

January 25, 2001

Mr. J. Sorensen  
Site General Manager  
Prairie Island Nuclear Generating Plant  
Nuclear Management Company, LLC  
1717 Wakonade Drive East  
Welch, MN 55089

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT - NOTIFICATION OF AN  
NRC TRIENNIAL FIRE PROTECTION BASELINE INSPECTION 50-282/01-05;  
50-306/01-05

Dear Mr. Sorenson:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC), Region III staff will conduct a triennial fire protection baseline inspection at the Prairie Island Nuclear Generating Plant in April and May 2001. The inspection will be lead by a senior reactor engineer from the NRC Region III Office and will be composed of personnel from NRC Region III. The inspection will be conducted in accordance with IP 71111.05, the NRC's baseline fire protection inspection procedure.

The schedule for the inspection is as follows:

- Information gathering visit - April 10 - 12, 2001
- Days of onsite inspection - April 23 - 27 and May 7 - 11, 2001

The purpose of the information gathering visit is to obtain information and documentation needed to support the inspection, to become familiar with the Prairie Nuclear Generating Plant fire protection programs, fire protection features, and post-fire safe shutdown capabilities and plant layout; and, as necessary, obtain plant specific site access training and badging for unescorted site access. A list of the types of documents the team may be interested in reviewing, and possibly obtaining, are listed in the Enclosure to this letter.

During the information gathering visit, the team will also discuss the following inspection support administrative details: office space size and location; specific documents requested to be made available to the team in their office spaces; arrangements for reactor site access (including radiation protection training, security, and safety); and the availability of knowledgeable plant engineering and licensing organization personnel to serve as points of contact during the inspection.

We request that during the onsite inspection weeks you ensure that copies of analyses, evaluations or documentation regarding the implementation and maintenance of the Prairie Island Nuclear Generating Plant fire protection program, including post-fire safe shutdown capability, be readily accessible to the team for their review. Of specific interest are those documents which establish that your fire protection program satisfies NRC regulatory

requirements and conforms to applicable NRC and industry fire protection guidance. Also, appropriate personnel, knowledgeable with respect to those plant systems required to achieve and maintain safe shutdown conditions from inside and outside the control room (including the electrical aspects of the relevant post-fire safe shutdown analyses), reactor plant fire protection systems, and the Prairie Island Nuclear Generating Plant fire protection program and its implementation, should be available at the site during the inspection.

Your cooperation and support during this inspection will be appreciated. If you have questions concerning this inspection, or the inspection team's information or logistical needs, please contact Mr. Ronald A. Langstaff at (630) 829-9747 or myself at (630) 829-9751.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

Sincerely,

\RA\

Ronald N. Gardner, Chief  
Electrical Engineering Branch  
Division of Reactor Safety

Docket Nos. 50-282; 50-306  
License Nos. DPR-42; DPR-60

Enclosure: Reactor Fire Protection Program Supporting Documentation

cc w/encl: Plant Manager, Prairie Island  
M. Wadley, Chief Nuclear Officer  
G. Eckholt, Site Licensing Manager  
S. Northard, Nuclear Asset Manager  
J. Malcolm, Commissioner, Minnesota  
Department of Health  
State Liaison Officer, State of Wisconsin  
Tribal Council, Prairie Island Dakota Community  
J. Silberg, Esquire  
Shawn, Pittman, Potts, and Trowbridge  
P. Tester, Assistant Attorney General  
Minnesota Office of Attorney General  
S. Bloom, Administrator  
Goodhue County Courthouse  
Commissioner, Minnesota Department  
Of Commerce

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## **Reactor Fire Protection Program Supporting Documentation**

This is a broad list of the documents the NRC inspection team may be interested in reviewing, and possibly obtaining, to support the inspection. The lead inspector will discuss specific information needs with the licensee staff and may request additional documents.

### Prior to the Information Gathering Visit

1. The reactor plant's Individual Plant Examination for External Events (IPEEE), results of any post-IPEEE reviews, and listings of actions taken/plant modifications conducted in response to IPEEE information.

### During the Information Gathering Visit

1. The current version of the Fire Protection Program and Fire Hazards Analysis.
2. Listing of plant fire protection licensing basis documents.
3. The NRC Safety Evaluation Reports (SERs) and actual copies of the 50.59 reviews which form the licensing basis for the reactor plant's post-fire safe shutdown configuration.
4. Listing of Generic Letter 86-10 evaluations.
5. National Fire Protection Association (NFPA) code versions committed to (NFPA codes of record).
6. Listing of plant deviations from code commitments.
7. A list of applicable codes and standards related to the design of plant fire protection features and evaluations of code deviations.
8. Post-fire safe shutdown systems and separation analysis.
9. Post-fire alternative shutdown analysis.
10. Piping and instrumentation (flow) diagrams showing the components used to achieve and maintain hot standby and cold shutdown for fires outside the control room and those components used for those areas requiring alternative shutdown capability.
11. Plant layout and equipment drawings which identify the physical plant locations of hot standby and cold shutdown equipment.
12. Plant layout drawings which identify plant fire area delineation, areas protected by automatic fire suppression and detection, and the locations of fire protection equipment.
13. Current versions of the fire protection program implementing procedures (e.g., administrative controls, surveillance testing, fire brigade).

14. Organization charts of site personnel down to the level of fire protection staff personnel.
15. A listing of abbreviations/designators for plant systems.
16. Listing of open and closed fire protection condition reports (problem identification forms and their resolution reports).
17. Listing of fire impairments for previous year.
18. Pre-fire plans for fire areas/zones selected by the inspectors for review.
19. Plant operating procedures which would be used and describe shutdown for a postulated fire in the fire areas/zones selected by the inspectors.

#### During the Inspection

1. Fire brigade training program.
2. Operator training for shutdown procedures in the event of fire.
3. Plant layout drawings which identify the general location of the post-fire emergency lighting units.
4. Maintenance and surveillance testing procedures for alternative shutdown capability and fire barriers, detectors, pumps and suppression systems.
5. Maintenance procedures which routinely verify fuse breaker coordination in accordance with the post-fire safe shutdown coordination analysis.
6. Procedures/instructions that control the configuration of the reactor plant's fire protection program, features, and post-fire safe shutdown methodology and system design.
7. Procedures/instructions that govern the implementation of plant modifications, maintenance, and special operations, and their impact on fire protection.
8. Significant fire protection and post-fire safe shutdown related design change package descriptions (including their associated 10 CFR Part 50.59 evaluations) and Generic Letter (GL) 86-10 evaluations.
9. Temporary modification procedures.
10. The three most recent fire protection Quality Assurance (QA) audits and/or fire protection self-assessments.
11. Recent QA surveillances of fire protection activities.