Mr. Michael R. Kansler Vice President, Operations Support Entergy Operations, Inc. P.O. Box 31995 Jackson, MS 39286-1995

SUBJECT:

ARKANSAS NUCLEAR ONE, UNIT NO. 1 - ISSUANCE OF AMENDMENT RE: LABORATORY TESTING OF ACTIVATED CHARCOAL (TAC NO. MA7280)

Dear Mr. Kansler:

The Commission has issued the enclosed Amendment No. 210 to Facility Operating License No. DPR-51 for Arkansas Nuclear One, Unit No. 1 (ANO-1). The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated November 23, 1999, as supplemented by letters dated February 24 and October 19, 2000.

The amendment incorporates the use of American Society for Testing and Materials (ASTM) D3803-1989, "Standard Test Method for Nuclear-Grade Activated Carbon," into the ANO-1 TSs.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

/RA/

William Reckley, Project Manager, Section 1 Project Directorate IV & Decommissioning Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket No. 50-313

## DISTRIBUTION

**Enclosures:** 

1. Amendment No. 210 to DPR-51 PDIV-1 Reading

**PUBLIC** 

RidsNrrPMWReckley RidsNrrPMTAlexion

RidsNrrDripRtsb (WBeckner)

RidsNrrLADJohnson

2. Safety Evaluation RidsOgcRp

RidsNrrDlpmPdiv (SRichards)

J. Segala L.Hurley, RIV

cc w/encls: See next page

RidsAcrsAcnwMailCenter

D. Bujol, RIV

RidsNrrDlpmPdivLpdiv1 (RGramm) G.Hill(2)

RidsRgn4MailCenter (KBrockman)

Accession No.:

| OFFICE | PDIV-1/PM         | PDIV-1/LA   | SPLB/SC  | OGC Nil  | PDIV-1/SC |  |
|--------|-------------------|-------------|----------|----------|-----------|--|
| NAME   | TAlexion/WReckley | DJohnson ∭χ | EWeiss , | (Burber: | RGramm    |  |
| DATE   | 12 07 06 12/11    | 12/6/00     | 17/12/00 | 12/20/00 | 12/28/00  |  |

DOCUMENT NAME: G:\PDIV-1\ANO1\AMDA7280.wpd /



# UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

December 28, 2000

Mr. Michael R. Kansler Vice President, Operations Support Entergy Operations, Inc. P.O. Box 31995 Jackson, MS 39286-1995

SUBJECT:

ARKANSAS NUCLEAR ONE, UNIT NO. 1 - ISSUANCE OF AMENDMENT RE: LABORATORY TESTING OF ACTIVATED CHARCOAL (TAC NO. MA7280)

Dear Mr. Kansler:

The Commission has issued the enclosed Amendment No. 210 to Facility Operating License No. DPR-51 for Arkansas Nuclear One, Unit No. 1 (ANO-1). The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated November 23, 1999, as supplemented by letters dated February 24 and October 19, 2000.

The amendment incorporates the use of American Society for Testing and Materials (ASTM) D3803-1989, "Standard Test Method for Nuclear-Grade Activated Carbon," into the ANO-1 TSs.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

William Reckley, Project Manager, Section 1
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-313

**Enclosures:** 

1. Amendment No. 210 to DPR-51

2. Safety Evaluation

cc w/encls: See next page

#### Arkansas Nuclear One

CC:

Executive Vice President & Chief Operating Officer Entergy Operations, Inc. P. O. Box 31995 Jackson, MS 39286-1995

Director, Division of Radiation Control and Emergency Management Arkansas Department of Health 4815 West Markham Street, Slot 30 Little Rock, AR 72205-3867

Winston & Strawn 1400 L Street, N.W. Washington, DC 20005-3502

Manager, Rockville Nuclear Licensing Framatone Technologies 1700 Rockville Pike, Suite 525 Rockville, MD 20852

Senior Resident Inspector U.S. Nuclear Regulatory Commission P. O. Box 310 London, AR 72847

Regional Administrator, Region IV U.S. Nuclear Regulatory Commission 611 Ryan Plaza Drive, Suite 400 Arlington, TX 76011-8064

County Judge of Pope County Pope County Courthouse Russellville, AR 72801 Wise, Carter, Child & Caraway P. O. Box 651 Jackson, MS 39205

Mr. Craig G. Anderson Vice President Operations, ANO Entergy Operations, Inc. 1448 S. R. 333 Russellville, AR 72801



# UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

## ENTERGY OPERATIONS INC.

## **DOCKET NO. 50-313**

## ARKANSAS NUCLEAR ONE, UNIT NO. 1

## AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 210 License No. DPR-51

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Operations, Inc. (the licensee) dated November 23, 1999, as supplemented by letters dated February 24 and October 19, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-51 is hereby amended to read as follows:
  - 2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 210 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of issuance.

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FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Gramm, Chief, Section 1 Project Directorate IV & Decommissioning Division of Licensing Project Management

Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical

Specifications

Date of Issuance: December 28, 2000

## ATTACHMENT TO LICENSE AMENDMENT NO. 210

## FACILITY OPERATING LICENSE NO. DPR-51

## **DOCKET NO. 50-313**

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

| Remove | Insert       |
|--------|--------------|
| 66c    | 66c          |
| 66d    | 66d          |
| 66g    | 66g          |
| 66h    | 66h          |
| 109a   | 109a         |
| 110i   | <b>1</b> 10i |

#### Applicability

Applies to the operability of the penetration room ventilation system.

#### Objective

To ensure that the penetration room ventilation system will perform within acceptable levels of efficiency and reliability.

## Specification

- 3.13.1 Two independent circuits of the penetration room ventilation system shall be operable whenever reactor building integrity is required with the following performance capabilities:
  - a. The results of the in-place cold DOP and halogenated hydrocarbon tests at design flow ( $\pm$  10%) on HEPA filters and charcoal adsorber banks shall show  $\geq$  99% DOP removal and  $\geq$  99% halogenated hydrocarbon removal.
  - b. The results of laboratory carbon sample analysis from the charcoal adsorber banks shall show the methyl iodide penetration less than 5.0% when tested in accordance with ASTM D3803-1989 at a temperature of  $30^{\circ}\text{C}$  and a relative humidity of 95%.
  - c. Fans shall be shown to operate within ± 10% of design flow.
  - d. The pressure drop across the combined HEPA filters and charcoal adsorber banks shall be less than 6 inches of water at system design flow rate ( $\pm$  10%).
  - e. Air distribution shall be uniform within ± 20% across HEPA filters and charcoal adsorbers when tested initially and after any maintenance or testing that could affect the air distribution within the penetration room ventilation system.
  - f. Each circuit of the system shall be capable of automatic initiation.
- 3.13.2 If one circuit of the penetration room ventilation system is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding seven days provided that during such seven days all active components of the other circuit shall be operable.
- 3.13.3 If the requirements of Specifications 3.13.1 and 3.13.2 cannot be met, the reactor shall be placed in the cold shutdown condition within 36 hours.

ML003781900

#### Bases

The penetration room ventilation system is designed to collect and process potential reactor building penetration leakage to minimize environmental activity levels resulting from post accident reactor building leaks. The system consists of sealed penetration rooms, two redundant filter trains and two redundant fans discharging to the unit vent. The entire system is activated by a reactor building engineered safety features signal and initially requires no operator action. Each filter train is constructed with a prefilter, a HEPA filter and a charcoal adsorber in series. The design flow rate through each of these filters is 2000 scfm, which is significantly higher than the 1.25 scfm maximum leakage rate from the reactor building at a leak rate of 0.1% per day.

High efficiency particulate air (HEPA) filters are installed before the charcoal adsorbers to prevent clogging of the iodine adsorbers. The charcoal adsorbers are installed to reduce the potential release of radioiodine to the environment. The in-place test results should indicate a system leak tightness of less than 1 percent bypass leakage for the charcoal adsorbers and a HEPA efficiency of at least 99 percent removal of DOP particulates. The laboratory carbon sample test results should ensure a radioactive methyl iodide removal efficiency of a least 90 percent for expected accident conditions. Acceptable removal efficiency is shown by a methyl iodide penetration of less than 5.0% when tests are performed in accordance with ASTM D3803-1989, "Standard Test Method for Nuclear-Grade Activated Carbon," at a temperature of 30°C and a relative humidity of 95%. The penetration acceptance criterion is determined by the following equation:

Allowable =[100% - methyl iodide efficiency for charcoal credited in accident analysis] Penetration safety factor of 2

Applying a safety factor of 2 is acceptable because ASTM D3803-1989 is a more accurate and demanding test than older tests.

If the efficiencies of the HEPA filters and charcoal adsorbers are as specified, the resulting doses will be less than the 10CFR100 guidelines for the accidents analyzed. Operation of the fans significantly different from the design flow will change the removal efficiency of the HEPA filters and charcoal adsorbers.

If one circuit of the penetration room ventilation system is found to be inoperable, there is not an immediate threat to the containment system performance and reactor operation may continue for a limited period of time while repairs are being made.

#### 3.15 FUEL HANDLING AREA VENTILATION SYSTEM

#### Applicability

Applies to the operability of the fuel handling area ventilation system.

#### Objective

To ensure that the fuel handling area ventilation system will perform within acceptable levels of efficiency and reliability.

#### Specification

- 3.15.1 The fuel handling area ventilation system shall be in operation whenever irradiated fuel handling operations are in progress in the fuel handling area of the auxiliary building and shall have the following performance capabilities:
  - a. The results of the in-place cold DOP and halogenated hydrocarbon tests at design flows ( $\pm 10\%$ ) on HEPA filters and charcoal adsorber banks shall show  $\geq$  99% DOP removal and  $\geq$  99% halogenated hydrocarbon removal.
  - b. The results of laboratory carbon sample analysis shall show the methyl iodide penetration less than 5.0% when tested in accordance with ASTM D3803-1989 at a temperature of 30°C and a relative humidity of 95%.
  - c. Fans shall be shown to operate within ±10% design flow.
  - d. The pressure drop across the combined HEPA filters and charcoal adsorber banks shall be less than 6 inches of water at system design flow rate  $(\pm 10\%)$ .
  - e. Air distribution shall be uniform within ±20% across HEPA filters and charcoal adsorbers when tested initially and after any maintenance or testing that could affect the air distribution within the fuel handling area ventilation system.
- 3.15.2 If the requirements of Specification 3.15.1 cannot be met, irradiated fuel movement shall not be started (any irradiated fuel assembly movement in progress may be completed). The provisions of Specification 3.0.3 are not applicable.

#### Bases

The fuel handling area ventilation system is designed to filter the auxiliary building atmosphere during fuel handling operations to limit the release of activity should a fuel handling accident occur. The system consists of one circuit containing two exhaust fans and a filter train. The fans are redundant and only one is required to be operating. The filter train consists of a prefilter, a HEPA filter and a charcoal adsorber in series.

High efficiency particulate air (HEPA) filters are installed before the charcoal adsorbers to prevent clogging of the iodine absorbers. The charcoal adsorbers are installed to reduce the potential release of radioiodine to the environment. The in-place test results should indicate a system leak tightness of less than 1 percent bypass leakage for the charcoal adsorbers and a HEPA efficiency of at least 99 percent removal of DOP particulates. The laboratory carbon sample test results should ensure a radioactive methyl iodide removal efficiency of at least 90 percent for expected accident conditions. Acceptable removal efficiency is shown by a methyl iodide penetration of less than 5.0% when tests are performed in accordance with ASTM D3803-1989, "Standard Test Method for Nuclear-Grade Activated Carbon," at a temperature of 30°C and a relative humidity of 95%. The penetration acceptance criterion is determined by the following equation:

Allowable Penetration = [100% - methyl iodide efficiency for charcoal credited in accident analysis] safety factor of 2

Applying a safety factor of 2 is acceptable because ASTM D3803-1989 is a more accurate and demanding test than older tests.

If the efficiencies of the HEPA filters and charcoal adsorbers are as specified, the resulting doses will be less than the 10CFR100 guidelines for the accidents analyzed. Operation of the fans significantly different from the design flow will change the removal efficiency of the HEPA filters and charcoal adsorbers.

#### Bases

The penetration room ventilation system is designed to collect and process potential reactor building penetration room leakage to minimize environmental activity levels resulting from post accident reactor building leaks. The system consists of a sealed penetration room, two redundant filter trains and two redundant fans discharging to the unit vent. The entire system is activated by a reactor building pressure engineered safety features signal and initially requires no operator action.

Since the system is not normally operated, a periodic test is required to show that the system is available for its engineered safety features function. During this test the system will be inspected for such things as water, oil, or other foreign material, gasket deterioration in the HEPA units, and unusual or excessive noise or vibration when the fan motor is running.

Pressure drop across the combined HEPA filters and charcoal adsorbers of less than 6 inches of water at the system design flow rate will indicate that the filters and adsorbers are not clogged by excessive amounts of foreign matter. Pressure drop should be determined at least once per 18 months to show system performance capability.

The frequency of tests and sample analysis are necessary to show that the HEPA filters and charcoal adsorbers can perform as evaluated. The charcoal adsorber efficiency test procedures should allow for obtaining at least two samples. Each sample should be at least two inches in diameter and a length equal to the thickness of the bed. Tests of the charcoal adsorbers with halogenated hydrocarbon refrigerant and of the HEPA filter bank with DOP aerosol shall be performed in accordance with ANSI N510 (1975) "Standard for Testing of Nuclear Air Cleaning Systems." Any HEPA filters found defective shall be replaced with filters qualified according to Regulatory Position C.3.d. of Regulatory Guide 1.52. Radioactive methyl iodide removal efficiency tests shall be performed in accordance with ASTM D3803-1989. If laboratory test results are unacceptable, all charcoal adsorbents in the system shall be replaced with charcoal adsorbents qualified according to ASTM D3803-1989.

Operation of the system each month for 1 hour will demonstrate operability of the active system components and the filter and adsorber system. If significant painting, fire or chemical release occurs such that the HEPA filter or charcoal adsorber could become contaminated from the fumes, chemicals or foreign material, the same tests and sample analysis shall be performed as required for operational use. The determination of significant shall be made by the operator on duty at the time of the incident. Knowledgeable staff members should be consulted prior to making this determination.

Pressure drop across the combined HEPA filters and charcoal adsorbers of less than 6 inches of water at the system design flow rate will indicate that the filters and adsorbers are not clogged by excessive amounts of foreign matter. Pressure drop and air distribution should be determined once every 18 months to show system performance capability.

The frequency of tests and sample analysis are necessary to show that the HEPA filters and charcoal adsorbers can perform as evaluated. The charcoal adsorber efficiency test procedures should allow for obtaining at least two samples. Each sample should be at least two inches in diameter and a length equal to the thickness of the bed. Tests of the charcoal adsorbers with halogenated hydrocarbon refrigerant and of the HEPA filter bank with DOP aerosol shall be performed in accordance with ANSI N510 (1975) "Standard for Testing of Nuclear Air Cleaning Systems." Any HEPA filters found defective shall be replaced with filters qualified according to Regulatory Position C.3.d. of Regulatory Guide 1.52. Radioactive methyl iodide removal efficiency tests shall be performed in accordance with ASTM D3803-1989. If laboratory test results are unacceptable, all charcoal adsorbents in the system shall be replaced with charcoal adsorbents qualified according to ASTM D3803-1989.



# UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

## SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 210 TO

FACILITY OPERATING LICENSE NO. DPR-51

ENTERGY OPERATIONS, INC.

ARKANSAS NUCLEAR ONE, UNIT NO. 1

**DOCKET NO. 50-313** 

### 1.0 INTRODUCTION

By letter dated November 23, 1999, as supplemented by letters dated February 24 and October 19, 2000, Entergy Operations, Inc. (the licensee), submitted a request for changes to the Arkansas Nuclear One, Unit No. 1 (ANO-1), Technical Specifications (TSs). The requested changes would incorporate the use of American Society for Testing and Materials (ASTM) D3803-1989, "Standard Test Method for Nuclear-Grade Activated Carbon," into the ANO-1 TSs.

The application was renoticed to include the February 24, 2000, supplement as indicated in Section 4.0 of this safety evaluation.

The October 19, 2000, supplemental letter provided clarifying information and revised Bases pages that did not change the scope of the Federal Register notice or the no significant hazards consideration determination.

## 2.0 EVALUATION

The Nuclear Regulatory Commission (NRC) staff, with technical assistance from Brookhaven National Laboratory (BNL), has reviewed the licensee's submittals. In addition, the staff has reviewed the attached BNL Technical Evaluation Report (TER) regarding the proposed TS changes for ANO-1. Based on its review, the staff adopts the TER. In view of the above, and because the NRC staff considers ASTM D3803-1989 to be the most accurate and most realistic protocol for testing charcoal in safety-related ventilation systems, the NRC staff finds that the proposed TS changes satisfy the actions requested in Generic Letter (GL) 99-02, "Laboratory Testing of Nuclear-Grade Activated Charcoal," dated June 3, 1999, and are acceptable.

The NRC received a letter from ASTM dated March 9, 2000, in response to a March 8, 2000, Federal Register Notice (65 FR 12286 - 12299) related to revising testing standards in accordance with ASTM D3803-1989 for laboratory testing of activated charcoal. ASTM notified the NRC that the 1989 standard is out of date and should be replaced by D3803-1991(1998). The staff acknowledges that the most current version of ASTM D3803 is ASTM D3803-1991 (reaffirmed in 1998). However, it was decided, for consistency purposes, to have all of the nuclear reactors test to the same standard (ASTM D3803-1989) because, prior to GL 99-02 being issued, approximately one third of nuclear reactors had TSs that referenced

ASTM D3803-1989 and there are no substantive changes between the 1989 and 1998 versions.

## 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Arkansas State official was notified of the proposed issuance of the amendment. The State official had no comments.

## 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding\* (65 FR 12291, dated March 8, 2000, and 65 FR 15378, dated March 22, 2000). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

## 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Attachment: Technical Evaluation Report, Brookhaven National Laboratory

Principal Contributor: J. Segala

Date: December 28, 2000

\* One public comment dated March 9, 2000, which was not related to the no significant hazards consideration finding, was received, and it has been addressed in Section 2.0 of this safety evaluation.

TECHNICAL EVALUATION REPORT
BROOKHAVEN NATIONAL LABORATORY
FOR THE OFFICE OF NUCLEAR REACTOR REGULATION
DIVISION OF SYSTEMS SAFETY AND ANALYSIS
PLANT SYSTEMS BRANCH
RELATED TO AMENDMENT TO FACILITY OPERATING LICENSE NO. DPR- 51
ENTERGY OPERATIONS, INC.
ARKANSAS NUCLEAR ONE - UNIT 1
DOCKET NO. 50 - 313

#### 1.0 INTRODUCTION

By letters dated November 23, 1999 (CNRO-99/00026), and August 2, 1999 (0CAN089902), Entergy Operations submitted its response to the actions requested in Generic Letter (GL) 99-02, "Laboratory Testing of Nuclear-Grade Activated Charcoal," dated June 3, 1999, for the Arkansas Nuclear One - Unit 1 (ANO-1). By the letter dated November 23, 1999, Entergy Operations requested changes to the Technical Specifications (TS) Sections 3.13.1.b and 3.15.1.b, covering the Penetration Room Ventilation System and the Fuel Handling Area Ventilation System, respectively, for ANO-1. By letter dated February 24, 2000, the licensee submitted supplemental revision to TS change request by deleting the specified test velocity from the specification. By letter dated October 19, 2000, Entergy Operations submitted additional information related to ventilation systems requested by the Plant Systems Branch and related to accident analyses requested by the Probabilistic Safety Assessment Branch. The proposed changes would revise the TS surveillance testing of the safety related ventilation system charcoal to meet the requested actions of GL 99-02.

#### 2.0 BACKGROUND

Safety-related air-cleaning units used in the engineered safety features (ESF) ventilation systems of nuclear power plants reduce the potential onsite and offsite consequences of a radiological accident by filtering radioiodine. Analyses of design basis accidents assume particular safety related charcoal adsorption efficiencies when calculating offsite and control room operator doses. To ensure that the charcoal filters used in these systems will perform in a manner that is consistent with the licensing basis of a facility, licensees have requirements in their TS to periodically perform a laboratory test (in accordance with a test standard) of charcoal samples taken from these ventilation systems.

By letter dated August 2,1999, Entergy stated that it will not be seeking a technical specification (TS) change for the Reactor Building Purge Filtration System on ANO-1. The basis for this is that this system is not used during power operation and provides no safety function for design basis accidents. ANO-1 TS 3.23.1 requires the purge valves in this system to be closed whenever containment integrity is required, and therefore would not provide a release path in the event of an accident. This system is only used to vent the reactor building during shutdown. The fuel handling accident in containment, as discussed in ANO-1 Safety Analysis Report (SAR) Section 14.2.2.3,

does not credit filtration in the release of radionuclides to the environment. Also, the safety evaluation for Amendment 195 (dated April 16, 1999) to the ANO-1 Operating License, which granted the ability to leave the reactor building equipment hatch open during fuel movement for potential fuel handling events inside containment, does not take credit for filtration in meeting Part 100 limits for potential fuel handling events inside containment. Therefore, the ANO-1 safety analysis does not credit the Reactor Building Purge Filtration System in meeting 10CFR100 accident dose limits. Entergy is also in the process of submitting proposed TS for ANO-1 that will be based on the Revised Standard TS Format of NUREG-1430, and states that one of the planned changes to be submitted to the NRC will be the deletion of the TS requirements for the Reactor Building Purge System. Based on review of the technical information provided by the licensee, there is no need to modify the charcoal testing requirements, per GL99-02, for this system.

In GL 99-02, the staff alerted licensees that testing nuclear-grade activated charcoal to standards other than American Society for Testing and Materials (ASTM) D3803-1989, "Standard Test Method for Nuclear-Grade Activated Carbon," does not provide assurance for complying with their current licensing bases with respect to the dose limits of General Design Criterion (GDC) 19 of Appendix A to Part 50 of Title 10 of the <u>Code of Federal Regulations</u> (10 CFR) and Subpart A of 10 CFR Part 100.

GL 99-02 requested that all licensees determine whether their TS reference ASTM D3803-1989 for charcoal filter laboratory testing. Licensees whose TS do not reference ASTM D3803-1989 were requested to either amend their TS to reference ASTM D3803-1989 or propose an alternative test protocol.

### 3.0 EVALUATION

#### 3.1 Laboratory Charcoal Sample Testing Surveillance Requirements

The current and proposed laboratory charcoal sample testing TS surveillance requirements for the Penetration Room Ventilation System (PRVS) and the Fuel Handling Area Ventilation System (FHAVS) are shown in Table 1 and Table 2, respectively, for ANO-1.

With respect to the Control Room Ventilation System (CRVS), since the current TS calls for laboratory charcoal testing in accordance with ASTM D3803-1989, ANO-1 is considered as a Group 1 plant under GL 99-02. Therefore, no TS amendment is warranted. On the basis of the information provided in letter dated October 19, 2000, the TS surveillance requirements for this system is also included in Table 1 and Table 2.

The proposed use of ASTM D3803-1989 is acceptable because it provides accurate and reproducible test results. The proposed test temperature of 30°C and relative humidity of 95% for both PRVS and FHAVS is acceptable because it is consistent with ASTM D3803-1989. This is consistent with the actions requested in GL 99-02.

By letter dated October 19, 2000, the credited removal efficiency for radioactive organic iodine for the PRVS and FHAVS is 90%. The proposed test penetration for radioactive methyl iodide for these two systems is less than 5%. The proposed test penetration was obtained by applying a safety factor of 2 to the credited efficiency. The proposed safety factor of 2 for both systems is

acceptable because it ensures that the efficiency credited in the accident analysis is still valid at the end of the surveillance interval. This is consistent with the minimum safety factor of 2 specified in GL 99-02.

The August 23, 1999 errata to GL 99-02 clarified that if the maximum actual face velocity is greater than 110% of 40 fpm, then the test face velocity should be specified in the TS. By letter dated February 24, 2000, the licensee deleted the  $\pm 20\%$  margin for the proposed test velocity in the TS. By letter dated October 19, 2000, the actual face velocities of the PRVS and FHAVS are 41.25 and 42.55 fpm, respectively. The proposed testing of the charcoal adsorbers will be performed in accordance with ASTM D3803-1989 which specifies a test face velocity of 40 fpm with appropriate margins. This is acceptable because it ensures that the testing will be consistent with the operation of the ventilation system during accident conditions. Therefore, it is not necessary to specify the face velocity in the proposed TS change. This is consistent with the errata to GL 99-02 dated August 23, 1999.

#### 4.0 CONCLUSION

On the basis of its evaluation, BNL recommends that the NRC staff consider the proposed TS changes to be acceptable.

Principal Contributors: Richard E. Deem and Mano Subudhi

Date: November 29, 2000

## **ARKANSAS NUCLEAR ONE - UNIT 1**

|                | TABLE 1 - CURRENT TS REQUIREMENTS                         |                              |                                   |                           |  |   |                         |                              |                      |                   |                                    |  |  |
|----------------|---|------------------------------|-----------------------------------|---------------------------|--|---|-------------------------|------------------------------|----------------------|-------------------|------------------------------------|--|--|
|                | System Description  |                              |                                   |                           |  |   | Current TS Requirements |                              |                      |                   |                                    |  |  |
| TS<br>Section  | System  | Bed<br>Thickness<br>(inches) | Actual (<br>Res.<br>Time<br>(sec) | Face<br>Velocity<br>(fpm) | Credited Efficiency (% organic iodine) | Test<br>Penetration<br>(% methyl<br>iodide) | Safety<br>Factor        | Test<br>Standard             | Test<br>Temp<br>(°C) | Test<br>RH<br>(%) | Test<br>Face<br>Velocity<br>(fpm)  |  |  |
| 4.10.2.<br>b.2 | Control Room<br>Ventilation<br>System<br>(CRVS)*          | 2                            | 0.2182                            | 45.83                     | Not<br>stated                          | ≤2.5/<br>2" bed<br>≤0.5/<br>4" bed          | Not<br>stated           | ASTM D3803-<br>1989          | 30                   | 95                | Not<br>stated<br>(40)***           |  |  |
| 3.13.1.<br>b   | Penetration<br>Room<br>Ventilation<br>System<br>(PRVS)    | 2                            | 0.242                             | 41.25                     | 90                                     | <10%  | Not<br>stated<br>(1)**  | RDT M 16-1T<br>(in TS Bases) | ≥87.8<br>(190°F)     | ≥95               | 20% of<br>system<br>design<br>flow |  |  |
| 3.15.1.<br>b   | Fuel Handling<br>Area<br>Ventilation<br>System<br>(FHAVS) | 2                            | 0.235                             | 42.55                     | 90                                     | <10%  | Not<br>stated<br>(1)**  | RDT M 16-1T<br>(in TS Bases) | ≥51.7<br>(125°F)     | ≥70               | 20% of<br>system<br>design<br>flow |  |  |

<sup>\*</sup> With respect to the Control Room Ventilation System, ANO Unit 1 is considered as a Group 1 plant under GL 99-02.

<sup>\*\*</sup> Safety factor is calculated based on the credited efficiency and test penetration.

<sup>\*\*\*</sup> Test face velocity is in accordance with ASTM D3803-1989 requirements.

## **ARKANSAS NUCLEAR ONE - UNIT 1**

|                | TABLE 2 - PROPOSED TS REQUIREMENTS                        |                              |                       |                           |                    |                                    |                          |                  |              |            |                         |  |  |
|----------------|---|------------------------------|-----------------------|---------------------------|--------------------|------------------------------------|--------------------------|------------------|--------------|------------|-------------------------|--|--|
|                | System Description  |                              |                       |                           |                    |                                    | Proposed TS Requirements |                  |              |            |                         |  |  |
|                | 1   | Bed<br>Thickness<br>(inches) | Actual Charcoal       |                           | Efficiency         |                                    | Safety<br>Factor         | Test<br>Standard | Test<br>Temp | Test<br>RH | Test<br>Face            |  |  |
| TS<br>Section  |   |                              | Res.<br>Time<br>(sec) | Face<br>Velocity<br>(fpm) | (methyl<br>iodide) | (methyl<br>iodide)                 |                          |                  | (°C)         | (%)        | Velocity<br>(fpm)       |  |  |
| 4.10.2.<br>b.2 | Control Room<br>Ventilation<br>System<br>(CRVS)*          | 2                            | 0.2182                | 45.83                     | Not<br>stated      | ≤2.5/<br>2" bed<br>≤0.5/<br>4" bed | Not<br>stated            | ASTM D3803-1989  | 30           | 95         | Not<br>stated<br>(40)** |  |  |
| 3.13.1.<br>b   | Penetration<br>Room<br>Ventilation<br>System<br>(PRVS)    | 2                            | 0.242                 | 41.25                     | 90                 | <5.0%                              | 2                        | ASTM D3803-1989  | 30           | 95         | Not<br>stated<br>(40)** |  |  |
| 3.15.1.<br>b   | Fuel Handling<br>Area<br>Ventilation<br>System<br>(FHAVS) | 2                            | 0.235                 | 42.55                     | 90                 | <5.0%                              | 2                        | ASTM D3803-1989  | 30           | 95         | Not<br>stated<br>(40)** |  |  |

With respect to the Control Room Ventilation System, ANO Unit 1 is considered as a Group 1 plant under GL 99-02. Test face velocity is in accordance with ASTM D3803-1989 requirements.