



South Texas Project Electric Generating Station P.O. Box 289 Wadsworth, Texas 77483

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U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
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South Texas Project
Units 1 and 2
Docket Nos. STN 50-498, STN- 50-499
10CFR50.59 Summary Report

Pursuant to the requirements of 10CFR50.59, the attached report contains a brief description and summary of the safety evaluations of changes, tests, and experiments conducted at the South Texas Project.

If there are any questions regarding this summary report, please contact Ms. M. K. Johnson at (361) 972-8385 or me at (361) 972-7206.

A handwritten signature in black ink, appearing to read "M. A. McBurnett", is written over a horizontal line.

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Attachment

IE47

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Brief Description of Changes, Tests, and Experiments

Unreviewed Safety Question Evaluation 95-4343-39

Description: Replace reactor cooling pump vibration monitoring indicating meters and selector switches with a Human Machine Interface that provides indication of all vibration parameters.

Safety Evaluation Summary: The Integrated Vibration Monitoring System provides enhanced capabilities for monitoring, alarming, trending and diagnostics, improving pump reliability and availability. Therefore, because monitoring features as described are maintained in substance and enhancements result in an overall increase in capabilities, the change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 95-5319-13

Description: Change the type of trolley, hoist, and bridge motors in the refueling machine control system. Install a programmable logic controller to control the refueling machine instead of the existing electrical interlocks. Add a switch to allow bypassing the upper slow zone.

Safety Evaluation Summary: The change does not impact the operability requirements or reduce the margin of safety as defined in the basis for any Technical Specification. Therefore, the change does not involve an Unreviewed Safety Question.

Unreviewed Safety Question Evaluation 95-11217-14

Description: The units have experienced spurious rod control stepping due to hot leg streaming during steady state operation when the rod control system is in the automatic control mode. Change the lead/lag/lag values for the automatic rod control circuit for the T_{avg} auctioneered signal.

Safety Evaluation Summary: Westinghouse has reviewed the accident analysis for all applicable events and provided a margin to trip and operability analysis. The new lead/lag/lag values do not impact current accident analyses, reduce the margin of safety provided by the reactor protection system and engineered safety features (ESF), or impact rod control system stability. The analysis performed with the revised assumptions is acceptable and does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 96-1619-74

Description: Upgrade each SDG fuel oil storage tank (FOST) level instrument loop.

Safety Evaluation Summary: This change eliminates FOST level measurement uncertainties associated with fuel oil density variations. Physical installations are designed to address Seismic

II/I impact. Electrical changes are in accordance with approved site specifications and procedures, and address Reg. Guide 1.75 separation requirements. This change does not impact the function or operation of any safety-related equipment or activity in the plant. It does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 96-2845-27

Description: As part of Unit 1 steam generator replacement, a portion of the "D" steam generator biological shield wall in front of the Reactor Containment Building (RCB) equipment hatch will be cut and reattached to the adjoining wall by steel splice plates and through-bolts. Some commodities will be temporarily removed and dust control, debris and water/slurry collection devices installed.

Safety Evaluation Summary: The changes do not result in any new accidents, an increase in consequences of existing accidents, any adverse effects on equipment important to safety, or changes in the bases for Technical Specifications. The new design meets the original design requirements. These changes do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 96-2847-6

Description: As part of Unit 1 steam generator replacement, the insulation on the existing steam generators (SG) and on piping segments of the reactor coolant (RC) system, main steam (MS) system, feedwater (FW) system, auxiliary feedwater (AF) system, steam generator blowdown (SB) system, and SG level instrumentation lines will be removed. New insulation will be installed on the replacement SGs, on the FW, AF, and SB Systems, and on SG level instrumentation lines. Insulation removed from the RC and MS systems will be reinstalled. Rev 1 allows removal of steam generator insulation upon entering Mode 5 without declaring the steam generators "out of service."

Safety Evaluation Summary: The replacement insulation will satisfy existing design requirements. These modifications will not result in a change to the containment temperature or pressure design conditions. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 96-9330-5

Description: Add flow and pressure gauges to the essential chilled water system to facilitate Section XI testing.

Safety Evaluation Summary: Adding these gauges and leaving their root valves normally closed does not result in an unreviewed safety question.

Unreviewed Safety Question Evaluation 96-10998-11

Description: Replace Fuel Transfer System chain-drive system with a cable-drive system and replace control panels, proximity switches, and up-ender hydraulic control valve. Modification also provides for remote operation of the system from the refueling machine.

Safety Evaluation Summary: System upgrade does not affect equipment that is important to safety. The proposed modification does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 96-12589-2

Description: Determine if operating with an idle Component Cooling Water (CCW) train and a running Essential Cooling Water (ECW) train is acceptable.

Safety Evaluation Summary: Operating in this manner does not prevent the CCW system from performing its design ESF support functions. The CCW system provides an effective intermediate barrier between the ECW system and potentially radioactive systems. Leakage of ECW into the CCW system can be identified and corrected before degradation of system materials occurs. No changes to the Technical Specifications are required and no unreviewed safety question exists.

Unreviewed Safety Question Evaluation 97-0005

Description: Insulate a 3" liquid waste processing system line inside containment.

Safety Evaluation Summary: The decrease in available heat sink surface area inside containment is enveloped by margins included in the UFSAR. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 97-0022

Description: Eight core exit thermocouples (CETs) in Unit 2 are currently not capable of providing temperature information.

Safety Evaluation Summary: The CET system has four operable thermocouples per channel per quadrant, which meets UFSAR and SER statements. There is no unreviewed safety question involved with this condition.

Unreviewed Safety Question Evaluation 97-0044

Description: Split technical support center (TSC) HVAC chillers into two separate chilled water plants and abandon the air pre-heater for the TSC charcoal filter unit.

Safety Evaluation Summary: The final TSC chilled water system configuration provides two independent 100% chillers per unit. The change does not result in any area temperature

exceeding its design requirement. There is no impact on the dose analysis for personnel inside the TSC. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 97-0046

Description: Reset ATWS mitigation system actuation circuitry interlock C-20 from 40% turbine impulse pressure to 30%.

Safety Evaluation Summary: The design requirement is to ensure AMSAC is activated above 40%. This change activates AMSAC at 30%. Therefore, the change is conservative and consistent with the NRC safety evaluation that accepted STP design. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 97-8008-3

Description: Replace rod cluster control assemblies (RCCAs) with an “enhanced performance (EP)” type.

Safety Evaluation Summary: EP-RCCAs are provided as “fit, form, function” direct replacements by Westinghouse. The slight design changes in the EP-RCCAs do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 97-9644-4

Description: Remove statements in UFSAR Section 15.6.2 that sample lines are only open during sampling and that loss of flow at the sample panel indicates that a break has occurred. Clarify that isolation valves may also be open for line purging and degassing.

Safety Evaluation Summary: Using the system for purging, sampling, and degassing is within the system design bases and intended system function. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 97-11955-2

Description: Revise Operation Quality Assurance Plan (OQAP) to include ASME Class MC and CC components in the Inspection/Examination and Repair/Replacement ASME Section XI Program.

Safety Evaluation Summary: This is an addition to our current commitment to facilitate upcoming activities for these components and does not represent a change that introduces any adverse safety impact. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 97-15781-9

Description: Reload Safety Evaluation for Unit 2 Cycle 7, Modes 1 and 2.

Safety Evaluation Summary: Operation of Unit 2 Cycle 7 to a total burn-up (including coastdown) of 373.5 effective full power days does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 97-15781-35

Description: Revise UFSAR Section 15.6.5 description of the large break LOCA due to revised analysis caused by robust fuel assemblies.

Safety Evaluation Summary: The acceptance criteria of 10CFR50.46 are still satisfied and there is no reduction in the margin of safety. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 97-15781-40

Description: Determine impact of incomplete rod insertion on the safety analysis and if this condition involves an unreviewed safety question for Unit 2 Cycle 7.

Safety Evaluation Summary: The Safety Analysis provides bounding results with respect to the Reload Safety Analysis Checklist. Failure of the rod cluster control assemblies to fully insert to the rod bottom position does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 97-17779-5

Description: Remove reactor coolant pump (RCP) oil changing system inside containment isolation valve, adjacent piping and supports inside containment, weld on two caps, and abandon the outside containment isolation valve in place.

Safety Evaluation Summary: The capped penetrations have no interaction with any safety related system, structure, or component. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 97-17793-2

Description: Control of Heavy Loads Procedure revisions: add clarification for loads < 2500 lbs, safe shutdown definition, use of RCB jib cranes, movement over pressurizer, polar crane interlock bypass, restriction for Fuel Handling Building (FHB) jib crane, and approval before moving Mechanical Auxiliary Building (MAB) floor plugs.

Safety Evaluation Summary: Revisions are in accordance with the evaluation guidelines presented in the previously approved submittal to the NRC in October 1984 concerning Control of Heavy Loads. The proposed changes to the procedure maintain restrictions to protect spent fuel and safe shutdown equipment from a load drop accident. Thus the changes are acceptable and do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 97-18267-9

Description: Designate the refurbished Cement Unloading Building as the Refueling Equipment Building (REB).

Safety Evaluation Summary: Using the REB for tool and equipment storage is acceptable since it does not impact any safety-related systems, structures, or components. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0003

Description: Install sixteen ASTM A36 baseplates on the steam generator side of the secondary shield walls for pipe supports associated with feedwater lines rerouted due to steam generator replacement.

Safety Evaluation Summary: Baseplates are designed to meet Seismic II/I criteria and will be installed in accordance with approved procedures. Calculation MC-6477 will be revised to include the commodity changes. There is no impact on hydrogen generation or containment pressure/temperature (P/T). This modification does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0012

Description: Store the four steam generators removed from Unit 1 in the Old Steam Generator Storage Facility (OSGSF), a new reinforced concrete and steel structure constructed outside the protected area, but within the exclusion area and site boundary.

Safety Evaluation Summary: Construction and operation of the OSGF does not result in any new accidents, increase in consequences of existing accidents, any adverse effects on equipment important to safety, or changes in the Technical Specifications. This modification does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0014

Description: Use a temporary reverse osmosis system as needed to clean silica from the contents of the Refueling Water Storage Tank (RWST).

Safety Evaluation Summary: The temporary system is connected to a non-safety related portion of the Spent Fuel Pool Cooling and Cleanup System and will not challenge the safety-related functions or performance of plant equipment. The water volume of the RWST is constantly monitored by level alarm instrumentation and is surveilled in accordance with Technical Specifications. The boron concentration of water returned to the RWST is monitored to ensure that the RWST minimum boron concentration is maintained and that Spent Fuel Pool boron concentration commitments of Justification for Continued Operation 940005 are maintained. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0015

Description: Permanently connect a dehumidifier unit to auxiliary boiler #11 as an enhancement to provide dehumidified air to the firebox while the boiler is in lay-up.

Safety Evaluation Summary: The auxiliary boiler is classified as non-nuclear safety and non-seismic. Loss of the functional capability of the boiler will not preclude safe shutdown of the plant. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0021

Description: Replace standby diesel generator (SDG) governors with new model.

Safety Evaluation Summary: The replacement governors are seismically qualified and perform the same safety functions as the old governors. This replacement does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0026

Description: Modify blowdown system piping and supports to accommodate fit-up to the revised blowdown system nozzle locations on the replacement steam generators.

Safety Evaluation Summary: The modification does not change, degrade or prevent actions described or assumed in any accident or transient evaluated in the UFSAR. Upon completion, all changes addressed in this proposed change restore conditions to comply with the original design basis. This modification does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0027

Description: As part of the Unit 1 steam generator replacement, a portion of MS, FW, and AF will be temporarily disconnected. Some permanent modifications will be made to accommodate new nozzle locations, meet pipe stress criteria, increase erosion-corrosion resistance, and provide access for construction inspections. The FW and AF piping reroutes require some interfering commodities to be relocated.

Safety Evaluation Summary: The piping will be reinstalled to satisfy existing design requirements in accordance with ASME Sections III and XI. There are no modifications that result in a change to the dose mitigating functions of the affected systems and commodities, in any new accident initiators or in any adverse effect on equipment important to safety. This replacement does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0028

Description: Reviews SG replacement activities associated with 1.) vessel preparatory work; 2.) design, installation and removal of temporary restraints for RCS piping; 3.) design, installation

and removal of temporary SG restraints; 4.) cutting, machining, welding and non-destructive examination of RCS piping; 5.) decontamination of severed RCS pipe ends; and 6.) removal and installation of the SG permanent supports, including the design, removal and installation of the SG and RCP support shims. Reviews impact of work activities performed during Modes 5 and 6 and activities related to the three RCS cut method of replacement should new crossover leg elbows be used.

Safety Evaluation Summary: The work will be performed in accordance with approved plant procedures, uses materials consistent with original material requirements, and restores the plant to a condition conforming to the original design basis. Work activities during Modes 5 and 6 do not impact Technical Specification requirements related to maintenance of decay heat removal capability, maintenance of containment integrity and maintenance of other safety-related systems, structures and components required to be operable during Modes 5 and 6 by Technical Specifications. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0030

Description: Remove the Reactor Cavity Filtration System

Safety Evaluation Summary: The system is non-safety related, and is mechanically and electrically isolated during power operation. Removal of this equipment will not impact and safety-related equipment or system. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0031

Description: Install a flow meter with a micrometer control valve on the condensate pump suction header test valve. The flow is used to adjust the condensate oxygen concentration to specific values within STP and EPRI Specifications.

Safety Evaluation Summary: The condensate pump suction header test valves are not a safety feature and their use to adjust condensate oxygen does not adversely impact plant safety. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0032

Description: Reduce time to hot leg switchover from 6.5 hours to 5.5 hours after initiation of LOCA.

Safety Evaluation Summary: Hot leg switchover ensures that boron precipitation does not occur. Reducing the time ensures that the design basis is met. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0033

Description: Remove plant annunciator system electronics and data acquisition/signal processing from the Emergency Response Facility Data Acquisition and Display System (ERFDADS) and replace it with the Integrated Control System (ICS)-ERFDADS Subsystem.

Safety Evaluation Summary: The plant annunciator system is non-safety related. Replacement of the existing annunciator system with a state-of-the-art computer system meets or exceeds the requirements of the existing system and does not introduce an adverse safety impact. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0036

Description: To facilitate replacement of the Unit 1 steam generators, the Outside Lift System (OLS)/runway foundations in the yard area outside the RCB equipment hatch may be installed prior to the beginning of the outage. After the outage, the foundations for these structures, one of which ties into the existing foundation for the tower structure for RCP motor removal will be abandoned in place. Components or commodities interfering with placement of the new foundations will be rerouted, temporarily removed, and restored or embedded.

Safety Evaluation Summary: Placement of the OLS/runway foundations in the yard outside the RCB equipment hatch satisfy existing design requirements. There are no modifications within the scope of this change that result in any new accident initiators or in any adverse effects on equipment important to safety. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0043

Description: Delete SDG fuel oil storage tank (FOST) level indication and high level alarm functions at the filtration skid. Delete low level alarm functions from local control panel annunciator and main control room panel annunciator.

Safety Evaluation Summary: Operating experience has proven the subject instruments provide ambiguous level indication and alarm functions. This change does not impact the function or operation of any safety-related equipment or activity in the plant. FOST level instruments remaining after this change provide adequate information to prevent violating Technical Specification minimum level requirements and to prevent overflow of the tanks. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0044

Description: Install alarms and interlock functions for steam generator blowdown demineralizer inlet temperature to protect demineralizer resin bed from over-temperature conditions.

Safety Evaluation Summary: This change is in the non-safety related portion of the system. The change is consistent with demineralizer high temperature protection discussed in UFSAR Section 10.4.8.4. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-0045

Description: As part of Unit 1 steam generator replacement, steam generator water level instrumentation system piping/tubing will be modified to accommodate rerouted FW and AF piping and the slightly higher instrumentation taps on the replacement steam generators. This evaluation reviews the UFSAR change adding the description of the PS+CAEPIPE computer code.

Safety Evaluation Summary: The engineering design and qualification of the modifications to the steam generator water level instrumentation piping/tubing are performed in accordance with the plant design basis. The work to remove and install the steam generator water level instrumentation system piping/tubing is performed to approved plant procedures, uses materials consistent with original material requirements and restores the plant to a condition in conformance with the original design basis. Work activities during Modes 5 and 6 do not impact Technical Specification requirements related to maintenance of decay beat removal capability, maintenance of containment integrity and maintenance of other safety related systems, structures and components required to be operable during Modes 5 and 6 by Technical Specifications. No unreviewed safety questions result from this evaluation.

Unreviewed Safety Question Evaluation 98-243-024

Description: Change Pressurizer Relief Tank (PRT) temperature alarm from 114 to 90 degrees. Change UFSAR initial PRT analytical temperature from 120 degrees to 95 degrees. Change PRT High Level Alarm from 80 to 76%. Change PRT Low Level Alarm from 69 to 64%.

Safety Evaluation Summary: The PRT will remain at or below 200 degrees and at or below 50 psig and the rupture disc will not be deformed. Therefore, this is not an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-622-18

Description: Revise UFSAR to reflect actual essential cooling water (ECW) design basis temperature for SDGs and jacket water heat load due to removal of intercooler preheater thermostatic control valves.

Safety Evaluation Summary: The revised ECW temperature is that which was actually used in plant design and licensing basis analysis. Operation with higher heat loads is acceptable because engine temperatures remain within design limits. These changes do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-1690-4

Description: Revise procedure to invite offsite fire departments to participate in a drill annually.

Safety Evaluation Summary: This a change to the UFSAR, but has no impact on the plant design basis. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-1786-1

Description: UFSAR change notice includes revisions to the steam generator physical description; changes in the materials of construction; changes in the loads, pipe stresses, and quality assurance plan for the replacement steam generators; and new computer codes used in the pipe stress analysis.

Safety Evaluation Summary: The proposed changes demonstrate that the replacement steam generators continue to comply with codes, standards, and regulatory requirements. The changes do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-2394-3

Description: Add flange connections the above- and below-seat drain lines of the turbine-driven auxiliary feedwater pump trip and throttle valve to improve leak detection during insitu seat leakage testing of the valve.

Safety Evaluation Summary: The change will not increase the potential of an accident or the malfunction of plant equipment important to safety. The reason for this USQE is to document changes to P&IDs that are part of the UFSAR. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-2460-3

Description: Clarify Condensate Polisher system description in UFSAR Section 10.4.6.2.

Safety Evaluation Summary: The proposed changes are administrative in nature and do not involve a physical alteration of the plant. The design and function of the system are not changed and there is no adverse safety impact. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-3213-3

Description: Conduct a special test to demonstrate that the qualified display processing system (QDPS) is "Year 2000 Ready" during an outage when the unit is defueled and QDPS is not required by Technical Specifications or operating procedures. This test may also be conducted in Mode 6 when precautions are taken to protect the required Residual Heat Removal pumps by testing the Auxiliary Processing Cabinet (APC) that does not have an RHR pump (APC-D1).

Safety Evaluation Summary: The remaining seven APCs will be unaffected and thus there will be no RHR low flow trip caused by de-energizing QDPS. The QDPS systems will be restored to “operable” prior to entering leaving Mode 6 with the Reactor Pressure Vessel head removed and water level at the refueling level. Therefore, this test does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-3839-5

Description: A temporary vendor chemical injection skid will discharge into the Unit 2 side of the open loop auxiliary cooling water (OLACW) pump discharge. Tubing, valves, and fittings will be extended to the pump discharge drain valve to allow injection of molluscicide 12 hours prior to shutdown of Unit 2 for 2RE06.

Safety Evaluation Summary: The OLACW systems are non-safety related. The unit is pre-engineered equipment supplied by Nalco that includes a chemical storage container, a pump mounted on a skid, and polyethylene containment basin (berm). This chemical process does not change any evaluation, component or basis in the UFSAR. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-4555-7

Description: Revise steam generator subcompartment analysis to remove existing surge line and accumulator line results. Add additional break cases to the design basis of the structures, systems and components within the secondary shield wall and the secondary shield walls themselves. Revise methodologies to evaluate break mass and energy release rates and subcompartment pressure responses.

Safety Evaluation Summary: The line break accident is already described in the UFSAR. All design criteria remain satisfied. No new failure modes or operator actions are introduced and the existing dose remains bounding. The revised analysis does not constitute an Unreviewed Safety Question.

Unreviewed Safety Question Evaluation 98-4896-2

Description: Change the unit cross-connect valves for instrument air (IA) and service air (SA) from IA1498 and SA1121 to IA1088 and 2-SA0030, which are more accessible to the operators.

Safety Evaluation Summary: Neither of these systems is safety-related nor supplies air for any safety-related function. The systems will operate identically after the valve functions are changed. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-5155-3

Description: Clarify UFSAR Section 7A.S8 to state that the emergency mode of the emergency operation facility (EOF) HVAC system is a pressurization lineup (100% outside HEPA filtered air) with excess cooling/heating air shunted to the ceiling.

Safety Evaluation Summary: The system is located in a building outside the protected area and the system does not support the plant. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-5559-41

Description: Add ultrasonic feedwater flow (UTF) monitoring system as an enhancement to compensate the existing venturi-based flow rate to overcome the effects of corrosion.

Safety Evaluation Summary: The UTF device is an enhancement for the FW system. Seismic, environmental, separation, implementation, and reliability issues have been previously addressed and have not changed. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-5910-9

Description: Revise UFSAR Section 6.2.1.1.3.6 and Table 6.2.1.1-2 to change the calculated maximum external containment overpressure from 2.92 psid to 3.1 psid due to changes in the instrument uncertainty of the containment pressure measurement instrument.

Safety Evaluation Summary: The containment external design pressure of 3.5 psid remains bounding. No change to the Technical Specifications is required and no unreviewed safety question is involved.

Unreviewed Safety Question Evaluation 98-6819-1

Description: Allow using a temporary mobile laundry system to support refueling outages.

Safety Evaluation Summary: STP has evaluated the placement of this temporary facility and imposed limitations to assure issues of safety, effluents, and site-radiological controls are maintained at the current high standards. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-8475-9

Description: Reload Safety Evaluation for Unit 1 Cycle 9, Modes 1-5

Safety Evaluation Summary: Operation of Unit 1 Cycle 9 to a total cycle burnup (including coastdown) of 335 effective full power days does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-8475-30

Description: Revise the design of the fuel pellets (annular pellets) in fuel rods containing Integral Fuel Burnable Absorber (IFBA) fuel. Annular pellets in Unit 1 Cycle 9 will not be used in fuel rods without IFBA.

Safety Evaluation Summary: The modified fuel pellet design is acceptable and does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-8475-36

Description: Determine impact of incomplete rod insertion on the safety analysis and determine if this condition involves an unreviewed safety question for Unit 1 Cycle 9.

Safety Evaluation Summary: The Safety Analysis provides bounding results with respect to the Reload Safety Analysis Checklist. Failure of the rod cluster control assemblies to fully insert to the rod bottom position does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-8831-5

Description: Temporary modification allows hydrazine to be injected into a high pressure feedwater heater outlet vent instead of into the existing injection point at the effluent of the condensate polishers.

Safety Evaluation Summary: The plant chemistry requirements are unaltered by this temporary modification. Changing the hydrazine injection point will reduce flow-accelerated corrosion in the condensate system while maintaining plant chemistry requirements. It does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-8993-99

Description: Replacement steam generator return-to-service tests have different methods than similar tests in UFSAR Chapter 14. These involve the water level control test, the load swing test, and the large load reduction test.

Safety Evaluation Summary: The changes in test methods are within the design of the plant and are bounded by existing UFSAR Chapter 15 safety analyses, and therefore do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-9594-7

Description: During refueling outages, use a seal leak-off reclamation system to collect refueling water leaking past the reactor coolant pump backseat and return it to the refueling cavity.

Safety Evaluation Summary: Reactor coolant grade borated water is conserved with no adverse impact to safe operation or equipment important to safety. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-10891-1

Description: Several tests/examinations that have not been used before will be performed on discharged fuel assemblies in the Unit 1 spent fuel pool.

Safety Evaluation Summary: Fuel assembly inspection in accordance with Westinghouse Procedure STD-FP-1998-8171, Rev. 0 does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-11741-1

Description: In the Fire Hazards Analysis report (FHAR), revise the Maximum Permitted Fire Loads to be consistent with the barriers installed, add the definition for a Maximum Permitted Fire Load, and add transient fire loading to allow for routine maintenance.

Safety Evaluation Summary: Fire loads described by this change are bounded by the original fire zone boundaries, or values previously evaluated in the FHAR. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-12148-2

Description: Revise UFSAR Tables 6.5-3 and 6.5-4 for RCS mass, refueling water storage tank (RWST) deliverable volume, accumulator water volume, resultant solution pH, and boundary conditions.

Safety Evaluation Summary: The minimum and maximum pH remains within the allowable range of 7.0 to 9.5 reported in the UFSAR and SER. Therefore there is no adverse safety impact and this change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-12223-4

Description: Temporary modification installs a slip blind in the ductwork downstream of the MAB Supply Fan discharge damper to allow rework of the damper assembly in place.

Safety Evaluation Summary: This modification does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-12879-2

Description: Assess the mechanical changes associated with loading robust fuel assemblies, which will strengthen the fuel assemblies, reduce guide tube distortion, and preclude incomplete rod insertion.

Safety Evaluation Summary: This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-13132-2

Description: Control of Heavy Loads Procedure: revise safe load paths (SLPs) to allow movement over pressurizer when primary system is depressurized; revise SLPs for RCP motor replacement, inservice inspection rig, in-containment storage area (ICSA) gate, and jib crane installation; clarify parking requirements for polar crane.

Safety Evaluation Summary: Revisions are in accordance with the evaluation guidelines presented in the previously approved submittal to the NRC in October 1984 concerning Control of Heavy Loads. The proposed changes to the procedure maintain restrictions to protect spent fuel and safe shutdown equipment from a load drop accident. Thus the changes are acceptable and do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-13786-6

Description: Revise OQAP to reflect organization changes.

Safety Evaluation Summary: These changes to the OQAP do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-13836-2

Description: Evaluate non-compliance with a UFSAR commitment in that a small number of fuel rod bottom end-plug girth welds were not inspected in accordance with a description in the UFSAR.

Safety Evaluation Summary: Consideration of the multiple barriers involved, design bases, weld process consistency, and conservative and overlapping inspections techniques, leads to the conclusion that it is very unlikely that there are any fuel rods in either core with an unacceptable bottom end plug girth weld. This event does not represent an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-15031-2

Description: Add information to UFSAR Table 3.12-1, Table 9.4-4.1, and the response to Q321.4: the HEPA filter installed in the RCB Supplemental Purge Exhaust and in the Radioactive Header Vent was not designed, tested or installed to meet RG 1.140 and ANSI N509/N510 requirements.

Safety Evaluation Summary: The HEPA filters were provided to prevent the migration of particulate to the unit vent from reaching the MAB roof. The filters are not required to meet plant 10CFR100 requirements. They are not credited in any DBA dose calculation. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-15054-5

Description: Numerous safety-related transmitters are being replaced to improve performance and to solve product obsolescence. WCAPs 11273 and 11488 are being revised to reflect the changes and the UFSAR must be revised to reflect the WCAP revision numbers.

Safety Evaluation Summary: This UFSAR revision does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-16327-10

Description: Revise the UFSAR to address methods used in accounting for the DNB penalty associated with the loop Tav_g asymmetry issue identified in Westinghouse Technical Bulletin 96-07.

Safety Evaluation Summary: The methods used to address loop Tav_g asymmetry are acceptable and do not represent an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-17560-3

Description: Add new UFSAR Section 9.2.2.3.3 that describes extended outages of spent fuel pool cooling due to maintenance of the CCW system.

Safety Evaluation Summary: CCW maintenance outage is performed such that the SFP licensing basis temperature limit of 150.7°F is not exceeded. No changes to the Technical Specifications are required and no unreviewed safety question is involved.

Unreviewed Safety Question Evaluation 98-18127-15

Description: Replace pressure switches in the closed loop auxiliary cooling water supply to the IA and SA compressors with annubar flow sensors and differential pressure switches.

Safety Evaluation Summary: All of the systems affected by this change are non-safety related. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-19225-2

Description: Revise UFSAR to allow flexibility in the way chemicals are added to the primary, secondary, chilled water, and cooling water systems for effective chemistry control.

Safety Evaluation Summary: The changes will not degrade the performance of or increase challenges to the RCS or any safety-related systems assumed to function in accident analyses. Therefore, there is no unreviewed safety question involved with these changes.

Unreviewed Safety Question Evaluation 98-19641-4

Description: Revise the Offsite Dose Calculation Manual with regards to lower limit of detection calculations, soil sampling, sewage sludge land farming, and sampling frequencies.

Safety Evaluation Summary: The proposed changes do not reduce the control of radioactive effluents required by 10CFR20.1302, 40CFR190, 10CFR50.36a, and Appendix I to 10CFR50. The changes will not impact the accuracy or reliability of effluent, dose, or set point calculations. Therefore, the changes do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-19701-1

Description: Delete the requirement to monitor horizontal benchmarks and reduce monitoring frequency for site piezometers in UFSAR Table 2.5.C-1. Eliminate UFSAR geotechnical information updates in Section 2.5.1.2.

Safety Evaluation Summary: Experience shows the changes in geotechnical parameters are within the assumptions and calculated values and that these parameters change at a slow rate. There are no adverse trends in the data that support the frequent monitoring. Monitoring local subsidence and fault movement is better acquired through vertical benchmark deflections. These changes do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 98-20456-6

Description: Manual test methodology used for testing portions of the Solid State Protection System differs from method described in the UFSAR.

Safety Evaluation Summary: This alternate methodology of testing circuits still provides for proper evaluation of the logic circuits being tested without affecting, influencing, or damaging the circuitry. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-0066-13

Description: Supercede UFSAR Sections 6.2A1.1.3.1 and 6.2A.1.4 that describe the containment main steam line break (MSLB) P/T response analysis for the Delta 94 steam generators.

Safety Evaluation Summary: No new source terms or release paths are introduced and the margin of safety is not reduced. This change is not involved with an unanswered safety question.

Unreviewed Safety Question Evaluation 99-0066-20

Description: Revise containment LOCA hydrogen generation analysis to incorporate changes in the containment LOCA P/T analysis.

Safety Evaluation Summary: The revised analysis indicates a more rapid rise in hydrogen concentration than the original analysis, but the conclusions regarding initiation of hydrogen recombination and maximum calculated concentration remain in effect. All acceptance limits continue to be satisfied. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-0066-45

Description: Revise UFSAR Section 15.2.8 and Table 7A.11.E.1.1-2a for the Delta 94 steam generators. The feedwater system pipe break analysis was revised to correct an error in the reactor coolant pump homologous curves.

Safety Evaluation Summary: The revised safety analysis demonstrates the acceptance criteria are met; the peak pressure of the RCS and main steam system (MSS) is not challenged; and the pressurizer does not go water solid prior to operator action time. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-196-2

Description: Revise UFSAR Section 15.5.2 to reflect changes in the assumptions in the chemical and volume control system (CVCS) malfunction analysis.

Safety Evaluation Summary: In the event of a CVCS malfunction that increases in reactor coolant inventory, the pressurizer will not go water solid prior to the 10 minute operator response time allowed to mitigate the event. The analysis performed with the revised assumptions is acceptable and does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-218-3

Description: Revise rack calibration allowance design input to reflect present TADOT setting tolerance of under-frequency trip device 81. Revise instrument span of device 81. Revise WCAP-11273 to resolve conflicting setpoint basis and uncertainty documentation.

Safety Evaluation Summary: The changes identified do not require a physical change to plant systems, structures or components. The current LSSS setpoints and operation of the Reactor Protection System (RPS) are maintained and these changes do not degrade the performance of the RPS, or its operation as assumed in the accident analysis. The changes do not impact Technical Specifications. Therefore, this does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-546-2

Description: Revises the OQAP to reflect organization changes.

Safety Evaluation Summary: This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-1269-3

Description: Temporary cooling towers will provide cooling for RCB and MAB chiller condensers while steam generators are moved into containment during replacement.

Safety Evaluation Summary: The chillers are not accident initiators and do not provide cooling to operating accident-mitigating equipment. This change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-1371-3

Description: Control of Heavy Loads Procedure revisions: allow use of jib cranes for certain heavy loads; add foreign object search and retrieval box and Tri-Nuke box to list of heavy loads and provide the SLP figure for these items; and revise the SLP figure for studs, nuts, washers, and stud tensioners.

Safety Evaluation Summary: Revisions are in accordance with the evaluation guidelines presented in the previously approved submittal to the NRC in October 1984 concerning Control of Heavy Loads. The proposed changes to the procedure maintain restrictions to protect fuel and safe shutdown equipment from a load drop accident. Thus the changes are acceptable and do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-1792-5

Description: Remove part of the anti-water hammer circuit from feedwater isolation valves as part of Unit 1 steam generator replacement. The replacement steam generators do not have a preheater that requires water hammer protection.

Safety Evaluation Summary: The change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-2042-3

Description: Add and clarify notes to design drawings to ensure that the RHR heat exchanger flow control and bypass valves are properly positioned for plant Modes 1, 2, and 3. Revise UFSAR Section 5.4.7.

Safety Evaluation Summary: There is no safety impact because the accident analysis currently takes credit for these valves to be positioned in their emergency core cooling functional position. The change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-2087-9

Description: Revise containment LOCA P/T analysis to reflect reduced CCW flow through RHR heat exchangers during recirculation phase.

Safety Evaluation Summary: Maximum containment pressure remains ≤ 41.2 psig, which is less than the design pressure of 56.5 psig. Post-LOCA containment P/T profiles remain below the EQ limits. The change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-2411-1

Description: Revise procedure to reflect changes in testing fire pump diesel engine batteries; clarify when a continuous fire watch is required for the Plant Relay Room halon system; add a yard fire hydrant; change frequency for FA System supervised circuit testing from every 6 months to every 12 months; and change organization titles.

Safety Evaluation Summary: None of the proposed changes alter the results of any analysis previously performed as a basis in the SAR nor adversely affect the ability to safely achieve and maintain cold shutdown in case of a fire in the plant. The changes do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-2761-4

Description: Temporary modification to the IA and SA systems during the defueled condition for check valve maintenance requiring depressurization of the air systems and installation of two portable air compressors.

Safety Evaluation Summary: This temporary modification complies with the design intent of the UFSAR. There is no unreviewed safety question.

Unreviewed Safety Question Evaluation 99-2763-9

Description: Reload Safety Evaluation and Core Operating Limits Report (COLR) for Unit 1 Cycle 10, Modes 1 through 5 (from 0 to approximately 515 effective full power days of operation).

Safety Evaluation Summary: The limits provided in the Unit 1 Cycle 10 COLR reflect the design and are acceptable. Operation of Unit 1 Cycle 10 to a total cycle burnup (including coastdown) of 515 effective full power days does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-2763-39

Description: Determine impact of incomplete rod insertion on the safety analysis and determine if this condition involves an unreviewed safety question for Unit 1 Cycle 10.

Safety Evaluation Summary: The Safety Analysis provides bounding results with respect to the Reload Safety Analysis Checklist. Failure of the rod cluster control assemblies to fully insert to the rod bottom position does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-2771-9

Description: Reload Safety Evaluation and COLR for Unit 2 Cycle 8, Modes 1 - 5 (from 0 to approximately 498 effective full power days of operation).

Safety Evaluation Summary: The limits provided in the Unit 2 Cycle 8 COLR reflect the design and are therefore also acceptable. Operation of Unit 2 Cycle 8 to a total cycle burnup (including coastdown) of 498 effective full power days does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-2771-37

Description: Determine impact of incomplete rod insertion on the safety analysis and determine if this condition involves an unreviewed safety question for Unit 2 Cycle 8.

Safety Evaluation Summary: The Safety Analysis provides bounding results with respect to the Reload Safety Analysis Checklist. Failure of the rod cluster control assemblies to fully insert to the rod bottom position does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-2771-52

Description: Assess the impact of the fuel design change associated with the use of axial blankets consisting of natural, mid-enriched, and fully-enriched annular pellets or natural or mid-enriched solid pellets in the top and bottom axial regions of the fuel rod.

Safety Evaluation Summary: The use of axial blankets consisting of natural, mid-enriched, or fully-enriched solid or annual pellets in the fuel rods is acceptable and does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-3465-5

Description: Revise the OQAP to reflect an organizational change.

Safety Evaluation Summary: This reflects an organizational change and does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-4451-6

Description: Determine if a spent fuel pool demineralizer can be placed in service empty and/or bypassed.

Safety Evaluation Summary: The purification portion of the Spent Fuel Pool Cooling and Cleanup system is not safety-related and is not required to perform any safety functions. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-6044-1

Description: Revise OQAP to provide for the performance of independent technical reviews since the Independent Safety Engineering Group (ISEG) has been removed from Tech Specs.

Safety Evaluation Summary: This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-8183-2

Description: Revise OQAP to reflect organizational changes.

Safety Evaluation Summary: This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-8183-6

Description: Revise OQAP to reflect organizational changes and to provide guidance regarding correction of electronic records.

Safety Evaluation Summary: This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-9946-1

Description: Extend the EQ mild/harsh radiation threshold dose cut-off limit and revise the design criteria to reflect the new threshold.

Safety Evaluation Summary: Extending the threshold will not alter equipment function or plant configuration. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-9980-1

Description: Change the frequency for fire prevention surveys/inspections in safety-related areas from once/week to once/month, and in non-safety related plant areas and non-plant areas from once/month to once/quarter.

Safety Evaluation Summary: If the current weekly/monthly inspection failure/unsatisfactory rates were extrapolated to monthly/quarterly, they would still be < 0.15%. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-10912-13

Description: Allow replacing irradiated fuel assembly top nozzles only without fuel reconstitution. Delete requirement for mechanical stop on the new fuel elevator structure.

Safety Evaluation Summary: Revised process provides adequate administrative and physical controls to ensure previous safety evaluation conclusions remain satisfied. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-10912-17

Description: Forty fuel assemblies may be susceptible to top nozzle holddown spring screw failures leading to potential deviations from descriptions in the UFSAR.

Safety Evaluation Summary: The loading of 40 suspect assemblies will not 1) result in the generation of loose parts, 2) adversely affect the affected assembly, adjacent assemblies, or internals components, 3) adversely affect control rod insertion characteristics, or 4) increase the consequences of any previous analysis. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-10912-19

Description: Modify the refueling machine gripper mast to allow handling of damaged fuel assemblies and other fuel assemblies which can not be removed prior to the damaged fuel assemblies during core offload.

Safety Evaluation Summary: Additional procedural steps for the modified gripper mast are being incorporated in the fuel handling procedures to ensure proper orientation and alignment and gripper engagement. The proposed change does not affect the structural integrity or functional capability of the refueling machine for safe handling of the fuel assemblies. The proposed change does not impact equipment that is important to safety. There are no changes to accident initiators or radiological consequences. Therefore, the modification does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-11537-1

Description: Revise UFSAR Section 9.5.3.4 to delete the emergency DC lighting annual testing at the lamp head for a specific illumination value.

Safety Evaluation Summary: The Essential AC Lighting system is the primary lighting system used during the safe shutdown and the Emergency DC Lighting system is a backup to the Essential AC Lighting system. The deletion of the test is endorsed by NMAC and NRC under EPRI Guide TR-100249. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-12134-2

Description: Control of Heavy Loads Procedure revisions: allow use of polar crane for maintenance during power operation, allow unrestricted load movement in FHB truck bay, restrict mobile crane load movement over Class 1E ductbank manholes, and add SLP figures for lead shielding boxes and spent fuel storage racks.

Safety Evaluation Summary: Revisions are in accordance with the evaluation guidelines presented in the previously approved submittal to the NRC in October 1984 concerning Control of Heavy Loads. The proposed changes to the procedure maintain restrictions to protect fuel and safe shutdown equipment from a load drop accident. The changes do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-13150-1

Description: Changed setpoints for IVC, ECW, and DGB area temperature monitoring switches from 99°F to 104 °F require changes to the Technical Requirements Manual.

Safety Evaluation Summary: The changes do not increase the consequences of accidents or the probability of malfunction of equipment important to safety. The changes do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-13514-3

Description: Allow onsite land application of sanitary waste sludge containing trace quantities of radioactivity in accordance with applicable state regulations.

Safety Evaluation Summary: Radionuclides may be released to land as long as concentrations remain below effluent release limits and radionuclides in the soil do not build up to exceed the limits specified in Title 25 of the Texas Administrative Code Section 289.202. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-13895-12

Description: Change intrusive SDG maintenance from every 18 months to every 5 years in the Technical Requirements Manual.

Safety Evaluation Summary: Removing intrusive inspections that could increase failure mechanisms increases SDG reliability. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-15971-1

Description: Add a temporary decontamination facility for steam generator replacement.

Safety Evaluation Summary: There is no interference with systems, structures, or components important to safety. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-15627-4

Description: Permanently abandon two Unit 2 core exit thermocouples (CETs) in place and seal their instrument tubing (pressure boundary) with approved fittings.

Safety Evaluation Summary: The remaining 48 Unit 2 CETs meet the minimum requirements for monitoring inadequate core cooling as required by Technical Specifications and Bases. The broken CETs do not create loose parts concerns, and the pressure boundary seal is equal to the previous seal. Therefore, this does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 99-17651-2

Description: Revise UFSAR 12.5.3.6 to require processing thermoluminescent dosimeters (TLD) periodically rather than quarterly.

Safety Evaluation Summary: TLDs, TLD processing, and the time interval for processing were determined to have no impact on an accident analysis, plant system, structure, component, subsystem, safety feature or safety function. This does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 00-266-5

Description: Increase maximum time allowed to initiate operator-controlled cooldown for steam generator tube rupture to 40 minutes. (Safety analysis assumes cooldown is initiated 16 minutes and 29 minutes after tube rupture for margin to overfill and off-site dose cases, respectively.) Increase maximum time allowed to initiate RCS depressurization after completion cooldown from 3 minutes to 15 minutes. (Safety analysis assumes RCS depressurization initiates 3 minutes after completion of cooldown.)

Safety Evaluation Summary: Changes reflect observed operator action times. Acceptance limits for dose and steam generator overfill are not exceeded. The changes do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 00-1675-2

Description: Revise UFSAR Section 9.3.4.1.2.5 to resolve confusion on using P/T conditions and packless metal diaphragm valves in CVCS.

Safety Evaluation Summary: Using valves with design features that provide reliability, good performance, and meet system service conditions will minimize leakage and support licensing/design basis. The change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 00-1966-2

Description: Revise OQAP to reflect organization changes and clarify applicable inspection procedures for training in-line personnel performing inspections.

Safety Evaluation Summary: These changes do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 00-1966-5

Description: Revise OQAP to reflect organization change.

Safety Evaluation Summary: These changes do not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 00-3229-6

Description: Remove automatic loading of centrifugal charging pump from the SDG load sequencer.

Safety Evaluation Summary: The change makes the plant consistent with UFSAR Chapter 15 safety analyses. The existing dose remains bounding. There is a negligible change from the baseline core damage frequency. The change does not involve an unreviewed safety question.

Unreviewed Safety Question Evaluation 00-13212-2

Description: Allow make-up to RCS or boration/dilution of RCS without using RC makeup control switch and associated automatic controls in Modes 1-3.

Safety Evaluation Summary: Procedure will use same sources, equipment, and flow paths as normal procedure, and provides appropriate controls to limit quantity/quality of makeup water. The change does not involve an unreviewed safety question.