnote (a), 4.10.A.2, 4.10.A.3	The method used to verify control rods remain fully inserted (by verification using a second licensed operator or other technically qualified member of the unit technical staff).	Bases	Bases Control Program in ITS Chapter 5	3
3.10.2, Single Control Rod Withdrawal - Hot Shutdown					
NONE	NONE	NONE	NONE	NONE	
3.10.3, Single Control Rod Withdrawal - Cold Shutdown					
3.10.3-LA.1	3.10.I.4.a, 4.10.I.4.a	The details of the recommended procedures for disarming control rods (i.e., electrically or hydraulically).	Bases	Bases Control Program in ITS Chapter 5	3
3.10.4, Single Control Rod Drive Removal - Refueling					
3.10.4-LA.1	3.10.I.4.a, 4.10.I.4.a	The details of the recommended procedures for disarming control rods (i.e., electrically or hydraulically).	Bases	Bases Control Program in ITS Chapter 5	3
3.10.5, Multiple Control Rod Withdrawal - Refueling					
NONE	NONE	NONE	NONE	NONE	

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
SECTION 3.10 - SPECIAL OPERATIONS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
3.10.6, Control Rod Testing - Operating					
NONE	NONE	NONE	NONE	NONE	NONE
3.10.7, SDM Test - Refueling					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.12.A, Primary Containment Integrity					
None-A.1	NONE	NONE	NONE	NONE	NONE
Current Specification 3/4.12.C, Inservice Leak and Hydrostatic Testing Operation					
None-M.1	NONE	NONE	NONE	NONE	NONE

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
CHAPTER 4.0 - DESIGN FEATURES

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
4.0-LA.1	5.1.A	The description of the site location.	UFSAR	10 CFR 50.59	1
4.0-LA.2	5.2.A, 5.2.B, 5.2.C	Primary containment configuration and design details, primary containment design temperatures and pressures, and secondary containment design details.	UFSAR	10 CFR 50.59	1
4.0-LA.3	5.3.B	The nominal active control rod assembly absorber length.	UFSAR	10 CFR 50.59	1

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
CHAPTER 5.0 - ADMINISTRATIVE CONTROLS

ITS SECTION AND DOC #	CTS SECTION	SUMMARY	LOCATION	CHANGE CONTROL PROCESS	CHANGE TYPE
5.1, Responsibility					
5.1-LA.1	6.1.A	Replaces the specific title "Station Manager" with the generic title "station manager" and relocates the specific title.	QA Manual	10 CFR 50.54	3
5.1-LA.2	6.1.B	The requirement that delineates the responsibility of the Shift Manager for directing and commanding the overall operations on the shift.	UFSAR	10 CFR 50.59	3
5.2, Organization					
5.2-LA.1	6.2.A.2, 6.2.A.3, 6.2.B.6, 6.2.C	Replaces the specific title "Chief Nuclear Officer" with the generic term "a corporate officer." Replaces the specific title "Station Manager" with the generic title "station manager." Replaces the specific titles "Operations Manager" and "Shift Operations Supervisor" with the generic titles "operations manager" and "shift operations supervisor." Replaces the specific title, "Unit Supervisor" with "Shift Manager." The specific titles are relocated. In addition, the person to whom the STA provides advisory technical support will be changed to the shift manager instead of the unit supervisor. This portion is considered administrative.	QA Manual	10 CFR 50.54	3
5.2-LA.2	6.2.B.5	The details with respect to the development and implementation of procedures to limit the working hours of facility staff who perform safety-related functions.	UFSAR	10 CFR 50.59	3
5.2-LA.3	6.2.B.2	The details concerning the location of operators and senior operator.	UFSAR	10 CFR 50.59	3

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TABLE LA - REMOVAL OF DETAILS MATRIX AND SPECIFICATION REQUIREMENTS
CHAPTER 5.0 - ADMINISTRATIVE CONTROLS

5.3, Unit Staff Qualifications					
5.3-LA.1	6.3	Replaces the specific titles "Radiation Protection Manager" with the generic titles "radiation protection manager." The specific titles are relocated.	QA Manual	10 CFR 50.54	3
5.4, Procedures					
5.4-LA.1	6.8.A.5	The requirement that written procedures for the PROCESS CONTROL PROGRAM (PCP) be established, implemented, and maintained.	UFSAR	10 CFR 50.59	3
5.5, Programs and Manuals					
5.5-LA.1	6.8.D.2	The details contained in CTS 6.8.D.2, "In-Plant Radiation Monitoring."	UFSAR	10 CFR 50.59	3
5.5-LA.2	4.0.E	Details of the Inservice Inspection (ISI) Program are relocated; and since the Inservice Testing Program is the only requirement remaining, the reference to ASME Code Class 1, 2, and 3 “components” has been changed to “pumps and valves” for clarity.	ISI Program	10 CFR 50.55a	3
5.5-LA.3	4.0.E	Details of the Inservice Testing Program.	IST Program	10 CFR 50.55a	3
5.5-LA.4	4.7.P.2.b, 4.7.P.3, 4.8.D.3.b, 4.8.D.4	The details for implementing the Standby Gas Treatment (SGT) System and the Control Room Emergency Ventilation (CREV) System ventilation filter testing requirements.	TRM	10 CFR 50.59	3
5.5-LA.5	3.8.H, 3.8.J	The details for implementing the liquid holdup tank and explosive gas mixture requirements.	TRM	10 CFR 50.59	3

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5.5-LA.6	6.14.A.2	Replaces the specific title "Station Manager" with the generic title "station manager." The specific title is relocated.	QA Manual	10 CFR 50.54	3
5.5-LA.7	6.8.D.1	Details of the Reactor Coolant Sources Outside Primary Containment Program.	UFSAR	10 CFR 50.59	3
5.6, Reporting Requirements					
5.6-LA.1	6.9.A.6.a.(4)	The details associated with the MCPR Specification (i.e.,scram insertion times, rated and of-rated flow conditions).	Bases	Bases Control Program in ITS Chapter 5	1
5.6-LA.2	6.9.A.6.b	The details of the actual topical reports document date, revision number, volume, supplement and company.	COLR	COLR change control process in ITS Chapter 5	1
5.7, High Radiation Area					
NONE	NONE	NONE	NONE	NONE	NONE
Current Specification 6.4, Training					
None-LA.1	6.4	The details on training and replacement training of station personnel.	UFSAR	10 CFR 50.59	3
Current Specification 6.7, Safety Limit Violation					
None-LA.1	6.7.A.1	The requirement for notification of the Site Vice President or designated alternate.	QA Manual	10 CFR 50.54	3

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Current Specification 6.11, Radiation Protection Program					
None-LA.1	6.11	The details on Radiation Protection Program procedures.	UFSAR	10 CFR 50.59	3
Current Specification 6.13, Process Control Program					
None-LA.1	6.13	The details contained in the Process Control Program Specification and the definition of PROCESS CONTROL PROGRAM.	UFSAR	10 CFR 50.59	3

CHANGE TYPE

1. Details of system design and system description including design limits
2. Description of system operation
3. Procedural details for meeting TS requirement, relocated reporting requirements and relocated specification requirements.

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